

November 8, 1982

SBN-356
T. F. B7.1.2

United States Nuclear Regulatory Commission
Washington, D. C. 20555

Attention: Mr. George W. Knighton, Chief
Licensing Branch 3
Division of Licensing

- References:
- (a) Construction Permits CPPR-135 and CPPR-136, Docket Nos. 50-443 and 50-444
 - (b) USNRC Letter, dated February 12, 1982, "Request for Additional Information," F. J. Miraglia to W. C. Tallman
 - (c) PSNH Letter, dated March 12, 1982, "Response to 410 Series RAIs; (Auxiliary Systems Branch)," J. DeVincentis to F. J. Miraglia
 - (d) PSNH Letter, dated July 12, 1982, "Amendment 45 to March 30, 1973, Application to Construct and Operate Seabrook Station Unit 1 and Unit 2; Incorporation of Requests for Additional Information (RAIs)"
 - (e) PSNH Letter, dated September 24, 1982, "NUREG-0612; Control of Heavy Loads," J. DeVincentis to J. B. Kerrigan

Subject: Revised Response to RAI 410.21; (Auxiliary Systems Branch)

Dear Sir:

We have enclosed a revised response to the subject Request for Additional Information (RAI) which was forwarded in Reference (b).


The original response to this RAI was submitted in Reference (c) and subsequently incorporated into the FSAR [OL Application Amendment 45; Reference (d)].

As indicated in the enclosed response to RAI 410.21 the report addressing NUREG-0612, "Control of Heavy Loads," was submitted in Reference (e).

The enclosed response to RAI 410.21 will be included in FSAR Amendment 48.

Very truly yours,

YANKEE ATOMIC ELECTRIC COMPANY


J. DeVincentis
Project Manager

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NRC Question 410.21 (9.1.5)

- (1) Commit to implement the interim actions of NUREG-0612, "Control of Heavy Loads at Nuclear Power Plants" prior to receipt of your operating license.
- (2) Provide the results of an analysis of the effects of dropping heavy loads other than the spent fuel cask. The analysis should satisfy the evaluation criteria of NUREG-0612, Section 5.1, and consider the consequences of dropping the reactor vessel head and vessel internals during preparation for or completion of fuel handling. In addition, the lower load block of both the containment building polar gantry crane and the fuel building cask handling crane should be considered as a heavy load and an analysis of the consequences of their falling included in this analysis.

RESPONSE

- (1) As discussed at the meeting between the NRC, PSNH, YAEC, and UE&C on July 15, 1982 at the site, this requirement does not apply to Seabrook Station since the plant is not yet operational.
- (2) In addition to the spent fuel cask, the reactor vessel head, vessel internals, load block of the polar gantry crane and load block for the cask handling crane have been considered as heavy loads along with other loads in our detailed report entitled "Response to NRC Generic Request for Additional Information Relating to NUREG-0612 Control of Heavy Loads," and submitted to the NRC in September 1982*. The treatment of the reactor vessel head and upper internals is, however, not complete at this time since the generic analysis of the RV head lifting rig, internals lifting rig, associated load cell and linkage assemblies, and interfacing lift points being performed by Westinghouse is not yet available. The results of this analysis as well as an overall assessment of the RV head and the internals will be included in Appendix I of this report, and provided to the NRC at a later date. The load blocks of the polar and the cask handling cranes have been dealt with in this report as per the applicable requirements of NUREG-0612 and the NRC letter dated December 22, 1980.

* PSNH Letter, dated September 24, 1982,
"NUREG-0612; Control of Heavy
Loads", J. DeVincentis to
J.B. Kerrigan