

UNITED STATES NUCLEAR REGULATORY COMMISSION WASHINGTON, D. C. 20555

TENNESSEE VALLEY AUTHORITY

DOCKET NO. 50-259

BROWNS FERRY NUCLEAR PLANT, UNIT 1

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 177 License No. DPR-33

- 1. The Nuclear Regulatory Commission (the Commission) has found that:
 - A. The application for amendment by Tennessee Valley Authority (the licensee) dated August 6, 1990, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations set forth in 10 CFR Chapter I;
 - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
 - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
 - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
 - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.

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- Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment and paragraph 2.C.(2) of Facility Operating License No. DPR-33 is hereby amended to read as follows:
 - (2) Technical Specifications

The Technical Specifications contained in Appendices A and E, as revised through Amendment No. 177, are hereby incorporated in the license. The licensee shall operate the facility in accordance with the Technical Specifications.

3. This license amendment is effective as of its date of issuance and shall be implemented within 30 days from the date of issuance.

FOR THE NUCLEAR REGULATORY COMMISSION

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Frederick J. Hebdon, Director Project Directorate II-4 Division of Reactor Projects - I/II Office of Nuclear Reactor Regulation

Attachment: Changes to the Technical Specifications

Date of Issuance: December 31, 1990

ATTACHMENT TO LICENSE AMENDMENT NO. 177

FACILITY OPERATING LICENSE NO. DPR-33

DOCKET NO. 50-259

Revise the Appendix A Technical Spacifications by removing the pages identified below and inserting the enclosed pages. The revised pages are identified by the captioned amendment number and contain marginal lines indicating the area of change. Overleaf pages* are provided to maintain document completeness.

REMOVE	INSERT
3.2/4.2-7	3.2/4.2-7
3.2/4.2-8	3.2/4.2-8*
3.2/4.2-12	3.2/4.2-12
3.2/4.2-13	3.2/4.2-13*

TABLE 3.2.A PRIMARY CONTAINMENT AND REACTOR BUILDING ISOLATION INSTRUMENTATION

BFN Unit

Mini#um No. Instrument Channels One

provide a		

Per Trip Sys(1)(11)	Function	Trip Level Setting	Action (1)	Remarks
2	Instrument Channel – Reactor Low Water Level(6)	≥ 538" above vessel zero	A or (8 and E)	 Below trip setting does the following: a. Initiates Reactor Building Isolation b. Initiates Primary Containment Isolation (Groups 2, 3, and 6) c. Initiates SGTS
1	Instrument Channel – Reactor High Pressure (PS-68-93 and 94)	100 ± 15 psig	D	 Above trip setting isolates the shutdown cooling suction valves of the RHR system.
2	Instrument Channel - Reactor Low Water Level (LIS-3-56A-D, SW #1)	≥ 378" above vessel zero	А	 Below trip setting initiate Main Steam Line Isolation
2	Instrument Channel – High Drywell Pressure (6) (PS-64-56A-D)	⊻2.5 psig	A or (B and E)	 Above trip setting does the following: Initiates Reactor Building Isolation Initiates Primary Containment Isolation Initiates SGTS

TABLE 3.2.A (Continued) PRIMARY CONTAINMENT AND REACTOR BUILDING ISOLATION INSTRUMENTATION

BFN Unit

3.2/4.2-8

Minimum No. Instrument

211.9.2.2.2	OmiG11	6
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Per Trip Sys(1)(11)	Function	Trip Level Setting	Action (1)	Remarks
Z	Instrument Channel - High Radiation Main Steam Line Tunnel (6)	3 times normal rated full power background (13)	В	 Above trip setting initiates Main Steam Line Isolation
2	Instrument Channel - Low Pressure Main Steam Line	≥ 825 psig (4)	В	 Below trip setting initiates Main Steam Line Isolation
2(3)	Instrument Channel - High Flow Main Steam Line	≤ 140% of rated steam flow	B	 Above trip setting initiates Main Steam Line Isolation
2(12)	Instrument Channel - Main Steam Line Tunnel High Temperature	≤ 200°F	B	 Above trip setting initiates Main Steam Line Isolation.
2(14)	Instrument Channel - Reactor Water Cleanup System Floor Drain High Temperature	160 - 180°F	С	 Above trip setting initiates Isolation of Reactor Water Cleanup Line from Reactor and Reactor Water Return Line.
2	Instrument Channel – Reactor Water Cleanup System Space High Temperature	160 - 180°F	С	1. Same as above
1	Instrument Channel - Reactor Building Ventilation High Radiation - Reactor Zone	≤ 100 mr/hr or downscale	G	 1 upscale or 2 downscale will a. Initiate SGTS b. Isolate reactor zone and refueling floor. c. Close atmosphere control system.

NOTES FOR TABLE 3.2.A

- 1. Whenever the respective functions are required to be OPERABLE there shall be two OPERABLE or tripped trip systems for each function. If the first column cannot be met for one of the trip systems, that trip system or logic for that function shall be tripped (or the appropriate action listed below shall be taken). If the column cannot be met for all trip systems, the appropriate action listed below shall be taken.
 - A. Initiate an orderly shutdown and have the reactor in Cold Shutdown in 24 hours.
 - B. Initiate an orderly load reduction and have Main Steam Lines isolated within eight hours.
 - C. Isolate Reactor Water Cleanup System.
 - D. Administratively control the affected system isolation valves in the closed position within one hour and then deplace the affected system inoperable.
 - E. initiate primary containment isolation within 24 yours.
 - F. The handling of spent fuel will be prohibited and all operations over spent fuels and open reactor wells shall be prohibited.
 - G. Isolate the reactor building and start the standby gas treatment system.
 - H. Immediately perform a logic system functional test on the logic in the other trip systems and daily thereafter not to exceed 7 days.
 - I. Deleted
 - J. Withdraw TIP.
 - K. Manually isolate the affected lines. Refer to Section 4.2.E for the requirements of an inoperable system.
 - L. If one SGTS train is inoperable take actions H or A and F. If two SGTS trains are inoperable take actions A and F.
- 2. When it is determined that a channel is failed in the unsafe condition, the other channels that monitor the same variable shall be functionally tested immediately before the trip system or logic for that function is tripped. The trip system or the logic for that function may remain untripped for short periods of time to allow functional testing of the other trip system or logic for that function.
- There are four sensors per steam line of which at least one sensor per trip system must be OPERABLE.

NOTES FOR TABLE 3.2.A (Cont'd)

- 4. On required in RUN MODE (interlocked with Mode Switch).
- 5. Not required in RUM MODE (bypassed by Mode Switch).
- Channel shared by RPS and Primary Containment & Reactor Vessel Isolation Control System. A channel failure may be a channel failure in each system.
- 7. A train is considered a trip system.
- 8. Two out of three SGTS trains required. A failure of more than one will require actions A and F.
- 9. Deleted
- 10. Refer to Table 3.7.A and its notes for a listing of Isolation Valve Groups and their initiating signals.
- 11. A channel may be placed in an inoperable status for up to four hours for required surveillance without placing the trip system in the tripped condition provided at least one OPERABLE channel in the same trip system is monitoring that parameter.
- 12. A channel contains four sensors, all of which must be OPERABLE for the channel to be OPERABLE.

Power operations permitted for up to 30 days with 15 of the 16 temperature switches OPERABLE.

In the event that normal ventilation is unavailable in the main steam line tunnel, the high temperature channels may be bypassed for a period of not to exceed four hours. During periods when normal ventilation is not available, such as during the performance of secondary containment leak rate tests, the control room indicators of the affected space temperatures shall be monitored for indications of small steam leaks. In the event of rapid increases in temperature (indicative of steam line break), the operator shall promptly close the main steam line isolation valves.

- 13. The nominal setpoints for alarm and reactor trip (1.5 and 3.0 times background, respectively) are established based on the normal background at full power. The allowable setpoints for alarm and reactor trip are 1.2-1.8 and 2.4-3.6 times background, respectively.
- 14. Requires two independent channels from each physical location; there are two locations.