

U.S. NUCLEAR REGULATORY COMMISSION
REGION I

Report No. 50-286/82-17

Docket No. 50-286

License No. DPR-64 Priority -- Category C

Licensee: Power Authority of the State of New York
10 Columbus Circle
New York, New York 10019

Facility Name: Indian Point Nuclear Generating Station, Unit 3

Inspection at: Buchanan, New York

Inspection Conducted: September 16, 1982 to October 15, 1982

Inspectors:

<u>T. J. Kenny</u> T. J. Kenny, Resident Inspector	<u>10/15/82</u> date
<u>W. Baunack</u> W. Baunack, Regional Inspector	<u>10/18/82</u> date

Approved by:

<u>H. Kister</u> H. Kister, Chief, Reactor Project Section 1C, DPRP	<u>10/19/82</u> date
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Inspection Summary:

Inspections on September 16, 1982 to October 15, 1982 (Inspection Report 50-286/82-17)

Areas Inspected: Routine onsite regular and backshift inspections of plant operations including shift logs and records; plant tour; surveillance; maintenance; review of TMI action plan requirements; and, review of monthly operating report. The inspection involved 104 inspector hours by the resident and regional inspectors.

Results: Of the five areas inspected, no violations were identified.

DETAILS

1. Persons Contacted

M. Albright, Instrument and Control Superintendent
R. Allen, Training Superintendent
J. Brons, Resident Manager
J. Dube, Security and Safety Supervisor
D. Halama, Q.A. Superintendent
W. Hamlin, Assistant to the Resident Manager
W. Josiger, Superintendent of Power
J. Perrotta, Radiological and Environmental Services Superintendent
S. Munoz, Technical Services Supervisor
E. Tagliamonti, Operations Superintendent
J. Vignola, Maintenance Superintendent

The inspector also interviewed and observed other licensee employees including members of the operations, health physics, technical services, maintenance, and security staffs.

2. Licensee Actions on Previous Inspection Findings

(Closed) Unresolved Item (50-286/78-20-02) Refueling cranes inspection deficiencies. By a review of procedures 3-PM-V-CM-2, Polar Crane Inspection, and 3-PM-R-CM-1, Crane Outage Inspection, and certification papers for the refueling cranes, the inspector verified that any previous deficiencies have been corrected, and the current inspections are deficiency free.

(Closed) Unresolved Item (50-286/78-20-04) Verification of snubber piston positions. The inspector reviewed 3PT-V1B, Revision 3, "Accessible Snubber Inspection", and verified that the current method of visual inspections documents that the snubber is positioned properly to function, if needed.

(Closed) Unresolved Item (50-286/78-23-08) Isolation Valve Seal Water System (IVSWS), as a Technical Specification item. The inspector's concern was that of hydraulically testing the containment valves for "B&C" testing utilizing the IVSWS. The practice of using the IVSWS continues at this time, but not with hydraulics; nitrogen pressure is now used to test the containment valves, which allows testing of a bank of valves utilizing the same method as the testing rigs equipped with calibrated gauges. The original use of the IVSWS remains unchanged for plant operation, therefore, Technical Specifications should not be changed.

(Closed) Unresolved Item (50-286/78-23-09) Pipe hanger deficiencies. The inspector had noted missing hangers in the rad monitoring system for R11 and R12. A recent tour of containment, and, in particular, the tracing of these systems, by the inspector, noted no deficiencies in these systems.

(Closed) Inspector Follow Item (50-286/82-14-01) Additional training for staff licensed personnel. The inspector reviewed recent changes to Training Procedure 15-5, "Licensed Operator Requalification", which delineates the type of retraining staff licensed personnel will receive. The concerns of the inspector have been satisfied by this revision.

3. Plant Tour

A. Normal and backshift inspections were conducted during routine entries into the protected area of the plant, including the control room, PAB, fuel building, and containment. During the inspection activities, discussions were held with operators, technicians (HP & I&C), mechanics, foremen, supervisors, and plant management. The purpose of the inspection was to affirm the licensee's commitments and compliance with 10 CFR, Technical Specifications, and Administrative Procedures. Particular attention was directed in the following areas:

- Instrumentation and recorder traces for abnormalities;
- Proper control room and shift manning;
- Proper use of procedures;
- Review of logs to obtain plant conditions;
- Verification of proper radiologically controlled areas and access points;
- Verification of surveillance testing for timely completion;
- Verification of safety related tagouts;
- Plant housekeeping and cleanliness;
- That protected area access controls were in conformance with the security plan, including sufficient guards to perform duties, and that selected gates and doors were closed and locked;
- Selected liquid and gaseous samples to verify conformance with regulatory requirements prior to release; and,
- Boric acid samples to confirm proper boric acid level for plant shutdown conditions.

B. During the inspection, the inspector reviewed the following procedures, documents, or evolutions:

- Radioactive Waste Release Permit (liquid & gaseous)
- Various shift turnover checklists
- Security Station Logs and Radio Checks
- Jumper Log
- Selected Operators' Logs
- Selected Tagouts
- Selected Radiation Exposure Authorizations (REA's)

No violations were identified.

4. Surveillance

The inspector either directly observed the performance of or reviewed completed surveillance procedures to ascertain the following:

- That the instrumentation used was properly calibrated;
- That the redundant system or component was operable where required;
- That properly approved procedures were used by qualified personnel;
- That the acceptance criteria were met;
- That the test data were accurate and complete;
- That proper reviews, by the licensee, had been conducted;
- That the results of the tests met Technical Specification requirements; and,
- That the testing was done within the required surveillance schedule.

The inspector reviewed the following tests:

- 3PT-W01 Diesel Generator
- 3PT-M27 Station Air
- 3PT-M40 Emergency Locker Equipment
- 3PT-M49 Fire Protection Diesel Battery Inspection
- 3PT-Q30 Hydrogen Recombiner VC Isolation Valves

No violations were identified.

5. Maintenance

The inspector selected completed maintenance activities listed below to ascertain the following:

- The activities did not violate a limiting condition for operation;
- That redundant components were operable;

- That equipment was tagged out in accordance with licensee approved procedures;
- That approved procedures, adequate to control the activity, were being used by qualified technicians;
- That Q/C hold points were observed, and that materials were properly certified;
- That radiological controls were proper and in accordance with licensee approved radiation exposure authorizations; and,
- That the equipment was properly tested prior to return to service.

1) Repair of Damper on #32 Fan Cooler Unit

Documents Reviewed:

- Work Request 3015
- Maintenance Checklist

2) Repair of Circuit #33 on Line 225 Boric Acid Heat Tracing

Documents Reviewed:

- Work Request 3022
- Maintenance Checklist
- Certification for Butt Splice, used to make repairs (D-690 and E-316)

3) Repair of Fuel Storage Building Crane

Documents Reviewed:

- Work Request 3047
- Maintenance Checklist
- Certification papers for replacement contactors (B-115)

No violations were identified.

6. Review of TMI Action Plan Category B Requirements

A review was conducted of the licensee's programs in the areas of upgrading of reactor and senior reactor operator training and qualifications (NUREG 0737, Item 1.A.2.1), and training for mitigating core damage (NUREG 0737, Item II.B.4).

The following records were reviewed:

- Indoctrination and Training Procedure No. 15-5, Licensed Operator Requalification, Revision 10, March 4, 1982;

- 1982-84 Licensed Operators/S.T.A. Retraining Schedule, Revision 2;
- Selected Records of Licensed Retraining; and,
- Selected Records of Simulator Training.

Findings:

The formal licensed operator requalification program is intended to consist of one week of training out of each six weeks of work. This includes two weeks of simulator training (80 hours), and the remainder classroom training (32 hours/week) including examinations. Training schedules are adjusted to accommodate refuelings, holidays, vacations, etc.

Adequate records are maintained to show that each individual has received the required training.

The licensed operator retraining program consists of classroom training in the following categories:

- Theory and Principles of Operation
- Plant Specific Operating Characteristics
- Plant Instrumentation and Control Systems
- Plant Protection Systems
- Engineered Safety Systems
- Normal, Abnormal, Emergency and Special Operating Procedures
- Applicable Portions of Title 10 CFR
- Mitigating Reactor Core Damage
- Technical Specifications and Administrative Procedures
- Changes in Equipment and Operating Procedures
- Radiation Control, Safety and Radiation Protection Procedures
- Facility Design and License Changes
- Emergency Plan and Procedures
- Fuel Handling Equipment and Techniques
- Theory and Principles of Thermodynamics and Fluid Mechanics

The category, Theory and Principles of Thermodynamics and Fluid Mechanics, which consists of 32 hours classroom work was verified to include all training required by NUREG 0737 in the areas of heat transfer, fluid flow, and thermodynamics.

The category, Mitigating Reactor Core Damage, also consists of 32 hours classroom training, and is taught from a General Physics Corporation Prepared Manual. This manual was verified to contain all NUREG 0737 criteria for mitigating core damage.

The portions of the retraining program specifically devoted to heat transfer, fluid flow, thermodynamics, and mitigating core damage consist of 64 hours of classroom training. In addition, related subjects are taught, portions of which deal with mitigating core damage. These are plant specific operating characteristics, plant instrumentation and controls, plant protection systems, and normal, abnormal and special operating procedures. Also, portions of the annual 80 hours of simulator training deal with mitigating core damage. No specific number of hours can be assigned to the portions of simulator training and related subjects which are devoted to mitigating core damage; however, the total retraining associated with this subject is in excess of 80 hours.

All operating personnel and shift technical advisors have received the training for mitigating core damage through their participation in the retraining program with the exception of the resident manager. A discussion with the resident manager, and a review of documentation, shows that the resident manager received equivalent training while participating in a Westinghouse Operations Training Program from August 4, 1980 to November 25, 1980.

To ensure that non-licensed positions in the operational chain receive the required mitigating core damage training, the licensee has included the required mitigating core damage training, as part of the position description.

No violations were identified.

7. Review of Monthly Report

A. Monthly Operating Report

The Monthly Operating Report for August, 1982 was reviewed. The review included an examination of selected maintenance work requests, and an examination of significant occurrence reports to ascertain that the summary of operating experience was properly documented.

B. Findings:

The inspector verified through record reviews and observations of maintenance in progress that:

- The corrective action was adequate for resolution of the identified items; and,
- The Operating Report included the requirements of TS 6.9.1.6.

The inspector has no further questions relating to the report.

8. Exit Interview

At periodic intervals during the course of the inspection, meetings were held with senior facility management to discuss the inspection scope and findings. An additional exit interview was held on October 15, 1982 to summarize inspection findings, and to discuss plant status and current inspector concerns.