

LER SUPPLEMENTAL INFORMATION

BFRO-50-259 / 82056 R1 Technical Specification Involved 3.6.G

Reported Under Technical Specification 6.7.2.a.(3) * Date Due NRC 11/05/82

Event Narrative:

Unit 1 and 3 were shut down for secondary containment testing and maintenance activities; unit 2 was shut down for a refueling outage. Only unit 1 was affected by this event. During an inspection of the drywell for leakage by maintenance personnel, a reactor coolant leak was found (T.S. 3.6.G). The leak occurred at a cracked pipe-to-elbow weld in a defunct 1-inch pressure differential indicating switch sensing line to a jet pump riser. The switch had been removed previously and the line was blanked. Repair was made by replacing the elbow and leak testing. There was no effect on public health and safety. Leakage was less than that allowed by T.S. 3.6.C.1 during previous unit operation. The removed elbow and cracked weld were analyzed and it was determined that the crack was due to fatigue caused by vibration of the piping. An inspection of all other PDIS lines was made on the unit and no cracks were found. We consider this a random failure and plan no further corrective action at this time.

* Previous Similar Events:

BFRO-50-296/78019

Responsible Period - Lifetime; Responsibility - Document Control Supervisor

Signature J.R.L.