U.S. NUCLAR RESULATORY COMMISSION ¥ UPDATE REPORT LICENSEE EVENT REPORT PREVIOUS REPORT DATE 8/23/82 CONTROL BLOCK: \square (1) (PLEASE PRINT OR TYPE ALL REQUIRED INFORMATION) 1 3 0 - 0 0 0 0 0 - 0 0 3 4 1 1 1 1 1 57 CAT 58 3 1 A L B R F LICENSEE CODE N'T REPORT 1 SOURCE EVENT DESCRIPTION AND PROBABLE CONSEQUENCES (10) 2] During a unit shutdown for secondary containment testing and maintenance, an 57 inspection in the drywell revealed a crack in an elbow weld in a defunct 1-inch 4 pressure differential indicating switch line on a jet pump riser. (T.S. 3.6.G) There was no effect on public health and safety. The reactor coolant leakage 5 during prior unit operation did not exceed values allowed by Technical 3 Specification 3.6.C.1. 7 SYSTEM CODE 80 CAUSE CAUSE COMP CODE VALVE SUBCODE SUBCODE COMPONENT CODE SUBCODE 3 X 1(12 I F GI Z (13) I P E X X (11) P A 1(15 Z (16) 19 SEQUENTIAL OCCURRENCE REPORT LER HO EVENT YEAR REVISION REPORT NO CODE TYPE REPORT 8 12 1:0. 0 5 6 0 | 1 NUMBER Х 11 28 ACTION FUTUPE EFFECT ON PLANT SHUTDOWN METHOD ATTACHMENT SUBMITTED NPRD-4 PRIME COMP HOURS (22) COMPONENT FORM SUB SUPPLIER (18) X (19 MANUFACTURER A Z Z (2 N 24 0 0 01 L 0 Y X | 9 | 9 | 9 4.0 CAUSE DESCRIPTION AND CORRECTIVE ACTIONS (27) Root cause of the weld crack has been determined to be fatigue caused by vibration of the piping. An inspection of other similar lines on unit 1 was made and no cracks found. We consider this a random failure and plan no further corrective action at this time. FACILITY 80 METHOD OF * POWER 30) OTHER STATUS NICOVERY DISCOVERY DESCRIPTION Glas 0 0 0 0 NA A (31 Maintenance personnel observation ACTIVITY CONTENT HJ. RELEASED_ FRELEASE AMOUNT OF ACTIVITY (55) LOCATION OF HELEASE (36) Z (34) Z (34) NA NA PERSONALL EXPROVIES 44 80 1.1.2 1.1.5 11 TYPE DESCRIPTION (39) 0 0 0 0 0 2 (NA PERFORMET PORT 80 TSCHIPTION (41 \$12. ***** 18 0 01 01 01 NA STORE OF STRAIGE TO FACILITY (1) 80 L'OPINTON. Z NA 8211090418 821102 PUPLICITY PDR DESCRIPTION (1) NIG ADOCK 05000259 NAC USE DULY PDR 1.8 33 MANE OF PHERAPER ____ R. W. Carrigan (205) 729-0733

- Tennessee Valley Authority Browns Ferry Nuclear Plant

Form BF 17 BF 15.2 2/12/82

LER SUPPLEMENTAL INFORMATION

BFRO-50- 259 / 82056 R1 Technical Specification Involved 3.6.G

Reported Under Technical Specification 6.7.2.a.(3) * Date Due NRC 11/05/82

Evant Narrative:

Unit 1 and 3 were shut down for secondary containment testing and maintenance activities; unit 2 was shut down for a refueling outage. Only unit 1 was affected by this event. During an inspection of the drywell for leakage by maintenance personnel, a reactor coolant leak was found (T.S. 3.6.G). The leak occurred at a cracked pipe-to-elbow weld in a defunct 1-inch pressure differential indicating switch sensing line to a jet pump riser. The switch had been removed previously and the line was blanked. Repair was made by replacing the elbow and leak testing. There was no effect on public health and safety. Leakage was less than that allowed by T.S. 3.6.C.1 during previous unit operation. The removed elbow and cracked weld were analyzed and it was determined that the crack was due to fatigue caused by vibration of the piping. An inspection of all other PDIS lines was made on the unit and no cracks were found. We consider this a random failure and plan no further corrective action at this time.

Previous Similar Econts

BFR0-50-296/78019

Recention: Feriod - Lifetimer Responsibility - Document Control Supervisor the failer $\frac{R}{R}$