

APPENDIX

U. S. NUCLEAR REGULATORY COMMISSION  
REGION IV

NRC Inspection Report: STN 50-482/82-12

Docket: STN 50-482

Category A2

Licensee: Kansas Gas and Electric Company  
Post Office Box 208  
Wichita, Kansas 67201

Facility Name: Wolf Creek, Unit 1

Inspection at: Wolf Creek Site, Coffey County, Burlington, Kansas

Inspection Conducted: August 30 - September 3, 1982

Inspector: Claude E. Johnson 10-15-82  
C. E. Johnson, Reactor Inspector, Date  
Engineering Section

Approved: D. M. Hunnicutt 10/18/82  
D. M. Hunnicutt, Engineering Section Date

Inspection Summary

Inspection during August 30 - September 3, 1982 (Report STN 50-482/82-12)

Areas Inspected: Routine, unannounced inspection of construction activities including a site tour and observation of work and review of records and procedures for safety-related pipe supports, spent fuel storage racks, and followup on previous identified inspection findings. The inspection involved 32 inspector-hours on site by one NRC inspector.

Results: No violations or deviations were identified.

DETAILS1. Persons ContactedPrincipal Licensee Personnel

- \*D. W. Prigel, QA Manager, Site
- \*O. L. Thero, QA Surveillance Supervisor
- \*C. E. Parry, QA Systems Supervisor
- \*J. L. Stokes, Project Support Supervisor
- \*P. M. Burck, QA Engineer
- \*D. A. Colwell, QA Technologist
- M. Lindsay, Senior Engineer, Quality Systems
- D. Holloway, Construction Supervisor
- K. Land, Area Engineer

Other Personnel

- \*J. C. Grill, Project Piping Engineer, Daniel International Corporation (DIC)
- \*C. L. McDonald, Hanger Engineer, DIC
- \*R. S. Calabro, Area Hanger Engineer, DIC
- \*N. A. Schryer, Project Quality Inspection Manager, DIC
- \*C. D. Mauldin, Project Quality Engineer, DIC
- R. Reeves, Hanger Engineer Aide, DIC
- Bill Browning, Area Hanger Engineer, DIC
- R. Pearson, Nuclear Control Engineer, Westinghouse

The NRC inspector also contacted other licensee and contractor personnel during the inspection period.

\*Denotes presence at the exit interview conducted August 30 - September 3, 1982.

2. Site Tour

The NRC inspector walked through various construction and storage areas to observe construction activities in progress and to inspect the general state of cleanliness and adherence to housekeeping requirements. The tour included the reactor building, reactor auxiliary building, diesel generator building and several outside storage areas.

No violations or deviations were identified.

### 3. Licensee Action on Previous Findings

(Open) Unresolved Item (STN 50-482/81-15): NRC Inspection Report STN 50-482/81-15 identified an unresolved item concerning WP-VII-208, Revision 5 which requires additional alteration and cut of support material to shape and size by mechanical or thermal means without engineering approval. Pending final approval of the latest revision of WP-VII-208, Revision 11, this item remains open.

(Open) 10 CFR 50.55(e) Report - Concrete Block Walls: Precast masonry units used in the construction of certain internal walls within the control building contain reinforcing steel. The design for these units requires the reinforcing to be positioned outward toward the faceshell of the concrete masonry unit (CMU). The reinforcing steel in the nonconforming walls was not positioned outward as required. There are some block walls in the control building which are about 50% complete. In October, the HVAC will be leak tested and inspected by DIC. Estimated date of closure of this nonconforming report is February 1983. This item remains open.

### 4. Spent Fuel Storage Racks

#### a. Review of Procedures and Specifications

The NRC inspector reviewed procedures for the receiving, storage, installation, and inspection of the spent fuel storage racks.

The installation and inspection instructions were contained in the traveler in draft form. The spent fuel storage racks were on hold in the storage area. The draft copy was not final, but appears to be adequate for the installation and inspection of spent fuel storage racks. Technical requirements, acceptance criteria and specific instructions are specified. Procedures and specifications reviewed by the NRC inspector that were not in the draft are listed below:

Specification #10466-C175, "Technical Specification for Purchase of Spent Fuel Storage Racks for the SNUPPS."

F-8, Revision 8, "Fuel Assembly, Storage and Refueling Equipment Design Interface Specification."

#### b. Observation of Work

The NRC inspector observed the spent fuel storage racks in the storage area. Storage conditions were adequate.

These racks are not installed at this time because of a nonconforming condition that exists. Upon receipt of the spent fuel storage racks, the licensee observed projections such as spot welds connecting envelopes that protruded into the inside surfaces up to approximately 1/32". This is contrary to Bechtel Specification 10466-C-175 R/A, which states in part that "No projections, such as weld crown, rivets, or bolt heads are permitted into the inside surfaces or envelope." This matter is presently being resolved by the licensee, Westinghouse, U. S. Tool & Die, Inc., and Bechtel.

The NRC inspector will follow up this resolution on the next inspection.

c. Review of Records

The NRC inspector reviewed receiving inspection records, shop fabrication records by the vendors, drawings, material certifications, and nonconformance reports generated by the vendor and contractor. Quality assurance records indicate that applicable requirements and commitments have been met.

The NRC inspector also reviewed the spent fuel racks traveler documents listed below:

SPENT FUEL RACKS TRAVELER DOCUMENTS

DRAWINGS

	<u>BECHTEL #</u>	<u>WACHTER ASSOC. #</u>
1.	C-175-0024-J3	M120-41, R2
2.	C-175-0025-04	M120-44, R3
3.	C-175-0026-02	M120-22, R0
4.	C-175-0027-02	M120-23, R0
5.	C-175-0028-02	M120-24, R0
6.	C-175-0031-03	M120-34, R2
7.	C-175-0033-04	M120-36, R3
8.	C-175-0034-04	M120-37, R3
9.	C-175-0040-04	M120-20, R3
10.	C-175-0041-04	M120-21, R3
11.	C-175-0051-03	M120-7, R2
12.	C-175-0052-02	M120-8, R1
13.	C-175-0066-01	T120-77, R0
14.	C-175-0072-01	T120-81, R0
15.	C-175-0073-01	M120-25, R0
16.	C-175-0074-01	M120-26, R0

	<u>BECHTEL #</u>	<u>WACHTER ASSOC. #</u>
17.	C-175-0075-01	M120-27, RO
18.	C-175-0076-01	M120-28, RO
19.	C-175-0077-01	M120-29, RO
20.	C-175-0078-01	M120-30, RO
21.	C-175-0079-01	M120-31, RO
22.	C-175-0092-02	M120-50, RO
23.	C-175-0093-02	M120-51, RO

PROCEDURE

Westinghouse Specification # F-8, Revision 8

No violations or deviations were identified.

5. Safety-Related Pipe Supports and Restraints

a. Review of Procedures

The NRC inspector reviewed several procedures during the course of the inspection. These procedures give instruction on the fabrication and the installation of pipe supports. Means have been established in these procedures to assure technical adequacy, acceptance criteria, and required documentation are specified. WP-VII-208, Revision 11, is out for final review; this procedure will clarify items on the present WP-VII-208, Rev. 10. Procedures reviewed are listed below:

WP-VII-207, Revision 9, "Fabrication of Component Supports"

WP-VII-208, Revision 10, "Installation of Component Supports"

Specification #10466-M-204, "Specification for the Field Fabrication and Installation of Piping and Pipe Supports to ASME Section III for the SNUPPS Certification"

Interim Change to Procedure (ICP) #571, QCP-VII-204, "Inspection and Documentation of Pipe Supports"

b. Observation of Work

The NRC inspector selected approximately twenty-seven completed pipe support travelers. Fourteen pipe supports were accessible. The pipe supports were installed in various systems located in the reactor building, reactor-auxiliary building, and the diesel generator building. The NRC inspector, a licensee QA engineer, and a DIC hanger

engineer visually inspected and compared each support with the documents contained in the traveler. There was one traveler that had a discrepancy. There was a plate attachment to the support that was  $\frac{1}{2}$ " thick, the Bill of Materials listed the plate to be  $\frac{1}{2}$ " thick; however, the Bechtel drawing showed  $1\frac{1}{2}$ " thick plate. A brief investigation of the matter proved that the plate was  $1\frac{1}{2}$ " thick. The mistake was made when the material list was transferred from the Bechtel drawing to the DIC Bill of Materials list. However, this should have been identified by the QC inspector. The contractor generated an NCR to correct the problem. Due to the fact that this was the only traveler out of the sample selection taken that was in error, this will be treated as an isolated case.

c. Review of Records

The NRC inspector reviewed approximately twenty-seven hanger travelers. The travelers reviewed were completed packages. All required documentation was present as required by procedure. The NRC inspector visually inspected fourteen pipe supports, the remainder were inaccessible. Discrepancies noted are discussed in paragraph 5.b. Travelers reviewed by the NRC inspector are listed below:

<u>Diesel Gen Bldg</u>	<u>Reactor</u>	<u>Auxiliary</u>
**L-JE02-C004/511	L-EM03-C007/231	L-EG24-R003/121
**L-JE03-C002/511	L-EM03-C008/232	L-EG24-R004/121
L-KC11-R205/511	L-EM03-C009/232	**L-EG24-R006/121
L-KC11-R206/511	L-EG17-H00/241	L-EG24-R007/111
**L-EF07-A001/521	L-EG13-C001/232	**L-EG24-R017/111
L-EF07-C008/511	**0-BB06-H002/232	**L-EG07-R018/122
**L-EF07-C009/511	L-EG07-R019/132	
L-EF07-H002/511	**LEG07-R020/134	
**L-EF07-H003/511	**L-EG07-C012/122	
**L-EF07-R008/511	**L-EG07-C013/122	
**0-EG08-H002/511		

No violations or deviations were identified

6. Exit Interview

An exit interview was conducted on September 3, 1982, with the licensee representatives denoted in paragraph 1 and T. Vandell (NRC Resident Reactor Inspector) to discuss the scope and findings of this inspection.

\*\*Visual Inspection Performed