QUAD-CITIES NUCLEAR POWER STATION

UNITS 1 AND 2

MONTHLY PERFORMANCE REPORT

OCTOBER 1982

COMMONWEALTH EDISON COMPANY

IOWA-ILLINOIS GAS & ELECTRIC COMPANY

NRC DOCKET NOS. 50-254 AND 50-265

LICENSE NOS. DPR-29 AND DPR-30

TABLE OF CONTENTS

I. INTRODUCTION

Quad-Cities Nuclear Power Station is composed of two Boiling Water Reactors, each with a Maximum Dependable Capacity of 769 MWe net, located in Cordova, Illinois. The Station is jointly owned by Commonwealth Edison Company and Iowa-Illinois Gas & Electric Company. The Nuclear Steam Supply Systems are General Electric Company Boiling Water Reactors. The Architect/Engineer was Sargent & Lundy, Incorporated, and the primary construction contractor was United Engineers & Constructors. The condenser cooling method is a closed cycle spray canal, and the Mississippi River is the condenser cooling water source. The plant is subject to license numbers DPR-29 and DPR-30, issued October 1, 1971, and March 21, 1972, respectively, pursuant to Docket Numbers 50-254 and 50-265. The date of initial reactor criticalities for Units 1 and 2, respectively, were October 18, 1971, and April 26, 1972. Commercial generation of power began on February 18, 1973 for Unit 1 and March 10, 1973, for Unit 2.

This report was compiled by Becky Brown and Randall Buss, telephone number 309-654-2241, extensions 127 and 181.

II. SUMMARY OF OPERATING EXPERIENCE

A. UNIT ONE

October 1-31: The unit continued its End of Cycle Six Refueling Outage throughout the month.

B. UNIT TWO

October 1-7: The unit began the month derated approximately 30 MWe due to high vibration in the 2A Recirculation pump Motor Generator Set. At 1600 hours, on October 1, the unit began decreasing load at 20 MWe/hour while repairs were made to the High Pressure Coolant Injection System. The load drop was stopped at 0130 hours on October 2. The unit started increasing load at 0615 hours, reaching 780 MWe on October 3.

October 8-16: On October 8, Unit Two began dropping load in preparation for a weekend Maintenance Outage. On October 9, the unit was manually scrammed by putting the Reactor mode switch to SHUTDOWN. The Reactor was made critical again on October 12 and the unit went on line at 0343 hours on October 13. Between 0450 hours and 1530 hours, load was held at 200 MWe for control rod scram timing, before increasing at 100 MWe/hour to 500 MWe for a control rod pattern change. Load then increased to 792 MWe by October 16.

October 17-20: On October 17, at 0050 hours, the unit began decreasing 100 MWe/hour to 700 MWe to perform weekly Turbine tests. At 0127 hours, the Reactor scrammed on an Average Power Range Monitor High-High signal due to a Condensate Demineralizer valve problem. The unit was critical at 0840 hours and on line at 1048 hours. Load increased to 550 MWe, where it was held for seven hours before increasing to 791 MWe by October 19. Load was then increased by withdrawing additional control rods without changing speed on the Recirculation pumps to 811 MWe; thus, eliminating the Recirc MG Set high vibration derating.

October 21-31: On October 21, at 0950 hours, Unit Two began decreasing load 100 MWe/hour in preparation for unit shutdown and was manually scrammed at 1702 hours by putting the Reactor mode switch to SHUTDOWN. Unit shutdown was required to repair the 33A Drywell to Suppression Chamber vacuum breaker, which stuck open during surveillance testing. The outage was extended to perform repairs to the MO-2-202-5A Recirculation Pump Discharge Valve stem. On October 25, the Reactor was made critical at 0438 hours and the unit was on line by 1145 hours. After testing the repaired valves, load was increased to approximately 810 MWe by October 29. At 2300 hours, on October 30, load was reduced to 700 MWe to perform weekly Turbine tests. In the interim, the Nuclear Engineer decided to reduce load further, to 655 MWe, to change control rod pattern. During the load increase to full power, the "B" Recirculation Motor Generator Set tripped on low oil pressure causing load to drop to 313 MWe. At the end of the reporting period, the unit was increasing load 5 MWe/hour from 460 MWe, after restarting the 2B Recirculation Pump.

III. PLANT OR PROCEDURE CHANGES, TESTS, EXPERIMENTS, AND SAFETY RELATED MAINTENANCE

A. Amendments to Facility License or Technical Specifications

On August 6, 1982, Amendments 81 and 75 were issued to licenses DPR-29 and DPR-30, respectively. These amendments replaced the Appendix B (non-radiological reporting requirements) with reporting requirements based on the Station National Pollutant Discharge Elimination (NPDES) Permit.

B. Facility or Procedure Changes Requiring NRC Approval

There were no Facility or Procedure changes requiring NRC approval for the reporting period.

C. Tests and Experiments Requiring NRC Approval

There were no Tests or Experiments requiring NRC approval for the reporting period.

D. Corrective Maintenance of Safety Related Equipment

The following represents a tabular summary of the safety related maintenance performed on Unit One and Unit Two during the reporting period. The headings indicated in this summary include: Work Request Numbers, LER Numbers, Components, Cause of Malfunctions, Results and Effects on Safe Operation, and Action Taken to Prevent Repetition.

UNIT ONE MAINTENANCE SUMMARY

W.R. NUMBER	LER NUMBER	COMPONENT	CAUSE OF MALFUNCTION	RESULTS & EFFECTS ON SAFE OPERATION	ACTION TAKEN TO PREVENT REPETITION
Q21763	82-20/03L	RCIC Area High Temperature Switch 1-1360-16C	The temperature switch was defective.	The isolation logic was still operable.	A new temperature switch was installed and tested satisfactorily.
Q17253		HFA Relay 100C 590-100C	The relay spool was cracked.	The relay would still operate properly.	The coil spool was replaced.
Q22073	82-26/03L	RHR Service Water Vault Penetration	The link seal was found loose in the penetration	Leakage was found on both seals during leak rate testing.	The seal was tightened and the leak rate test was performed satisfactorily.

UNIT TWO MAINTENANCE SUMMARY

W.R. NUMBER	LER NUMBER	COMPONENT	CAUSE OF MALFUNCTION	RESULTS & EFFECTS ON SAFE OPERATION	ACTION TAKEN TO PREVENT REPETITION
Q21253	82-17/03L	HPCI Steam Line Flange 2-2305-10"B	The flange gasket was leaking.	HPCI steam line flange was leaking steam.	Removed the old gasket; cleaned surface and re-installed new gasket.
Q22080	82-18/03L	Diesel Generator (Unit 2) (6600)	The heat exchanger was fouled.	Unit 2 Diesel Generator trips on high temperature.	The heat exchanger was replaced and the Diesel was tested satisfactorily.
Q20354		Recirc Inboard Sample Valve AO-2-220-44	The solenoid operated pilot valve was worn.	The isolation valve was still operable.	The pilot valve was replaced.
Q21724	82-25/03L	"A" SBGT Heater Temperature Trip 1/2-7541-11A	The temperature switch setting was incorrect.	The heater would trip and require manual resetting.	Reset temperature trip to 425°F.
Q22009	82-17/03L	HPCI Steam Line Flange 2-2305-10"B	The flange gasket was leaking.	Steam was leaking from the flange.	Disassembled flange, took out two flexi-tallic gaskets and re-assembled.
Q22159		AO-2-1601-20B Reactor Bldg to Torus Vacuum Breaker	The solenoid valve was worn.	The valve will not close from the Control Room.	The solenoid valve was replaced.

UNIT TWO MAINTENANCE SUMMARY

W.R. NUMBER	LER NUMBER	COMPONENT	CAUSE OF MALFUNCTION	RESULTS & EFFECTS ON SAFE OPERATION	ACTION TAKEN TO PREVENT REPETITION
Q22222	82-17/03L	HPCI Steam Supply Line Flange 2-2305-10"-B	The flange gasket was found to be leaking.	Steam was leaking through the gasket.	A new gasket was installed.
Q21905	82-19/03L	2A Recirculation Pump Discharge Valve MO-2-202-5A	Found studs missing on valve and valve shaft was bent.	Valve will not close from Control Room.	Repaired the stem and tested.
Q22221		Main Steam Line Low Pressure Isolation Channel 'B' 595-103D	A steam leak was found at a fitting on the "B" RPS channel pressure switch.	The low pressure isolation would not clear until reactor pressure was 920 psig.	The switches were calibrated and the leaky fittings were tightened.
Q22360	82-22/03L	Suppression Chamber to Drywell Vacuum Breaker 2-1601-33A	The shaft was binding against the valve bushing and packing.	The vacuum breaker was stuck open. The unit was shutdown to repair it.	The shaft was lubricated and the valve was cycled.

IV. LICENSEE EVENT REPORTS

The following is a tabular summary of all licensee event reports for Quad-Cities Units One and Two occurring during the reporting period, pursuant to the reportable occurrence reporting requirements as set forth in sections 6.6.B.1. and 6.6.B.2. of the Technical Specifications.

UNIT ONE

Licensee Event Report Number	Date	Title of Occurrence
82-31/03L	9-5-82	RCIC Inoperable
82-32/03L	9-5-82	HPCI Inoperable
82-33/03L	10-5-82	1A1 24 V Battery Failed Discharge Test
82-34/03L	10-13-82	Fuel Pool Monitor
82-35/03L	10-21-82	O.G. Isolation Less Than 15 Minute Failure

UNIT TWO

82-17/03L	10-1-82	HPCI Inoperable to Repair Steam Leak
82-18/03L	10-6-82	#2 Diesel Generator Trip - High Temperature
82-19/03L	10-21-82	5A Recirc Pump Discharge Valve Failure
82-20/03L	10-13-82	Torus Level Out of Specifications
82-21/03L	10-13-82	Turbine Pressure Regulator Bypassed
82-22/03L	10-21-82	1601-33A Vacuum Breaker Suppression Chamber to Drywell Failed Open

V. DATA TABULATIONS

The following data tabulations are presented in this report:

A. Operating Data Report

B. Average Daily Unit Power Level

C. Unit Shutdowns and Power Reductions

OPERATING DATA REPORT

DOCKET NO. 50-254

UNIT ONE

DATE November 01 1982

COMPLETED BY Randall D Buss

TELEPHONE 309-654-2241x181

OPERATING STATUS

0000 100182

1. Reporting period: 2400 103182 Gross hours in reporting period: 745

2. Currently authorized power level (MWt): 2511 Max.Depend capacity (MWe-Net): 769* Design electrical rating (MWe-Net): 789

3. Power level to which restricted(if any)(MWe-Net): NA

4. Reasons for restriction (if any):

	This Month	Yr.to Date	Cumulative
5. Number of hours reactor was critical	0.0	5833.1	74932.2
6. Reactor reserve shutdown hours	0.0	0.0	3421.9
7. Hours generator on line	0.0	5777.1	71908.6
8. Unit reserve shutdown hours.	0.0	0.0	909.2
9. Gross thermal energy generated(MWH)	0	11000483	146058842
10. Gross electrical energy generated(MWH)	4	3533214	47062147
11. Net electrical energy generated(MWH)	-11188	3196684	43780768
12. Reactor service factor	0.0	79.9	81.6
13. Reactor availability factor	0.0	79.9	85.3
14. Unit service factor	0.0	79.2	78.3
15. Unit availability factor	0.0	79.2	79.3
16. Unit capacity factor (Using MDC)	-2.0	57.0	62.0
17. Unit capacity factor (Using Des.MWe)	-1.9	55.5	60.4
18. Unit forced outage rate	0.0	1.5	6.7

19. Shutdowns scheduled over next 6 months (Type,Date,and Duration of each):

20. If shutdown at end of report period,estimated date of startup 12-13-82

*The MDC may be lower than 769 MWe during periods of high ambient temperature due to the thermal performance of the spray canal.

*UNOFFICIAL COMPANY NUMBERS ARE USED IN THIS REPORT

OPERATING DATA REPORT

DOCKET NO. 50-265

UNIT TWO

DATE November 01 1982

COMPLETED BY Randall D Buss

TELEPHONE 309-654-2241x181

OPERATING STATUS

0000 100182

1. Reporting period: 2400 103182 Gross hours in reporting period: 745

2. Currently authorized power level (MWt): 2511 Max.Depend capacity (MWe-Net): 769* Design electrical rating (MWe-Net): 789

3. Power level to which restricted(if any)(MWe-Net): NA

4. Reasons for restriction (if any):

	This Month	Yr.to Date	Cumulative
5. Number of hours reactor was critical	564.8	5955.8	70807.6
6. Reactor reserve shutdown hours	0.0	0.0	2985.8
7. Hours generator on line	544.5	5894.4	68135.5
8. Unit reserve shutdown hours.	0.0	0.0	702.9
9. Gross thermal energy generated(MWH)	1160398	13359154	141246237
10. Gross electrical energy generated(MWH)	368912	4249105	44955345
11. Net electrical energy generated(MWH)	350438	4046220	42170804
12. Reactor service factor	75.8	81.6	77.9
13. Reactor availability factor	75.8	81.6	81.2
14. Unit service factor	73.1	80.8	75.0
15. Unit availability factor	73.1	80.8	75.7
16. Unit capacity factor (Using MDC)	61.2	72.1	60.3
17. Unit capacity factor (Using Des.MWe)	59.6	70.3	58.8
18. Unit forced outage rate	15.5	17.5	9.3

19. Shutdowns scheduled over next 6 months (Type,Date,and Duration of each):

20. If shutdown at end of report period,estimated date of startup NA

*The MDC may be lower than 769 MWe during periods of high ambient temperature due to the thermal performance of the spray canal.

*UNOFFICIAL COMPANY NUMBERS ARE USED IN THIS REPORT

APPENDIX B
AVERAGE DAILY UNIT POWER LEVEL

DOCKET NO. 50-254

UNIT ONE

DATE November 01 1982

COMPLETED BY Randall D Buss

TELEPHONE 309-654-2241x181

MONTH October 1982

DAY	AVERAGE DAILY POWER LEVEL (MWe-Net)	DAY	AVERAGE DAILY POWER LEVEL (MWe-Net)
1.	-20.0	17.	-16.3
2.	-12.5	18.	-13.3
3.	-18.5	19.	-22.5
4.	-18.2	20.	-18.5
5.	-19.0	21.	-16.8
6.	-18.2	22.	-18.0
7.	-18.3	23.	-11.2
8.	-17.7	24.	-3.4
9.	-11.1	25.	-13.0
10.	-3.9	26.	-18.7
11.	-5.2	27.	-18.2
12.	-4.3	28.	-16.9
13.	-11.0	29.	-16.3
14.	-16.0	30.	-18.0
15.	-17.0	31.	-18.7
16.	-15.7		

INSTRUCTIONS

On this form, list the average daily unit power level in MWe-Net for each day in the reporting month. Compute to the nearest whole megawatt.

These figures will be used to plot a graph for each reporting month. Note that when maximum dependable capacity is used for the net electrical rating of the unit, there may be occasions when the daily average power level exceeds the 100% line (or the restricted power level line). In such cases, the average daily unit power output sheet should be footnoted to explain the apparent anomaly

APPENDIX B
AVERAGE DAILY UNIT POWER LEVEL

DOCKET NO. 50-265
UNIT TWO
DATE November 01 1982
COMPLETED BY Randall D Buss
TELEPHONE 309-654-2241x181

MONTH October 1982

DAY	AVERAGE DAILY POWER LEVEL (MWe-Net)	DAY	AVERAGE DAILY POWER LEVEL (MWe-Net)
1.	702.0	17.	295.5
2.	625.2	18.	652.7
3.	731.3	19.	744.1
4.	729.8	20.	767.1
5.	724.3	21.	427.9
6.	725.6	22.	-10.8
7.	730.1	23.	-10.5
8.	642.4	24.	-9.3
9.	-9.5	25.	183.8
10.	-7.2	26.	512.7
11.	-2.3	27.	589.0
12.	-8.3	28.	712.4
13.	214.0	29.	761.6
14.	522.3	30.	746.2
15.	749.0	31.	518.2
16.	652.5		

INSTRUCTIONS

On this form, list the average daily unit power level in MWe-Net for each day in the reporting month. Compute to the nearest whole megawatt.

These figures will be used to plot a graph for each reporting month. Note that when maximum dependable capacity is used for the net electrical rating of the unit, there may be occasions when the daily average power level exceeds the 100% line (or the restricted power level line). In such cases, the average daily unit power output sheet should be footnoted to explain the apparent anomaly

ID/5A

APPENDIX D
UNIT SHUTDOWNS AND POWER REDUCTIONS

QTP 300-S13
Revision 6
August 1982

DOCKET NO. 050-254

UNIT NAME Quad-Cities Unit 1

COMPLETED BY R. Buss
ext. 181

DATE November 1, 1982

REPORT MONTH OCTOBER 1982

TELEPHONE 309-654-2241

NO.	DATE	TYPE F OR S	DURATION (HOURS)	REASON	METHOD OF SHUTTING DOWN REACTOR	LICENSEE EVENT REPORT NO.	SYSTEM CODE	COMPONENT CODE	CORRECTIVE ACTIONS/COMMENTS
82-85	820906	S	745:00	C	4		RC	FUELXX	Continuation of Cycle Six Refueling Outage

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APPENDIX D
UNIT SHUTDOWNS AND POWER REDUCTIONS

QTP 300-S13
Revision 6
August 1982

DOCKET NO. 050-265

UNIT NAME Quad-Cities Unit 2

COMPLETED BY R. Buss
ext. 181

DATE November 1, 1982

REPORT MONTH OCTOBER 1982

TELEPHONE 309-654-2241

NO.	DATE	TYPE F OR S	DURATION (HOURS)	REASON	METHOD OF SHUTTING DOWN REACTOR	LICENSEE EVENT REPORT NO.	SYSTEM CODE	COMPONENT CODE	CORRECTIVE ACTIONS/COMMENTS
82-73	821001	F	0.0	B	5	82-17/03L	SF	PIPEXX	Reduced load while performing repairs to High Pressure Coolant Injection System
82-74	821008	S	0.0	B	5		ZZ	ZZZZZZ	Load reduction prior to weekend Maintenance Outage
82-75	821008	S	100.3	B	2		ZZ	ZZZZZZ	Scheduled shutdown for weekend Maintenance Outage and Battery Discharge Test
82-76	821017	S	0.0	B	5		HA	XXXXXX	Reduced load to perform weekly Turbine test
82-77	821017	F	9.3	A	3		HG	DEMINX	Reactor scram on Average Power Range Monitor High-High signal due to increase in Feedwater flow caused by Condensate Demineralizer valve problems
82-78	821021	S	0.0	B	5	82-22/03L	SD	VALVEX	Load reduction prior to short Maintenance Outage to repair 33A vacuum breaker

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APPENDIX D
UNIT SHUTDOWNS AND POWER REDUCTIONS

QTP 300-S13
Revision 6
August 1982

DOCKET NO. 050-265

UNIT NAME Quad-Cities Unit 2

COMPLETED BY R. Buss
ext. 181

DATE November 1, 1982

REPORT MONTH OCTOBER 1982

TELEPHONE 309-654-2241

NO.	DATE	TYPE F OR S	DURATION (HOURS)	REASON	METHOD OF SHUTTING DOWN REACTOR	LICENSEE EVENT REPORT NO.	SYSTEM CODE	COMP NT COD	CORRECTIVE ACTIONS/COMMENTS
82-79	821021	F	19.2	B	1	82-22/03L	SD	VALVEX	Scheduled Maintenance Outage to repair 33A Vacuum Breaker
82-80	821022	F	71.7	B	4	82-19/03L	CB	VALVEX	Outage continued to repair 5A Recirculation Pump Discharge Valve stem
82-81	821030	S	0.0	B	5		HA	XXXXXX	Reduced load to perform weekly Turbine test
82-82	821031	F	0.0	A	5		CB	INSTRU	Load reduction due to "B" Recirculation MG Set trip on low oil pressure

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VI. UNIQUE REPORTING REQUIREMENTS

The following items are included in this re-ort based on prior commitments to the commission:

A. Main Steam Relief Valve Operations

Relief valve operations during the reporting period are summarized in the following table. The table includes information as to which relief valve was actuated, how it was actuated, and the circumstances resulting in its actuation.

Unit	Date	Valves Actuated	No. & Type Actuations	Plant Conditions	Description of Events
2	10-9-82	2-203-3A 2-203-3B 2-203-3C 2-203-3D	1 Manual 1 Manual 1 Manual 1 Manual	Rx Press 820	Surveillance T.S. 4.5.D.1.b
2	10-12-82	2-203-3E	1 Manual	Rx Press 215	Post-Maintenance (replace valve)
2	10-12-82	2-203-3A 2-203-3B 2-203-3C 2-203-3D	1 Manual 1 Manual 1 Manual 1 Manual	Rx Press 215	Surveillance T.S. 4.5.C.2 (HPCI Out of Service)

B. Control Rod Drive Scram Timing Data for Units One and Two

The basis for reporting this data to the Nuclear Regulatory Commission are specified in the surveillance requirements of Technical Specifications 4.3.C.1 and 4.3.C.2.

The following table is a complete summary of Units One and Two Control Rod Drive Scram Timing for the reporting period. All scram timing was performed with reactor pressure greater than 800 psig.

RESULTS OF SCRAM TIMING MEASUREMENTS

PERFORMED ON UNIT 1 & 2 CONTROL

ROD DRIVES, FROM 1-1 TO 12-31-82

DATE	NUMBER OF RODS	AVERAGE TIME IN SECONDS AT % INSERTED FROM FULLY WITHDRAWN				Max. Time For 90% Insertion	DESCRIPTION
		5	20	50	90		Technical Specification 3.3.C.1 & 3.3.C.2 (Average Scram Insertion Time)
		0.375	0.900	2.00	3.5	7 sec.	
10-13	89	0.29	0.67	1.44	2.55	2.95 (D-10)	Unit 2 Hot Scram Time "B" Sequence

VII. REFUELING INFORMATION

The following information about future reloads at Quad-Cities Station was requested in a January 26, 1978, licensing memorandum (78-24) from D. E. O'Brien to C. Reed, et al., titled "Dresden, Quad-Cities, and Zion Station -- NRC Request for Refueling Information", dated January 18, 1978.

QTP 300-S32
Revision 1
March 1978

QUAD-CITIES REFUELING INFORMATION REQUEST

*

1. Unit: 1 Reload: 6 Cycle: 7

2. Scheduled date for next refueling shutdown: Sept 12, 1982

3. Scheduled date for restart following refueling: Dec 4, 1982

4. Will refueling or resumption of operation thereafter require a technical specification change or other license amendment:

   YES

5. Scheduled date(s) for submitting proposed licensing action and supporting information:

   JULY 26, 1982

6. Important licensing considerations associated with refueling, e.g., new or different fuel design or supplier, unreviewed design or performance analysis methods, significant changes in fuel design, new operating procedures:

   IMPLEMENTATION OF THE ODYN TRANSIENT ANALYSIS CODE AND RESULTS
   (MCPR SCRAM TIME DEPENDENCE)

7. The number of fuel assemblies.

   a. Number of assemblies in core: 224 new/724 total

   b. Number of assemblies in spent fuel pool: after the outage 1940

8. The present licensed spent fuel pool storage capacity and the size of any increase in licensed storage capacity that has been requested or is planned in number of fuel assemblies:

   a. Licensed storage capacity for spent fuel: 2920

   b. Planned increase in licensed storage: 4636 new/7556 total

9. The projected date of the last refueling that can be discharged to the spent fuel pool assuming the present licensed capacity:

   LOSS OF FULL CORE DISCHARGE CAPABILITY - 3/84
   LOSS OF RELOAD CORE DISCHARGE CAPABILITY - 2/86

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QTP 300-S32
Revision 1
March 1978

QUAD-CITIES REFUELING INFORMATION REQUEST

*

1. Unit: 2 Reload: 6 Cycle: 7

2. Scheduled date for next refueling shutdown: Feb 27, 1983

3. Scheduled date for restart following refueling: April 23, 1983

4. Will refueling or resumption of operation thereafter require a technical specification change or other license amendment:

   NO

5. Scheduled date(s) for submitting proposed licensing action and supporting information:

   NONE

6. Important licensing considerations associated with refueling, e.g., new or different fuel design or supplier, unreviewed design or performance analysis methods, significant changes in fuel design, new operating procedures:

   NONE

7. The number of fuel assemblies.

   a. Number of assemblies in core: 192 new/724 total

   b. Number of assemblies in spent fuel pool: after the outage 2132

8. The present licensed spent fuel pool storage capacity and the size of any increase in licensed storage capacity that has been requested or is planned in number of fuel assemblies:

   a. Licensed storage capacity for spent fuel: 2920

   b. Planned increase in licensed storage: 4636 new/7556 total

9. The projected date of the last refueling that can be discharged to the spent fuel pool assuming the present licensed capacity:

   LOSS OF FULL CORE DISCHARGE CAPABILITY - 3/84
   LOSS OF RELOAD CORE DISCHARGE CAPABILITY - 2/86

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Q.C.O.S.R.

VIII. GLOSSARY

The following abbreviations which may have been used in the Monthly Report, are defined below:

ACAD/CAM	-	Atmospheric Containment Atmospheric Dilution/Containment Atmospheric Monitoring
ANSI	-	American National Standards Institute
APRM	-	Average Power Range Monitor
ATWS	-	Anticipated Transient Without Scram
BWR	-	Boiling Water Reactor
CRD	-	Control Rod Drive
EHC	-	Electro-Hydraulic Control System
EOF	-	Emergency Operations Facility
GSEP	-	Generating Stations Emergency Plan
HEPA	-	High-Efficiency Particulate Filter
HPCI	-	High Pressure Coolant Injection System
HRSS	-	High Radiation Sampling System
IPCLRT	-	Integrated Primary Containment Leak Rate Test
IRM	-	Intermediate Range Monitor
ISI	-	Inservice Inspection
LER	-	Licensee Event Report
LLRT	-	Local Leak Rate Test
LPCI	-	Low Pressure Coolant Injection Mode of RHRS
LPRM	-	Local Power Range Monitor
MAPLHGR	-	Maximum Average Planar Linear Heat Generation Rate
MCPR	-	Minimum Critical Power Ratio
MFLCPR	-	Maximum Fraction Limiting Critical Power Ratio
MPC	-	Maximum Permissible Concentration
MSIV	-	Main Steam Isolation Valve
NIOSH	-	National Institute for Occupational Safety and Health
PCI	-	Primary Containment Isolation
PCIOMR	-	Preconditioning Interim Operating Management Recommendations
RBCCW	-	Reactor Building Closed Cooling Water System
RBM	-	Rod Block Monitor
RCIC	-	Reactor Core Isolation Cooling System
RHRS	-	Residual Heat Removal System
RPS	-	Reactor Protection System
RWM	-	Rod Worth Minimizer
SBGTS	-	Standby Gas Treatment System
SBLC	-	Standby Liquid Control
SDC	-	Shutdown Cooling Mode of RHRS
SDV	-	Scram Discharge Volume
SRM	-	Source Range Monitor
TBCCW	-	Turbine Building Closed Cooling Water System
TIP	-	Traveling Incore Probe
TSC	-	Technical Support Center