



Commonwealth Edison
1400 Opus Place
Downers Grove, Illinois 60515

January 2, 1991

Office of Nuclear Reactor Regulation
U.S. Nuclear Regulatory Commission
Attention: Document Control Desk
Washington, D.C. 20555

Subject: LaSalle Nuclear Power Station Units 1 and 2
Reactor Vessel Head Inspection
NRC Docket Nos. 50-373 and 50-374

References: (a) R. Stols Letter to T.E. Murley dated
May 4, 1990.

(b) W.E. Morgan letter to A.B. Davis dated
August 11, 1990.

Dear Sir:

References (a) and (b) committed to a review of the fabrication contract variations of the LaSalle Unit 1 Reactor Vessel to support the development of an inspection plan. This inspection plan was to be submitted for your staff's review. The following attachment provides the inspection plan which is to be implemented during the upcoming Unit 1, February refueling outage.

If there are any questions regarding this subject, please contact this office.

Very truly yours,

Wayne E Morgan

W. E. Morgan
Nuclear Licensing Administrator

cc: A.B. Davis - Regional Administrator, RIII
J.B. Hickman - Project Manager, NRR
Senior Resident Inspector, LaSalle

WM:lmw
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Q PDR

W. E. Morgan
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ATTACHMENT
PROPOSED LASALLE UNIT 1 VESSEL
INSPECTION PLAN FOR THE UPCOMING
FEBRUARY REFUELING OUTAGE

The following is the inspection plan to be implemented during the next LaSalle Unit 1 refueling outage. The inspection plan includes the reactor vessel head.

The partial fabrication records review for the upper region of the LaSalle Unit 1 reactor pressure vessel revealed minor machining and preheat control errors, and a few repairs of base metal, seam welds, and cladding. No mismatches were identified.

The inspection of the reactor head will be performed in response to GE RICSIL 050.

The number of areas of manual backclad was replaced as a result of repairs to the vessel flange-to-upper shell weld. However, no examination of the vessel flange-to-upper shell weld is proposed because (1) no mismatch or contract variation was identified, (2) there is no indication of abnormally wide backclad regions, (3) the backclad material has low carbon content, and (4) this weld was completely ultrasonically examined for Section XI during 1988 and 1989 and no recordable indication was found.

REACTOR HEAD

1. Direct visual examination, VT-1, of 100% ID of the reactor head. Lighting adequacy must be verified with a 1/32 inch black line on a 18% neutral gray card.

Conditions to be examined for and recorded:

- A. Cracks or linear indications
 - B. Heavy/unusual rust streaks
 - C. Clad repair and/or clad thickness variations
 - D. Excessive grinding
2. Photograph representative areas of the ID of the reactor head.
 3. Dye penetrant test (PT) the following:
 - A. All linear indications
 - B. All heavy/unusual rust streaks
 - C. Sample of clad repair and/or clad thickness variation areas as determined by CECO Nuclear Engineering Department (NED).
 - D. Sample of excessive grinding areas as determined by NED.
 4. Ultrasonically test (UT), from the ID, all cracks detected by VT and/or PT.