BIG ROCK POINT PHYSICS METHODOLOGY REPORT

REVISION 3

Consumers Power Company

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2.2 OPERATION AND SURVEILLANCE (CONT'D)

shutdown margin, rod worth, notch worth, MAPLHGR, MCPR, MCHFR, assembly power and heat flux while allowing operation at the highest possible power level. The safety analysis verifies that these limits will be met, and in addition verifies that other parameters: reactivity coefficients, beta/lambda, liquid poison worth and scram reactivity insertions times, are within the assumptions used in the plant accident and transient analyses.

The startup physics test program consists of verification of shutdown margin, comparison of the zero power critical control rod density with predictions, and comparison of measured flux wire shapes with predicted ones.

During operation power distribution calculations are periodically performed to evaluate margin to thermal limits. Once a month calculated reactivity and flux wire activation shapes are compared with measurements to monitor the adequacy of the calculational model. The flux wire measurements are also used to calibrate the incore detectors and determine their alarm setpoints.

3.0 PHYSICS MODEL

3.1 OVERVIEW

The calculational sequence for Big Rock Point physics is diagrammed in Figure 3-1. The primary component of the sequence is the three dimensional reactor simulator program, GROK. The remainder of the sequence primarily involves the generation of input for GROK. The

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4.8 METHOD OF STATISTICAL ANALYSIS (CONT'D)

The assumption of independence is not exactly true, but the second order effects should be small enough to ignore. For example, an error in the radial power distribution will cause a change in the axial power distribution, but the difference in the axial should be small enough to ignore. Another aspect of the independence assumption is that the point in the core that has the largest deviation of calculated to actual in one parameter does not also contain a large deviation in one or all of the other parameters.

4.9 MCPR UNCERTAINTY FACTOR

The minimum critical power ratio (MCPR) is expressed as the ratio of critical power level (CPL) to actual power level (APL). This ratio has to be greater than or equal to 1.32^7 . The ratio may be formulated as P = CPL/APL \geq 1.32. Using statistical propogation of errors, the uncertainty factor (U) for P is:

$$U = \sqrt{\left(\frac{\partial P}{\partial CPL} - \sigma CPL\right)^2 + \left(\frac{\partial P}{\partial CPL} - \sigma APL\right)^2}$$

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By evaluating the partial derivatives, the equation becomes:

$$= \sqrt{\left(\frac{1}{APL}\int^{CPL}\right)^2 + \left(\frac{-CPL}{APL^2}\sigma^{-APL}\right)^2}$$

The standard deviation in the APL is equal to the standard deviation in the radial power distribution, 0.03766° (from Section 5.0), times the APL. The CPL can be formulated as CPL = APL(1.32 + U). The standard deviation in the CPL can be formulated as:

$$\mathbf{\nabla}_{\mathrm{CPL}^2} = \left(\begin{array}{c} \frac{\partial \mathrm{CPL}}{\partial \mathrm{CHFR}} & \mathbf{\nabla}_{\mathrm{CHFR}} \end{array} \right)^2$$

CHFR - Critical Heat Flux Ratio

CHFR is expressed as the ratio of critical heat flux (CHF) to actual heat flux (AHF). The MCFR calculation forces the CHFR to equal

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4.9 MCPR UNCERTAINTY FACTOR (CONT'D)

1.0 \pm 0.01, by adjustment of the critical power ratio (CPR). Using statistical propogation of errors, the uncertainty factor (V) for CHFR is:

$$V = \sqrt{\left(\frac{\partial CHFR}{\partial CHF} \sigma_{CHF}\right)^2 + \left(\frac{\partial CHFR}{\partial AHF} \sigma_{AHF}\right)^2}$$

By evaluating the partial derivatives, the equation becomes:

$$V = \sqrt{\left(\frac{1}{AHF} \, \sigma_{CHF}\right)^2 + \left(\frac{-CHF}{AHF^2} \, \sigma_{AHF}\right)^2}$$

As AHF increases, V decreases. Thus, AHF will be chosen as the largest allowed, that is, CHF minus allowance for the uncertainty factor, or AHF equals CHF(1-V). The standard deviation in the actual heat flux (\checkmark AHF) is equal to the standard deviation in the peak heat flux, 0.094017 (from Section 5.0), minus the 2% heat balance error, times CHF(1-V) or 0.09186 times CHF(1-V).

The standard deviation in the critical heat flux (\P CHF) is derived from the XN-2 critical heat flux correlation. Differentiating the correlation, with respect to the local peaking results in the following formulation for σ CHF:

$$(\boldsymbol{\sigma}_{CHF})^2 = \left(\frac{\partial XN-2}{\partial F} \boldsymbol{\sigma}_{FL}\right)^2$$

This relationship for the CHF indicates that the uncertainty in the CHF resulting from any other independent variable has been accounted for in the transient analysis. The partial derivative, in the above equation, is a function of mass velocity (G) and a non-uniform axial heat flux correction factor (F-factor). A study to determine conservative values for the mass velocity and the F-factor was performed by analyzing past cycles using GROK. Combining the results of the study with the local peaking factor uncertainty, from Section 5.0, the CHF is 0.03395. Evaluation of the uncertainty in the CHFR required that a conservative value for the CHF be determined. The CHF value was determined using the XN-2 correlation and conservative or limiting values for mass velocity, F-factor, reactor pressure, enthalpy, and local peaking. Values for mass velocity and F-factor are the same as those used in calculating σ CHF. The enthalpy and reactor pressure were assigned full power values of 570.96 Btu/lb and 1,350 PSIA, respectively. Maximization of the CHF required the smallest local peaking factor (F1). The limiting F1 was determined by evaluating past cycles, using GROK. The minimum F1 from the past cycles was determined to be 1.13986. This F1 must then be corrected for F1 uncertainty resulting in a F1 of 1.06593. Combining the above results yields a CHF of 1.0961 x 10⁶ Btu/hr-ft².

Substitution of the above calculated values for AHF, **V**AHF, **V**CHF and CHF into the relationship for V results in a CHFR uncertainty of 0.11005.

A study was performed to determine a conservative value for the partial derivative of CPL with respect to the partial derivative of CHFR (∂ CPL/ ∂ CHFR). For each assembly, a linear relationship was defined using the variables CPR, APL and CHFR. This relationship can be formulated as follows:

$$\frac{\partial \text{CPL}}{\partial \text{CHFR}} = \frac{\Delta \text{CPP}(\text{APL})}{\Delta \text{CHFR}}$$

The average $\partial CPL/\partial CHFR$, from the study, was 3.53846 \pm 0.59001 MWt. To bring the average $\partial CPL/\partial CHFR$ up to a 95/95 one-sided confidence level, the standard deviation should be multiplied by 1.645 and added to the average. Thus, $\partial CPL/\partial CHFR$ equals 4.50903 MWt. Substituting the values for the \mathcal{T}_{CHFR} and $\partial_{CPL}/\partial_{CHFR}$ into the equation for the ∇_{CPL} gives a ∇_{CPL} equal to 0.49622 MWt.

A formula for APL can be derived using Figure 4-6. Figure 4-6 is a graph of MCPR vs Assembly Power (MWt) A least squares fit was applied to the data and 99/99 confidence lines were drawn in. The equation for line 1 is:

MCPR = -0.71382(APL) + 3.95459

The above equation can be written in terms of U and APL as follows:

APL = U - 2.63459 / - 0.71382

Substitution of the above calculated values for ∇ APL, CPL, ∇ CPL and APL into the relationship for U results in a MCPR uncertainty of 0.1531.

5.0 SUMMARY AND CONCLUSIONS

The reactor physics methods employed at Consumers Power Company are very similar to methods used elsewhere in the industry. The computer models are or are derived from widely accepted codes which are well tested and documented.

Agreement with measured data and higher order calculations has demonstrated the accuracy and applicability of the methodology. Reactivity is consistently predicted at both cold and hot operating conditions, and power distributions agree well with the measurements, and higher order calculations indicating that the various neutronic effects are being properly modeled. The table below symmarizes the various uncertainty factors at a 95/95 one or two sided confidence level.

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Parameter	Type	Uncertainty Factor
Bundle Power	one sided	3.7661%
Axial Power	one sided	5.3045%
Local Peaking	one sided	6.4861%
Peak Heat Flux*	one sided	9.4017%
MAPLHGR*	one sided	0.0784
MCHFR*	one sided	0.3228
Void Coefficient	two sided	0.0351
MCPR	one sided	0.1531

*The uncertainty factor includes the effects of the 2% heat balance error.



REFERENCES

- A. Ahlin and M. Edenius, "CASMO The Fuel Assembly Burnup Program", AE-RF-76-4158, AB ATOMENERGI, Sweden (1976).
- E.E. Pilat et al, "Methods for the Analysis of Boiling Water Reactors Lattice Physics", YAEC-1232, Yankee Atomic Electric Co. (December 1980).
- G.S. Lellouche, "Mechanistic Model for Predicting Two-Phase Void Fraction for Water in Vertical Tubes, Channels and Rod Bundles", EPRI NP-2246-SR (February 1982).
- D.L. Delp, et al, "FLARE A Three Dimensional Boiling Water Reactor Simulator", GEAP-4598 (July 1964).
- 5. B.D. Webb, "Power Peaking at the Tip of a Cruciform Control Rod", Consumers Power Company, (January 6, 1972).
- 6. "Jersey Nuclear Company Development Fuel Assemblies for Loading in the Consumers Power Company Big Rock Point Reactor Cycle 9", JN-70-1, Jersey Nuclear Company, (November 15, 1970), pp. 100-105, corrected via conversation between SVanVolkinburg and GFPratt, Consumers Power Company, (August 29, 1977).
- K. Galbraith and J. Jaech, "The XN-2 Critical Power Correlation", XN-75-34, Exxon Nuclear Company, (August 1, 1975).
- A.A. Armand, "The Resistance During the Movement of a Two-Phase System in Horizontal Pipes", Translated by V. Beak, AERE Trans 828. Izvestiya Vsesojuznogo Teplotekhnicheskogo Instituta (1), pp 16-23, (1946).
- 9. "General Electric Company Analytical Model for Loss-of-Coolant Analysis in Accordance with 10CFR50 Appendix K", NED0-20566, (January, 1976).
- W.J. Eich et al, "Advanced Recycle Methodology Program System Documentation", EPRI-RP118-1, (October, 1976).
- K.P. Galbraith, "GAPEX, A Computer Program for Predicting Pellet to Cladding Heat Transfer Coefficients", XN-73-25, Exxon Nuclear Company (August 13, 1973).
- 12. W.R. Cadwell, "PDQ-7 Reference Manual", WAPD-TM-676 (1967).
- "Jersey Nuclear Company Development Fuel Assemblies for loading in the Consumers Power Company Big Rock Point Reactor Cycle 9", JN-70-1, (November 15, 1970).
- 14. "Jersey Nuclear Company Pu02-U02 Development Fuel Assemblies for loading in the Consumers Power Company Big Rock Point Reactor Cycle 10", JN-71-6, (September 1, 1971).