

LICENSEE EVENT REPORT

CONTROL BLOCK: \_\_\_\_\_ (1) (PLEASE PRINT OR TYPE ALL REQUIRED INFORMATION)

0 1 | C | O | F | S | V | 1 | 2 | 0 | 0 | - | 0 | 0 | 0 | 0 | 0 | - | 0 | 0 | 3 | 4 | 1 | 1 | 2 | 0 | 4 | \_\_\_\_\_ | 5  
7 8 9 LICENSEE CODE 14 15 LICENSE NUMBER 25 26 LICENSE TYPE 30 57 CAT 58

CON'T  
0 1 | R | P | T | S | O | U | R | C | E | L | 6 | 0 | 5 | 0 | 0 | 0 | 2 | 6 | 7 | 7 | 1 | 0 | 0 | 2 | 8 | 2 | 8 | 1 | 1 | 0 | 1 | 1 | 8 | 2 | 9  
7 8 REPORT SOURCE 60 61 DOCKET NUMBER 68 69 EVENT DATE 74 75 REPORT DATE 80

EVENT DESCRIPTION AND PROBABLE CONSEQUENCES (10)

0 2 | During the performance of scheduled surveillance testing on October 2 and 8, 1982,  
0 3 | all six steam pipe rupture (under PCRV) channels and seven of twelve steam pipe rup-  
0 4 | ture (pipe cavity) channels were found inoperable due to low response to a known  
0 5 | noise source. This is a degraded mode of LCO 4.4.1, Table 4.4-2, and reportable per  
0 6 | Fort St. Vrain Technical Specification AC 7.5.2(b)2. No affect on public health or  
0 7 | safety. No accompanying occurrence. Redundant systems available and operable.  
0 8 | Similar reports are RO's: 81-012 and 81-042.

0 9 | SYSTEM CODE | CAUSE CODE | CAUSE SUBCODE | COMPONENT CODE | COMP. SUBCODE | VALVE SUBCODE  
I B | X | Z | I N S T R U | E | Z  
9 10 11 12 13 18 19 20  
17 | LER/RO REPORT NUMBER | EVENT YEAR | SEQUENTIAL REPORT NO. | OCCURRENCE CODE | REPORT TYPE | REVISION NO.  
8 2 | - | 0 4 0 | / | 0 3 | - | 0  
21 22 23 24 26 27 28 29 30 31 32  
ACTION TAKEN | FUTURE ACTION | EFFECT ON PLANT | SHUTDOWN METHOD | HOURS | ATTACHMENT SUBMITTED | NPRD-4 FORM SUB. | PRIME COMP. SUPPLIER | COMPONENT MANUFACTURER  
E | X | Z | Z | 0 0 0 0 | Y | N | N | G 2 9 0  
33 34 35 36 37 40 41 42 43 44 47

CAUSE DESCRIPTION AND CORRECTIVE ACTIONS (27)

1 0 | Instrument calibration drift caused the channels to be outside the acceptable limits.  
1 1 | The gain of the microphones/transmitters was adjusted to acceptable levels, and the  
1 2 | test satisfactorily completed. Semi-annual calibration checks are being performed to  
1 3 | detect system drift. Nuclear Engineering Division is currently evaluating the detec-  
1 4 | tion system, the results of which will be included in a future supplemental report.

1 5 | FACILITY STATUS | % POWER | OTHER STATUS | METHOD OF DISCOVERY | DISCOVERY DESCRIPTION  
G | 0 0 0 | N/A | B | Scheduled Surveillance Testing  
7 8 9 10 12 13 44 45 46 80

1 6 | ACTIVITY CONTENT | AMOUNT OF ACTIVITY | LOCATION OF RELEASE  
Z | Z | N/A | N/A  
7 8 9 10 11 44 45 80

1 7 | PERSONNEL EXPOSURES | DESCRIPTION  
0 0 0 | Z | N/A  
7 8 9 10 11 12 13 80

1 8 | PERSONNEL INJURIES | DESCRIPTION  
0 0 0 | N/A  
7 8 9 10 11 12 13 80

1 9 | LOSS OF OR DAMAGE TO FACILITY | DESCRIPTION  
Z | N/A  
7 8 9 10 11 12 13 80

2 0 | PUBLICITY | DESCRIPTION  
N | N/A  
7 8 9 10 11 12 13 80

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PDR ADOCK 05000267  
S PDR

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