



December 21, 1990

MFN No. 155-90  
Docket No. STN 50-605  
EEN-9081

Document Control Desk  
U.S. Nuclear Regulatory Commission  
Washington, D.C. 20555

Attention: Charles L. Miller, Director  
Standardization and Non-Power Reactor Project Directorate

Subject: **Submittal of Amendment 15 to GE's ABWR SSAR, Figures**

Reference: Submittal of Amendment 15 to GE's ABWR SSAR,  
MFN No. 145-9, dated November 30, 1990

Enclosed are thirty-four copies of the Chapter 5 figures changed in Amendment 15. The instructions of filing these figures is noted on the page change instructions supplied with the above reference. A copy of page 2 & 3 of those instructions is attached, the affected pages are identified by the notation "Fig\*".

Sincerely,

R.C. Stirn, Acting Manager  
Regulatory and Analysis Services  
M/C 382, (408) 925-6948

cc: F. A. Ross (DOE)  
D. C. Scaletti (NRC)  
D. R. Wilkins (GE)  
J. F. Quirk (GE)

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# ABWR SSAR

## Amendment 15 - Page change instruction

The following pages have been changed, please make the specified changes in your SSAR. Pages are listed as page pairs (front & back). Bold page numbers represent a page that has been changed by Amendment 15.

REMOVE PAGE No.	ADD PAGE No.	REMOVE PAGE No.	ADD PAGE No.
<b>CHAPTER 4</b>			
4.0-ii,iii	4-ii,iii	4A Cover	4A Cover
		4A-ii,iii	4A-ii,iii
		4A-1	4A-1
4.1-ii,iii	4.1-ii,iii	Add	4B Cover
4.1-1,2	4.1-1,1,1	Add	4B-ii
Add	4.1-2	Add	4B-1 - 7
4.1-5,6	4.1-5,6		
4.2-ii	4.2-ii	Add	4C Cover
4.2-1	4.2-1	Add	4C-ii
4.2-2	4.2-2	Add	4C-1
4.3-ii,iii	4.3-ii,iii	<b>CHAPTER 5</b>	
4.3-1,2	4.3-1,2	5.0-ii,iii	5.0-ii,iii
4.3-3	4.3-3		
4.3-4	4.3-4,5	5.1-ii	5.1-ii
4.4-ii,iii	4.4-ii,iii	5.1-1,2	5.1-1,2
4.4-iv	4.4-iv	5.1-3,4	5.1-3,4
4.4-1,2	4.4-1,2	5.1-5,6	5.1-5,6 Fig*
4.4-3,4	4.4-3,4	5.1-7,8	5.1-7,8 Fig*
4.4-5,6	4.4-5,6	5.1-9	5.1-9,9.1 Fig*
4.4-7,8	4.4-7,8	Add	5.1-9.2,9.3 Fig*
4.4-9,10	4.4-9,10	Add	5.1-9.4,9.5 Fig*
4.4-11	4.4-11	Add	5.1-9.6 Fig*
4.5-ii	4.5-ii	5.1-10,11	5.1-10,11
4.5-4a	4.5-4.1	5.2-ii,iii	5.2-ii,iii
4.6-ii,iii	4.6-ii,iii	5.2-iv,v	5.2-iv,v
4.6-iv,v	4.6-iv,v	5.2-vii,viii	5.2-vii,viii
4.6-1,1a	4.6-1,1a	5.2-ix	5.2-ix
4.6-9,10	4.6-9,10	5.2-1,2	5.2-1,2
4.6-11,12	4.6-11,12	5.2-3,4	5.2-3,3.1
4.6-13,14	4.6-13,14	Add	5.2-4
4.6-15,16	4.6-15,16	5.2-5,6	5.2-5,6
		5.2-7,8	5.2-7,8
		5.2-10	5.2-10

\* These figures, 11x17 foldouts, are being sent under separate cover

# ABWR SSAR

## Amendment 15 - Page change instruction

The following pages have been changed, please make the specified changes in your SSAR. Pages are listed as page pairs (front & back). Bold page numbers represent a page that has been changed by Amendment 15.

REMOVE PAGE No.	ADD PAGE No.	REMOVE PAGE No.	ADD PAGE No.
5.2-12	5.2-12	5.4-3,4	5.4-3,4
5.2-13,14	5.2-13,14	5.4-5,5.1	5.4-5,5.1
5.2-15,15.1	5.2-15,15.1	5.4-6	5.4-6
5.2-16,17	5.2-16,17	5.4-7,7a	5.4-7,7.1
5.2-17.1,17.2	5.2-17.1,17.2	5.4-8	5.4-8
5.2-18	5.2-18	5.4-9,10	5.4-9,10
5.2-19,20	5.2-19,20	5.4-10a	5.4-10.1
5.2-21,22	5.2-21,22	5.4-11,12	5.4-11,12
5.2-23,24	5.2-23,24	5.4-13,14	5.4-13,14
5.2-25,26	5.2-25,26	5.4-14a	5.4-14.1
5.2-27,28	5.2-27,28	5.4-15,15a	5.4-15,15.1
5.2-28a	5.2-28.1	5.4-16,16a	5.4-16,16.1
5.2-30	5.2-30	5.4-17,18	5.4-17,18
5.2-31,32	5.2-31,32	5.4-18a	5.4-18.1
Add	5.2-32.1	5.4-19,20	5.4-19,20
5.2-33,34	5.2-33,34	5.4-21,21a	5.4-21,21.1
5.2-35,36	5.2-35,36	5.4-22,22a	5.4-22,22.1
5.2-37,38	5.2-37,38	5.4-23,24	5.4-23,24
5.2-41	5.2-41	5.4-25,26	5.4-25,26
5.2-42,43	5.2-42,43	5.4-27,28	5.4-27,28
5.2-44,45	5.2-44,45 Fig*	5.4-29,30	5.4-29,30
5.2-46,47	5.2-46,47 Fig*	5.4-31,31a	5.4-31,31.1
5.2-48,49	5.2-48,49 Fig*	5.4-32	5.4-32
5.2-50,51	5.2-50,51 Fig*	5.4-33,34	5.4-33,34
Add	5.2-52,53 Fig*	5.4-35,36	5.4-35,36
		5.4-37,38	5.4-37,38
5.3-1,2	5.3-1,2	5.4-39,40	5.4-39,40
5.3-2.1	5.3-2.1	5.4-41,42	5.4-41,42
5.3-2a	--	5.4-43	5.4-43
5.3-3,4	5.3-3,4	5.4-47,48	5.4-47,47.1 Fig*
5.3-5,6	5.3-5,6	Add	5.4-48 Fig*
5.3-6a	5.3-6.1	5.4-49,50	5.4-49,50
5.3-7,8	5.3-7,8	5.4-51,52	5.4-51,52 Fig*
5.3-9,10	5.3-9,10	Add	5.4-52.1 Fig*
5.3-11,11a	5.3-11,12	5.4-53,54	5.4-53,54 Fig*
5.3-12	--	5.4-55,56	5.4-55,56 Fig*
5.3-13,14	5.3-13,14	5.4-57,58	5.4-57,58 Fig*
		Add	5.4-58.1,58.2 Fig*
5.4-ii,iii	5.4-ii,iii	Add	5.4-58.3 Fig*
5.4-vi,vii	5.4-vi,vii	5.4-59,60	5.4-59,60 Fig*
5.4-1,2	5.4-1,2	5.4-61,62	5.4-61,62 Fig*
		5.4-63,64	5.4-63,64 Fig*





# ABWR Standard Plant

23A6100AB

REV. C

NO COMPONENTS ARE TYPICAL FOR THIS SHEET UNLESS SPECIFICALLY NOTED.

ON LINE 'C'

ON LINE 'C'

NOT TO SCALE FOR THE SIZE OF THE INLET OR OUTLET VALVE SERVICE.

THIS DRAWING AND APPROPRIATE CONNECTIONS WITH THE SUPPLIES THE PIPING REQUIREMENTS SHALL BE IN ACCORDANCE WITH THE REQUIREMENTS OF THE DESIGN AND PROCESS DRAWING.

THIS MAY NOT BE PREVENTED IF A VALVE WITH THE LINE TRANSMITTER SHUT-OFF VALVE IS PART OF THE LINE FOR THE INSTRUMENT.

THIS IS TO BE STRAIGHT RUN OF PIPE WITH SUFFICIENT HEIGHT LENGTH FROM THE TAP TO THE INSTRUMENT MEASUREMENT AS TO MEET SOME PFC & S&H PARAGRAPHS 4.7.4.

THIS IS PROVIDED IN THE INSTRUMENT LINE TO NOT BEYOND THE WATER TIGHT SEAL AND BEYOND THE CONNECTION JOINT SHALL BE DESIGNED TO ALLOW FOR VIBRATION LENGTH WITH THERMAL EXPANSION. THE SEAL OR CHANGE TO THE VESSEL ELEVATION. A SHALL BE MADE AT THE VESSEL HEAD JOINT. THE ELEVATION IS TO BE KEPT TO A MINIMUM OF 9 INCHES AND IS TO BE KEPT TO A MINIMUM OF 9 INCHES. THE INSTRUMENT LINE SHALL BE KEPT PROTECTED WITH THE VESSEL HEAD JOINT. THE INSTRUMENT LINE SHALL HAVE A MINIMUM OF 9 INCHES OF THERMAL EXPANSION.

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THIS AS POSSIBLE TO

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THIS WATER LEVEL INSTRUMENTS FOR VARIOUS RANGES ARE CALIBRATED AS STATED BELOW. ALL WATER LEVEL SWITCHES SHOULD BE NORMAL. ALL THE ANALYSES ARE PERFORMED WITH THE SWITCH TOP CASUALTY INCLUDED. THE CONTAINMENT BUILDING TEMPERATURE ASSUMED TO BE 30.7°C.

A. FUEL ZONE THE INSTRUMENTS ARE CALIBRATED FOR SATURATED WATER AND STEAM CONDITIONS AT 0 MPa(g) IN THE VESSEL AND 0 MPa(g) IN THE STEAM LINE.

B. PRESSURE RANGE: 0 MPa(g) TO 15 MPa(g) IN THE VESSEL, 0.2 MPa(g) TO 15 MPa(g) IN THE STEAM LINE. THE INSTRUMENTS ARE CALIBRATED FOR SATURATED WATER AND STEAM CONDITIONS AT 0 MPa(g) IN THE VESSEL AND 0.2 MPa(g) IN THE STEAM LINE.

C. NUCLEAR RANGE: 0 MPa(g) TO 15 MPa(g) IN THE VESSEL, 0.2 MPa(g) TO 15 MPa(g) IN THE STEAM LINE. THE INSTRUMENTS ARE CALIBRATED FOR SATURATED WATER AND STEAM CONDITIONS AT 0 MPa(g) IN THE VESSEL AND 0.2 MPa(g) IN THE STEAM LINE.

D. SHUTDOWN RANGE: 0 MPa(g) TO 15 MPa(g) IN THE VESSEL, 0.2 MPa(g) TO 15 MPa(g) IN THE STEAM LINE. THE INSTRUMENTS ARE CALIBRATED FOR SATURATED WATER AND STEAM CONDITIONS AT 0 MPa(g) IN THE VESSEL AND 0.2 MPa(g) IN THE STEAM LINE.

E. THE TEMPERATURE ELEMENT MP. 821-10333 MAY BE LOCATED ON THE TOP HEAD VENT LINE BETWEEN THE MOTOR-OPERATED VALVE MP. 821-10333 AND 821-10334 PROVIDING THE FOLLOWING CONDITIONS ARE MET: THE TEMPERATURE ELEMENT MP. 821-10333 SHALL NOT BE INFLUENCED BY THE POTENTIALLY HIGH TEMPERATURES UPSTREAM OF THE MOTOR-OPERATED VALVE MP. 821-10333 WHEN THERE IS LEAK LEAKAGE THROUGH THE MOTOR-OPERATED VALVE MP. 821-10333.

F. UNLESS OTHERWISE INDICATED, ALL REFERENCES MP. ARE PROVIDED BY S&H.

G. SEE REFERENCE DOCUMENT 1 FOR THE SPECIAL DESIGN REQUIREMENTS WHICH ARE APPLICABLE TO THE PIPING BETWEEN THE STEAM LINE INBOARD AND OUTBOARD CONTAMINANT ISOLATION VALVES.

H. SEE REFERENCE DOCUMENT 2 FOR THE SPECIAL DESIGN REQUIREMENTS WHICH ARE APPLICABLE TO THE PIPING BETWEEN THE FEEDWATER LINE INBOARD AND OUTBOARD CONTAMINANT ISOLATION VALVES.

I. OPERATION OF 2 OF 3 MANUAL SWITCHES IS REQUIRED FOR SHUTDOWN OPERATION OF THE 3 SW'S USED FOR THE ADS.

J. SEE SUPPORTING DOCUMENT 1 FOR SYSTEM IDENTIFICATION AT INTERCONNECTIONS.

K. PNEUMATIC SUPPLY FROM REFERENCE DOCUMENT 33.

L. THE CONDENSING CHAMBER SHALL BE KEPT TO A MINIMUM OF 9 INCHES OF THERMAL EXPANSION. THE CONDENSING CHAMBER SHALL BE KEPT TO A MINIMUM OF 9 INCHES OF THERMAL EXPANSION.

M. THE MOTOR-OPERATED VALVE MP. 821-10333 SHALL BE KEPT TO A MINIMUM OF 9 INCHES OF THERMAL EXPANSION.

N. THE THERMAL SLEEVE SHALL BE KEPT TO A MINIMUM OF 9 INCHES OF THERMAL EXPANSION.

O. SEE REFERENCE DOCUMENT 3 FOR THE INSTRUMENT SETPOINT REQUIREMENTS.

P. FOR INTERFACE CONNECTIONS, SEE THE NEW EQUIPMENT REQUIREMENTS SPECIFICATION SUPPORT DRAWING MP. 1008 AND 1009.

Q. FOR VENT AND DRAIN LINES OPEN TO THE ATMOSPHERE, DOWNSTREAM OF THE OUTBOARD SHUT-OFF VALVE, THE FOLLOWING BOUNDARY CONDITIONS APPLY:  
MAXIMUM OPERATING PRESSURE - 0 MPa(g)  
MAXIMUM OPERATING TEMPERATURE - 300°C  
MINIMUM OPERATING TEMPERATURE - 0°C  
SEISMIC CLASS - 0

R. PIPING DESIGN SPECIFICATION AS FOLLOWS:  
A. MAXIMUM OPERATING PRESSURE - SEE SPECIFIC BOUNDARIES ON DRAWING  
B. MAXIMUM OPERATING TEMPERATURE - SEE SPECIFIC BOUNDARIES ON DRAWING  
C. MATERIAL - SEE TABLE B  
D. PIPING THICKNESS - SEE TABLE B  
E. WTI CLASS - SEE SPECIFIC BOUNDARIES ON DRAWING  
F. GC CLASS - SEE SPECIFIC BOUNDARIES ON DRAWING  
G. SEISMIC CLASS - SEE SPECIFIC BOUNDARIES ON DRAWING  
H. FLUID - SEE TABLE B

S. THE RELIEF VALVE PIPING IS NOT REQUIRED IF INTERNAL PROTECTION IS PROVIDED WITHIN 1 FOOT TO LIMIT THE DIFFERENTIAL PRESSURE ACROSS THE RELIEF ELEMENT.

T. THE INSTRUMENT LINES SHALL BE SEISMIC CLASS AS FROM THE LAST ANCHOR POINT TO THE PRESSURE TRANSDUCERS PIPING AND PIPING.

U. PROVIDES INTERFACE BETWEEN SEISMIC CATEGORY 1 AND NON-SEISMIC CATEGORY PIPING.

V. THE 4 INCH OPERATING TEMPERATURE OF THE FEEDWATER TAP - 300°C THE SEISMIC RISK TO THE UPSTREAM SIDE OF THE MOTOR OPERATED VALVE MP. 821-10333 SHALL BE DETERMINED BY THE DESIGNER OF THE FEEDWATER SYSTEM.

REFERENCE DOCUMENT UNDER THE FOLLOWING IDENTIFIERS ARE TO BE USED IN CONNECTION WITH THIS DRAWING.

MP. NO.	MP. NO.
1. WATER QUALITY REQUIREMENTS	AP-2040
2. REACTOR SYSTEM, CO	AP-2030
3. NUCLEAR BOILER SYS. P&ID DATA	821-1030
4. NUCLEAR BOILER SYSTEM P&ID	821-1030
5. NUCLEAR BOILER SYSTEM, BD	821-1030
6. NUCLEAR BOILER SYSTEM DESIGN BOND	821-1030
7. NUCLEAR BOILER DESIGN SPEC	821-1030
8. FEEDWATER PIPING DESIGN SPEC	821-1030
9. CONTROL ROD DRIVE SYSTEM P&ID	821-1030
10. REACTOR REGENERATION SYSTEM P&ID	821-1030
11. FEEDWATER CONTROL SYSTEM, BD	821-1030
12. FEEDWATER CONTROL SYSTEM, ED	821-1030
13. REACTOR SHUTDOWN SYSTEM, ED	821-1030
14. REACTOR PROTECTION SYSTEM, ED	821-1030
15. REGENERATION FLOW CONTROL SYSTEM, BD	821-1030
16. REGENERATION FLOW CONTROL SYSTEM, ED	821-1030
17. REGENERATION HEAT REMOVAL SYSTEM P&ID	821-1030
18. REGENERATION HEAT REMOVAL SYSTEM, BD	821-1030
19. HIGH PRESSURE CORE FLOODED SYSTEM, BD	821-1030
20. LEAK DETECTOR AND ISOLATION SYSTEM, BD	821-1030
21. LEAK DETECTOR AND ISOLATION SYSTEM, ED	821-1030
22. REACTOR CORE ISOLATION COOLING SYSTEM P&ID	821-1030
23. REACTOR CORE ISOLATION COOLING SYSTEM, BD	821-1030
24. REACTOR WATER CLEANUP SYSTEM P&ID	821-1030
25. LIQUID WASTE SYSTEM P&ID	821-1030
26. LOW CONDUCTIVITY WASTE SYSTEM P&ID	821-1030
27. TURBINE NUCLEAR SYSTEM P&ID	821-1030
28. CONDENSATE AND FEEDWATER SYSTEM P&ID	821-1030
29. TURBINE CONTROL SYSTEM, BD	821-1030
30. TURBINE CONTROL SYSTEM, ED	821-1030
31. STEAM BYPASS & PRESSURE CONTROL SYSTEM, ED	821-1030
32. NUCLEAR CONDENSER	821-1030
33. INSTRUMENT AIR SYSTEM P&ID	821-1030
34. HIGH PRESSURE NUCLEAR GAS SUPPLY SYSTEM	821-1030
35. VALVE ISLAND LEAKAGE TREATMENT SYSTEM	821-1030
36. SAMPLING SYSTEM	821-1030
37. PRIMARY CONTAMINANT SYSTEM P&ID	821-1030
38. NUCLEAR STEAM ISOLATION VALVE LEAK TEST FACILITY P&ID	821-1030
39. ATMOSPHERIC CONTROL SYSTEM P&ID	821-1030
40. NUCLEAR STEAM PIPING ELEMENT REQUIREMENTS SPECIFICATION SUPPORT DRAWING	821-1030

XX DENOTES THAT THE COMPONENT IS PART OF AN ASSEMBLY WHERE THE ENTIRE ASSEMBLY HAS ONE MP. NUMBER.

YY REFERENCE INFORMATION TO BE PROVIDED AS INTERFACE DATA NOT AFFECT THE DESIGN INFORMATION SHOWN ON THE DRAWING OR ITS VERIFICATION.

### SUPPORTING DOCUMENTS

MP. NO.	MP. NO.
1. NUCLEAR PLANT SYSTEM STRUCTURE	A10-3030
2. PIPING AND INSTRUMENT SYMBOLS	A10-3030
3. GROUP CLASSIFICATION AND CONTAMINANT ISOLATION DIAGRAM	A10-3030
4. PROCESS INSTRUMENTATION REQUIREMENT SPEC	A10-3030

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CARD

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MP. NO. 821-1030

Figure 5.1-3 NUCLEAR BOILER SYSTEM P&amp;ID, Sheet 1 of 11

# ABWR Standard Plant

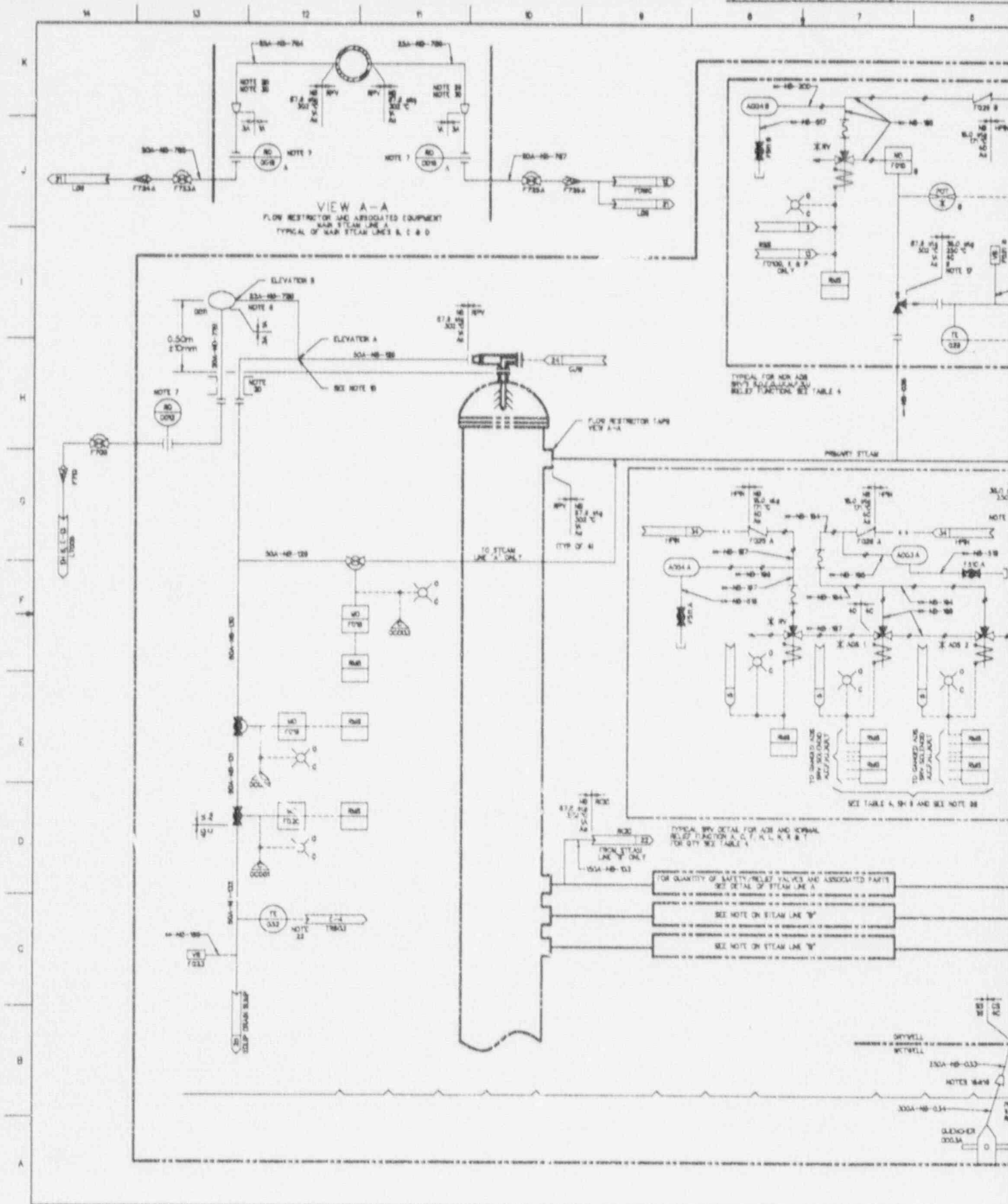
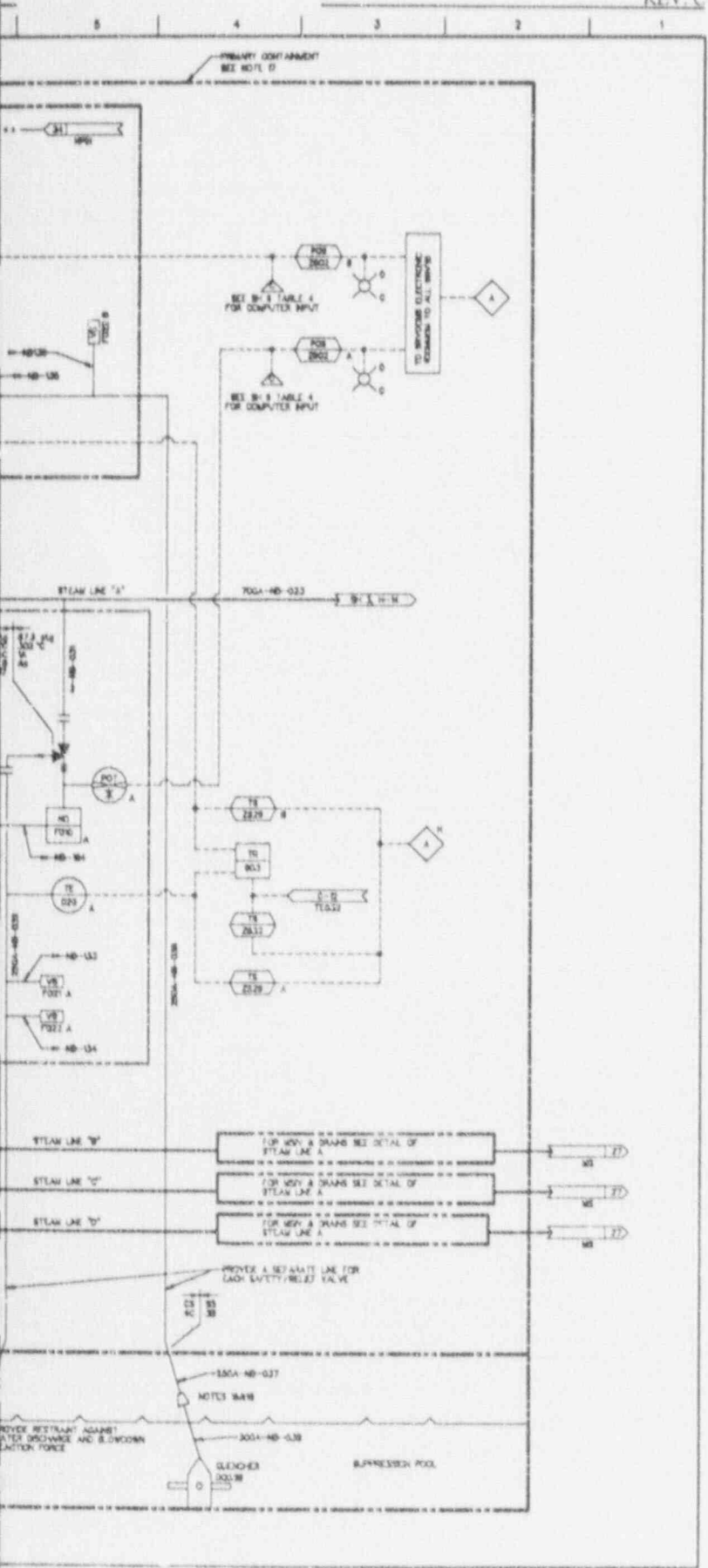
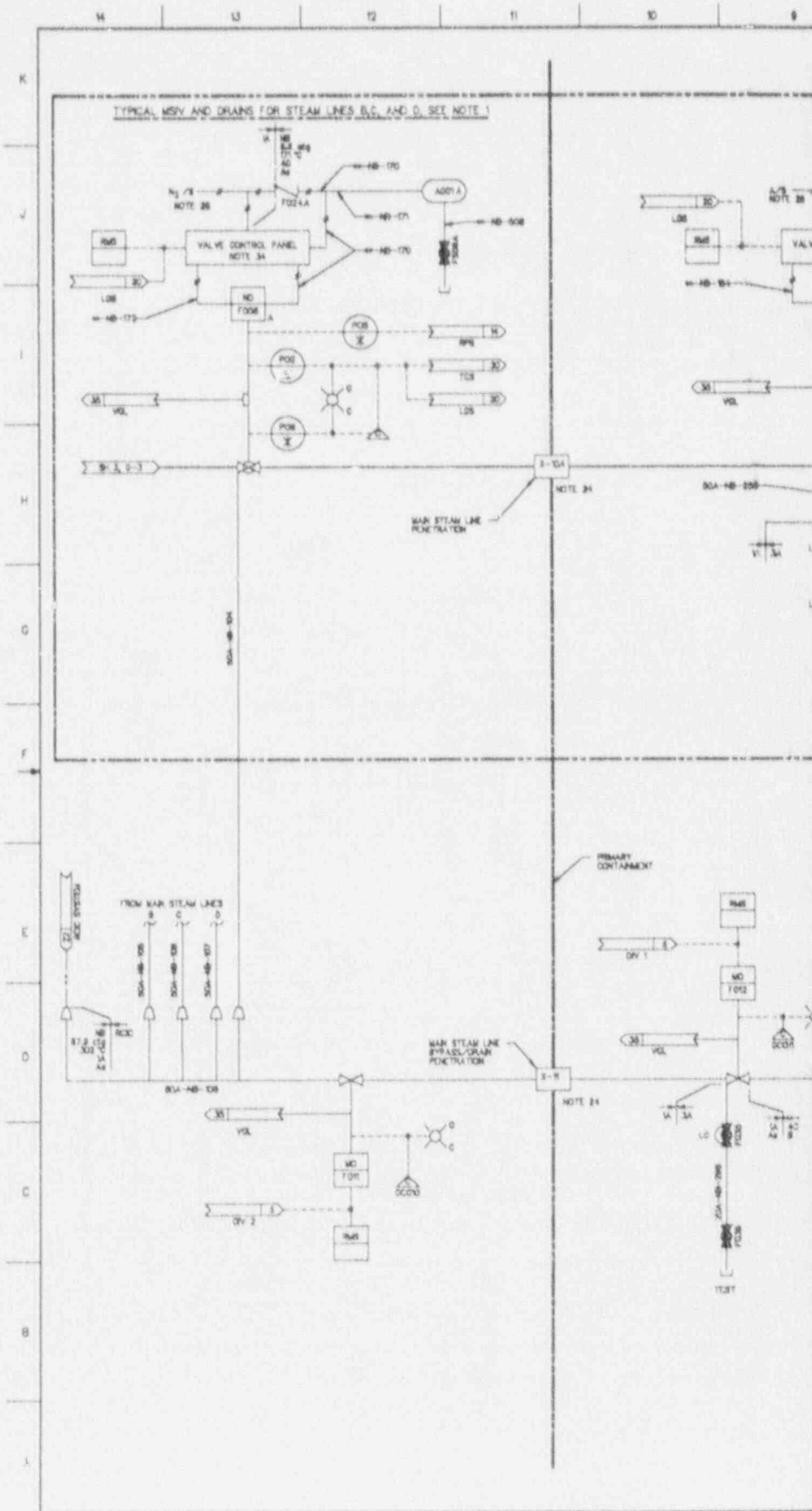


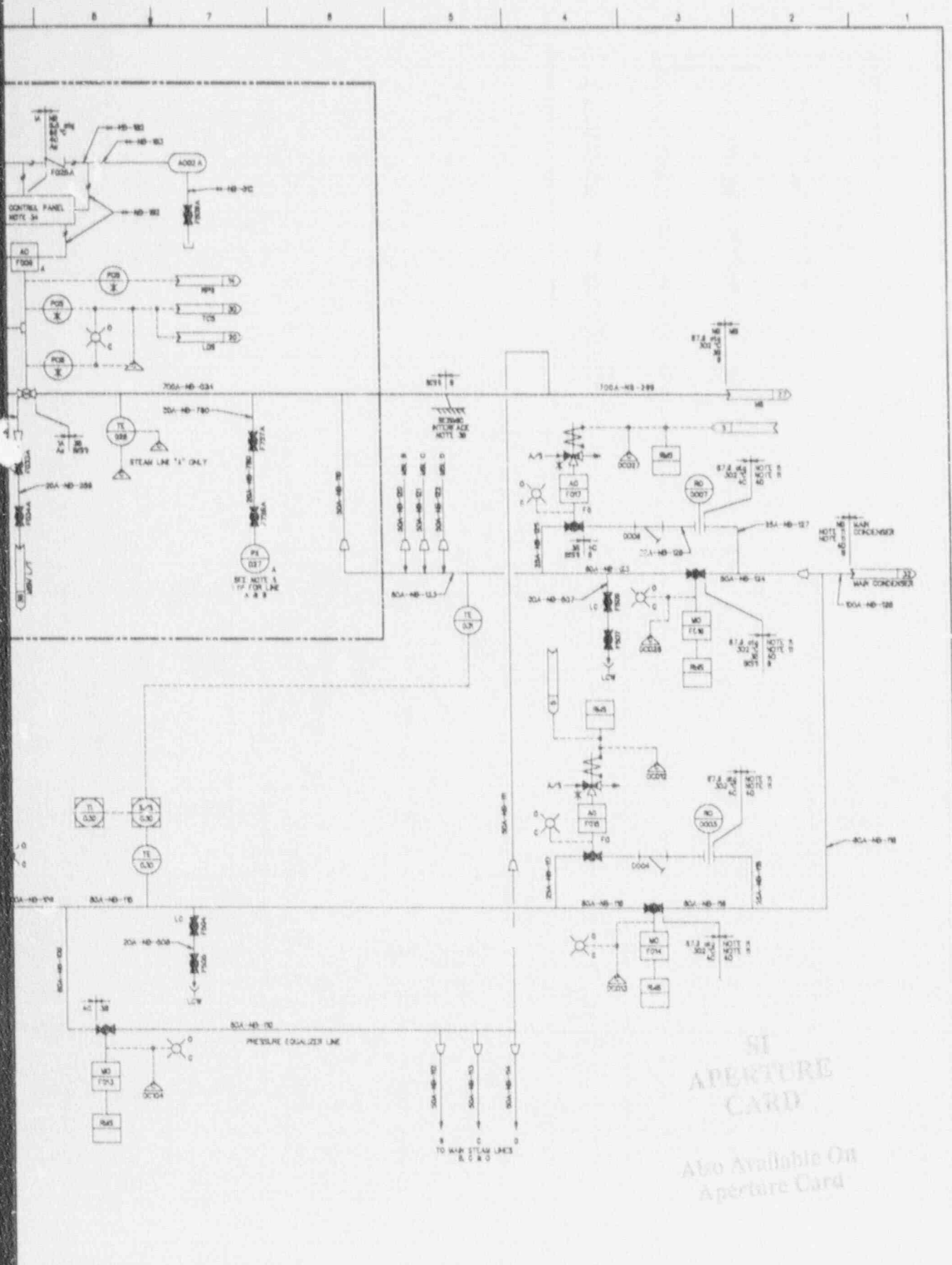
Figure 5.1-3



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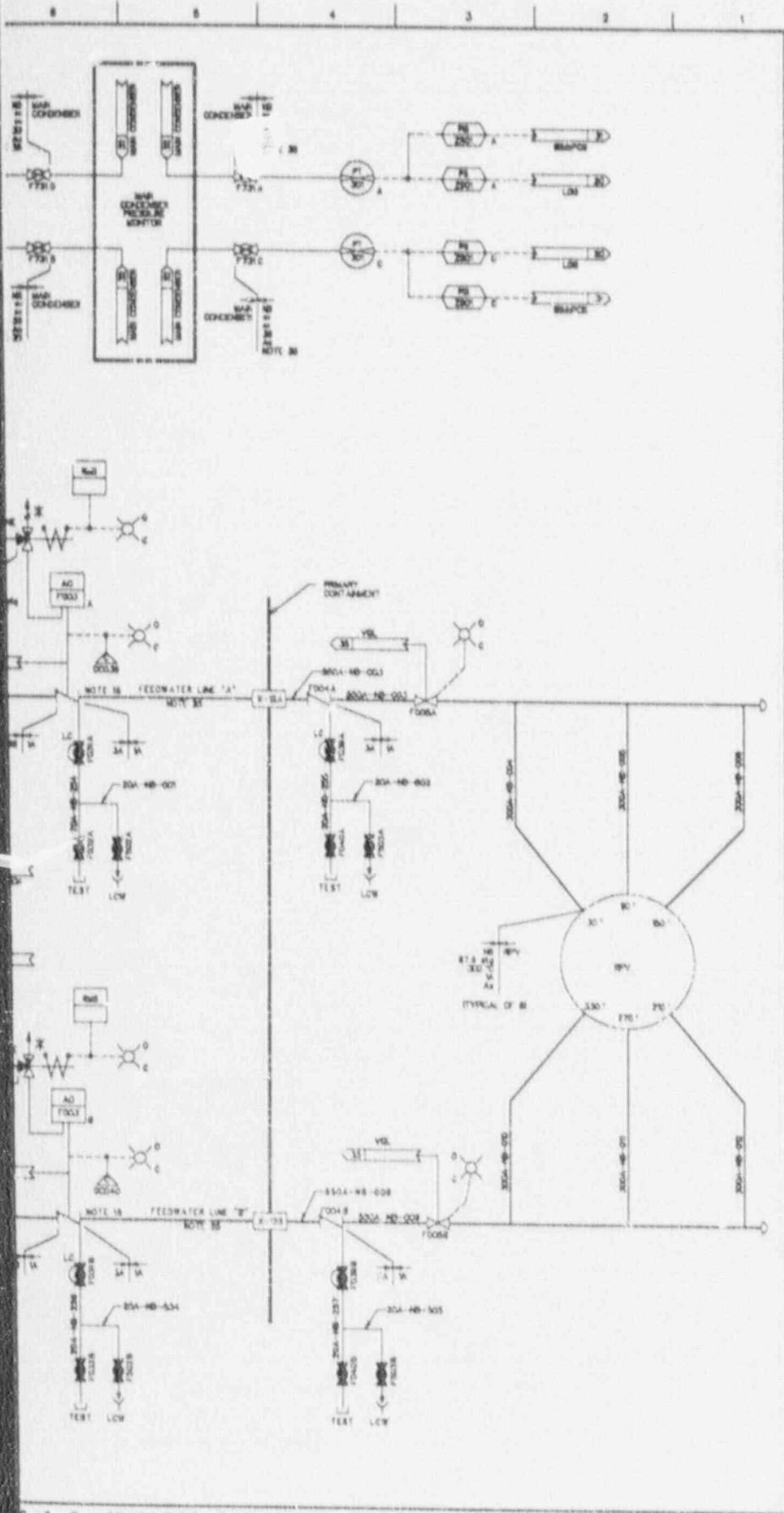




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Figure 5.1-3 NUCLEAR BOILER SYSTEM P&ID, Sheet 3 of 11





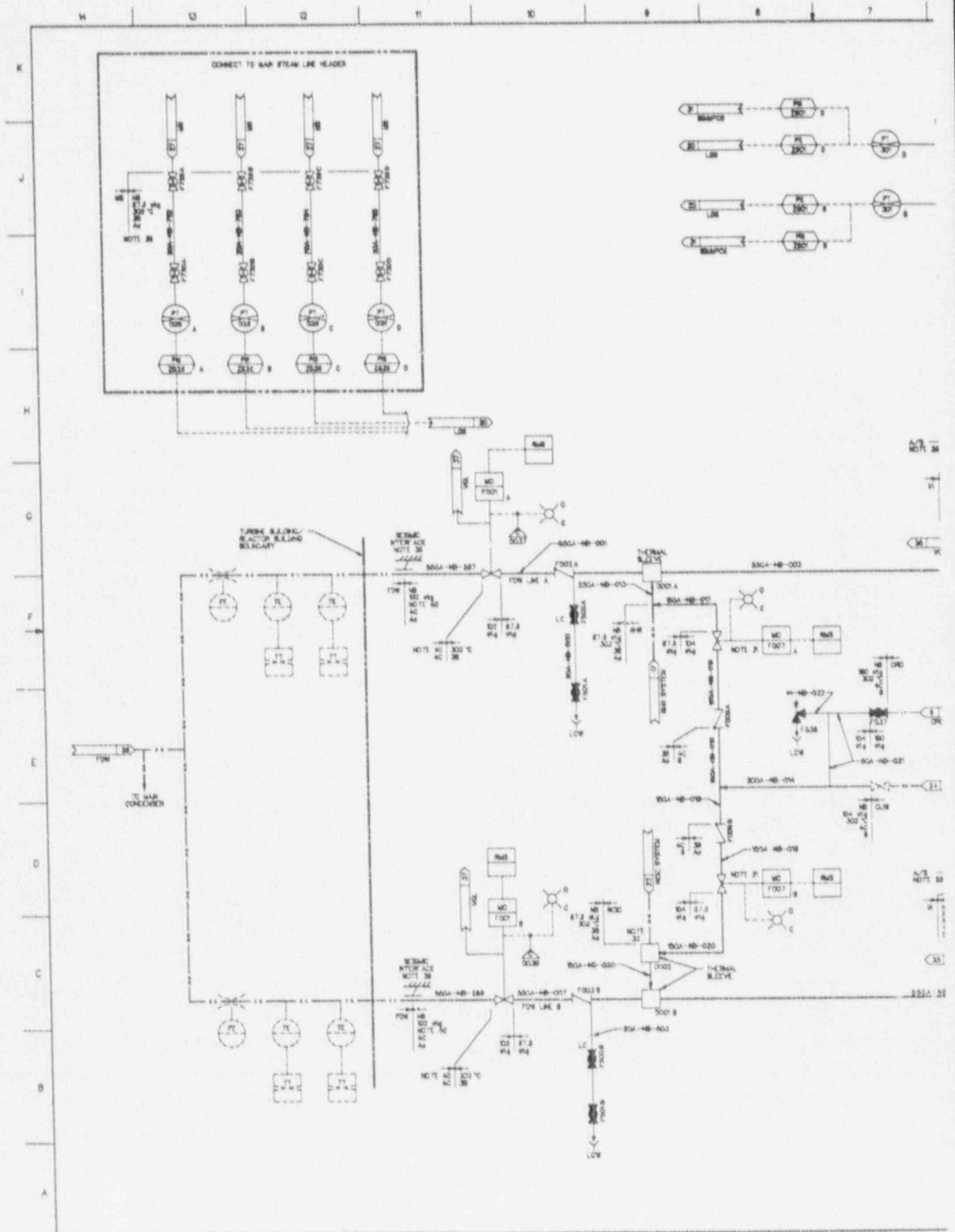
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6.1-3 NUCLEAR BOILER SYSTEM P&ID, Sheet 4 of 11

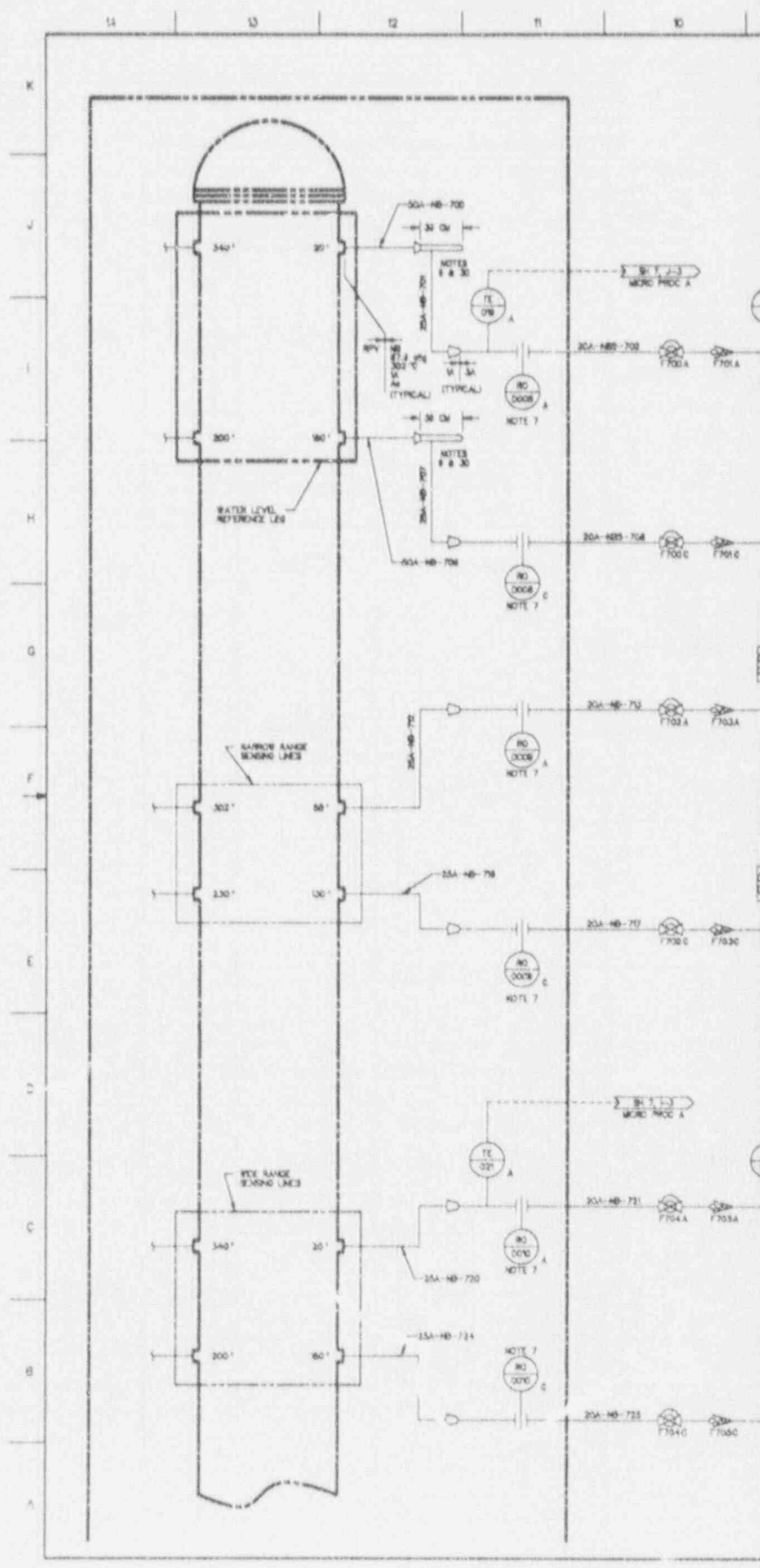
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# ABWR Standard Plant



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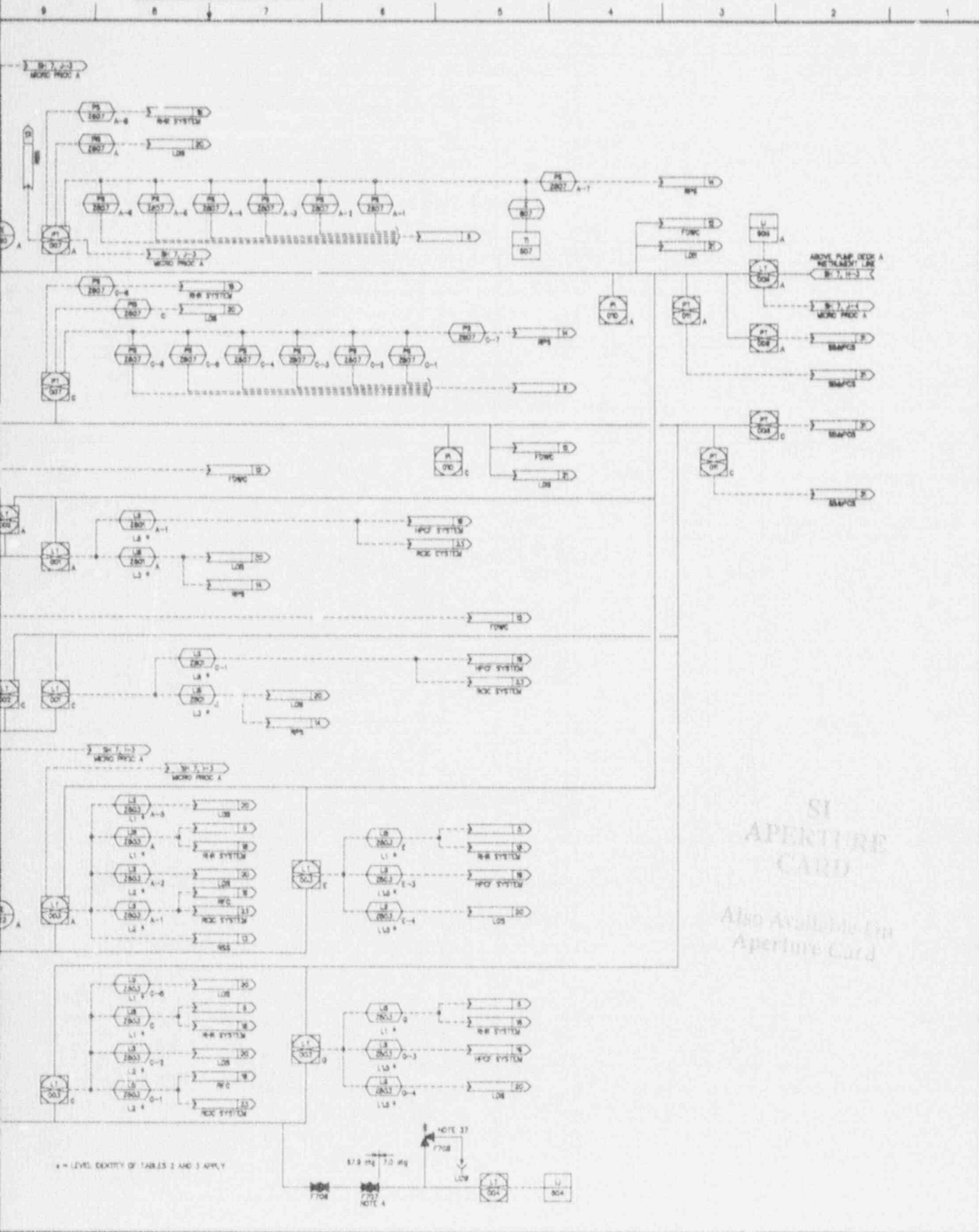


Figure 5.1-3 NUCLEAR BOILER SYSTEM P&ID, Sheet 5 of 11

# ABWR Standard Plant

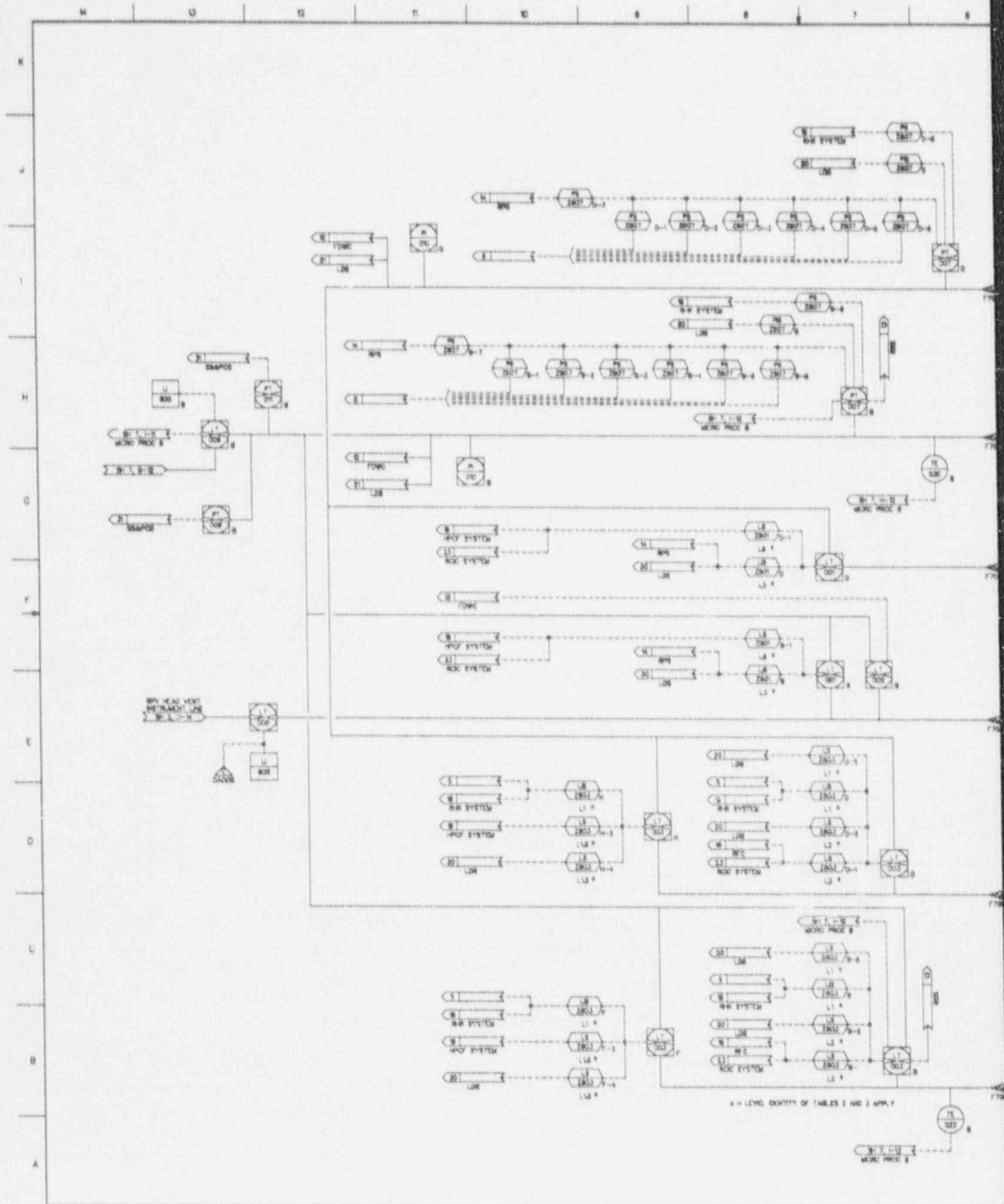
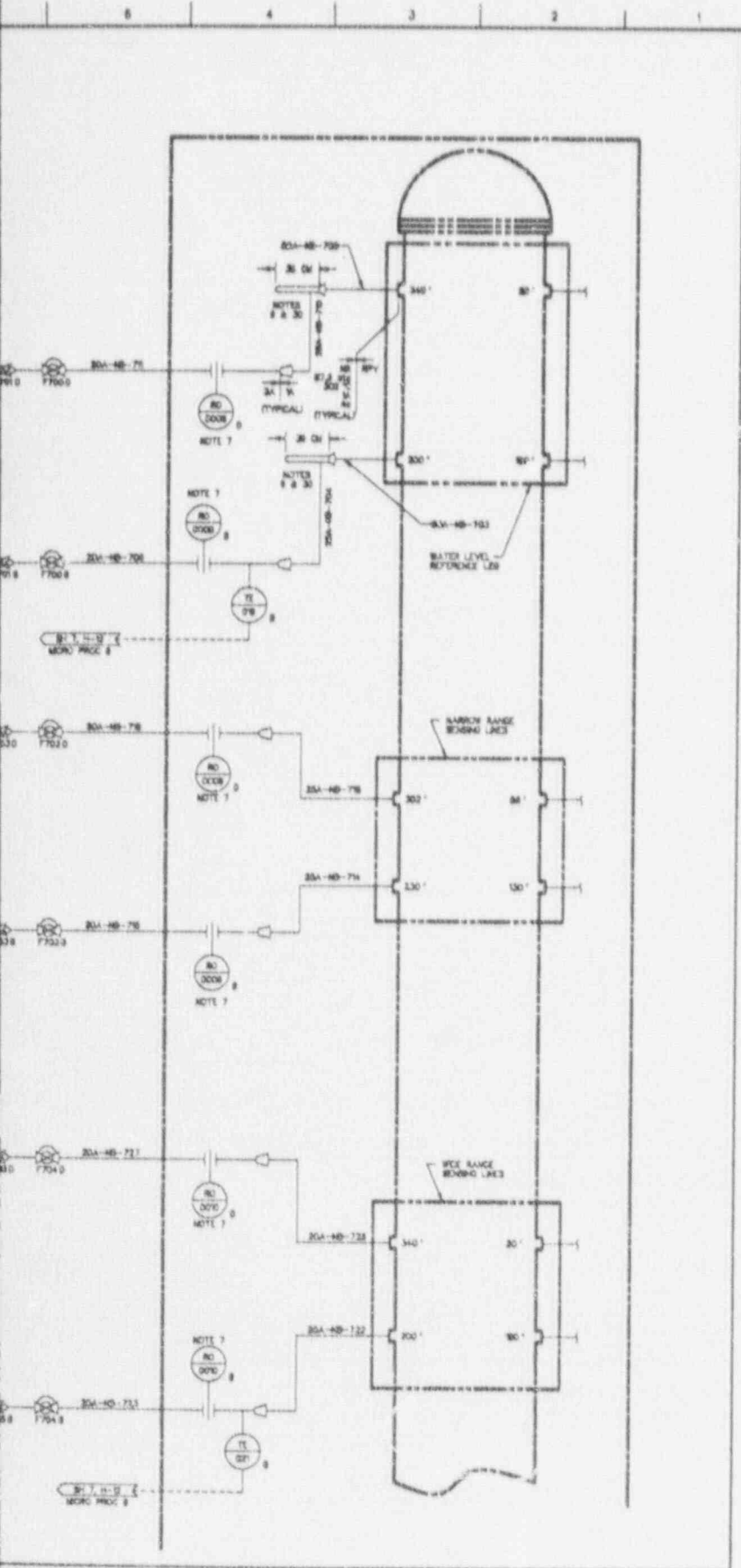
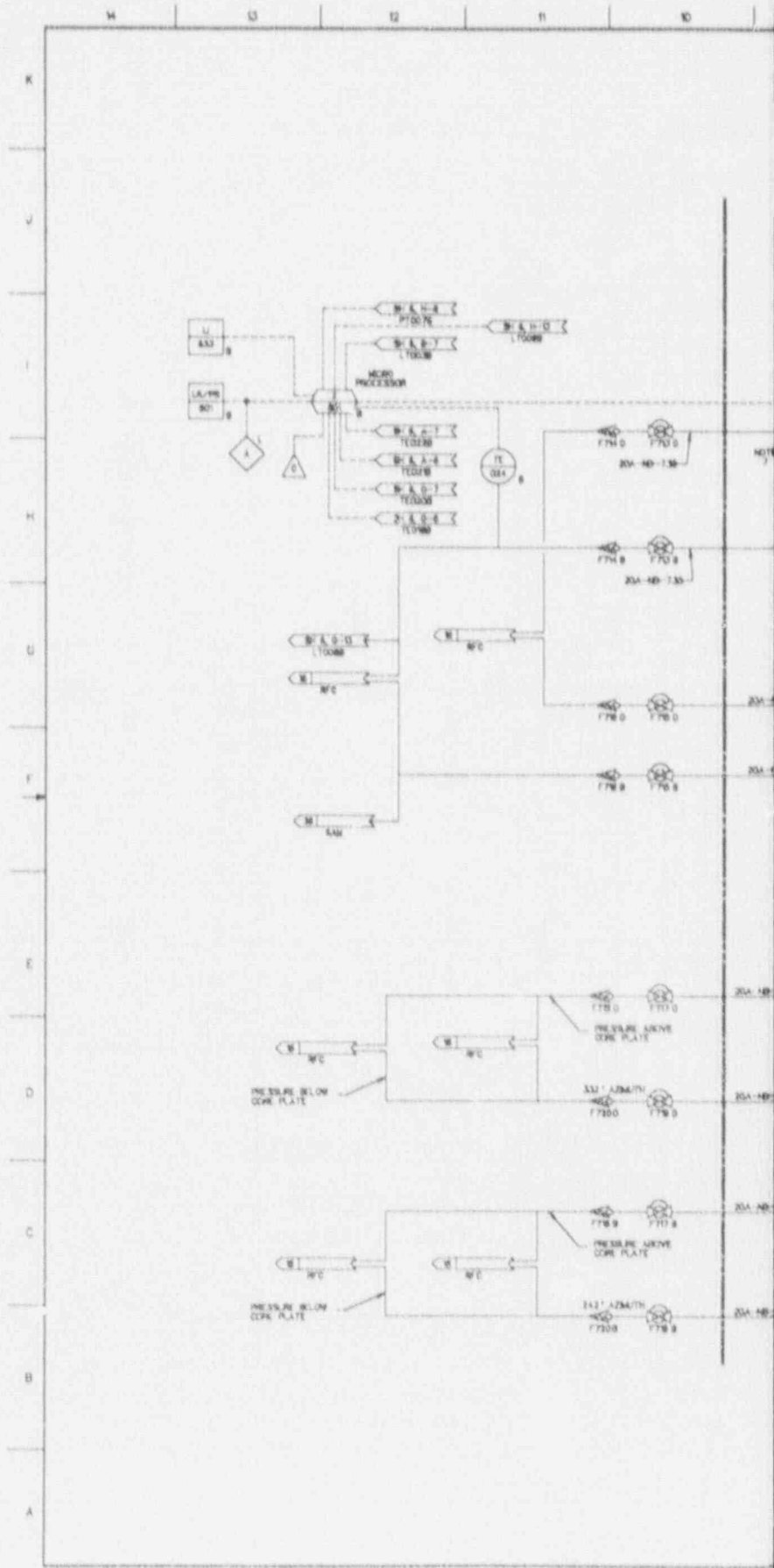


Figure 5.1-3



ST  
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CARD  
Also Available on  
Aperture Card





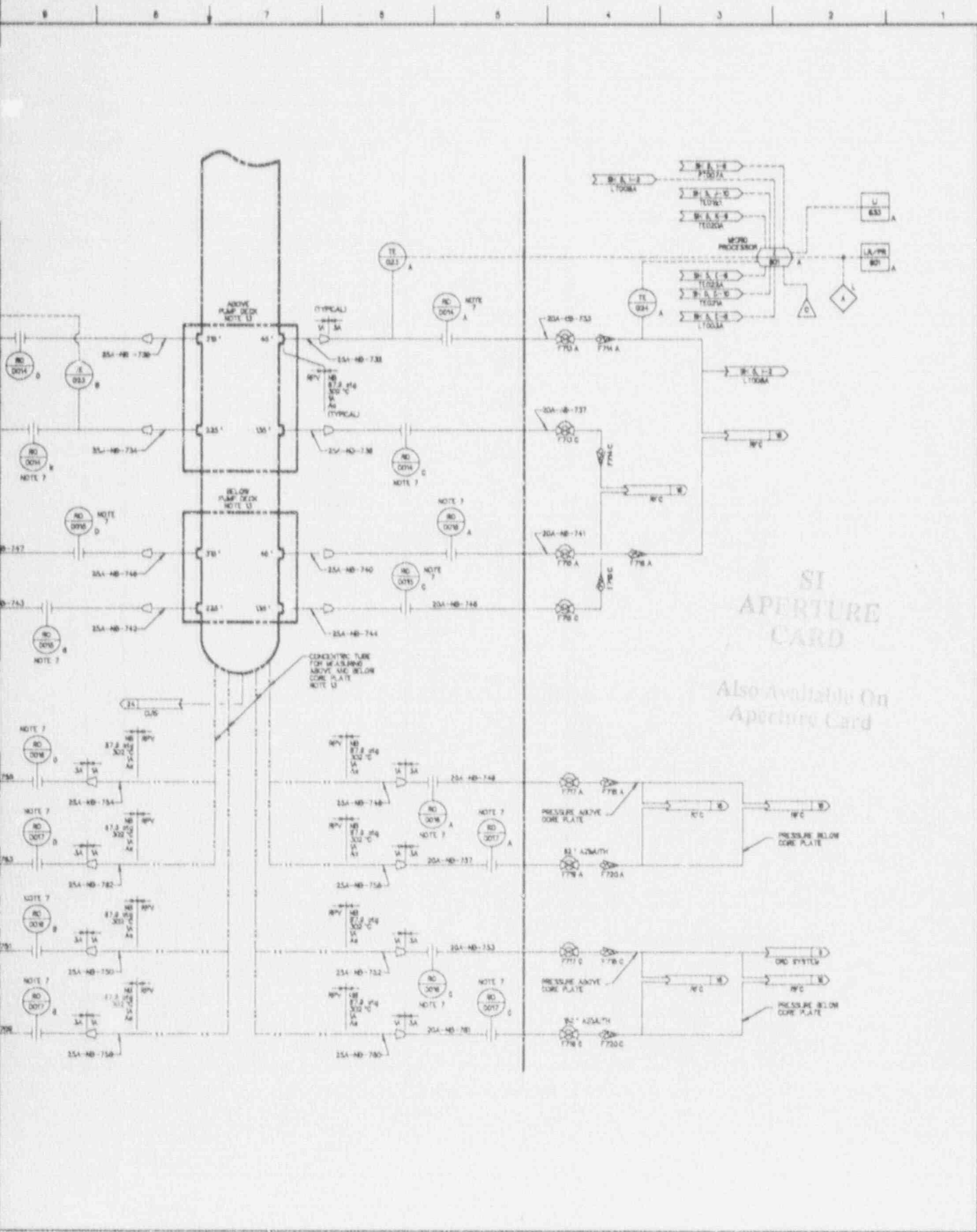


Figure 5.1-3 NUCLEAR BOILER SYSTEM P&ID, Sheet 7 of 11

# ABWR Standard Plant

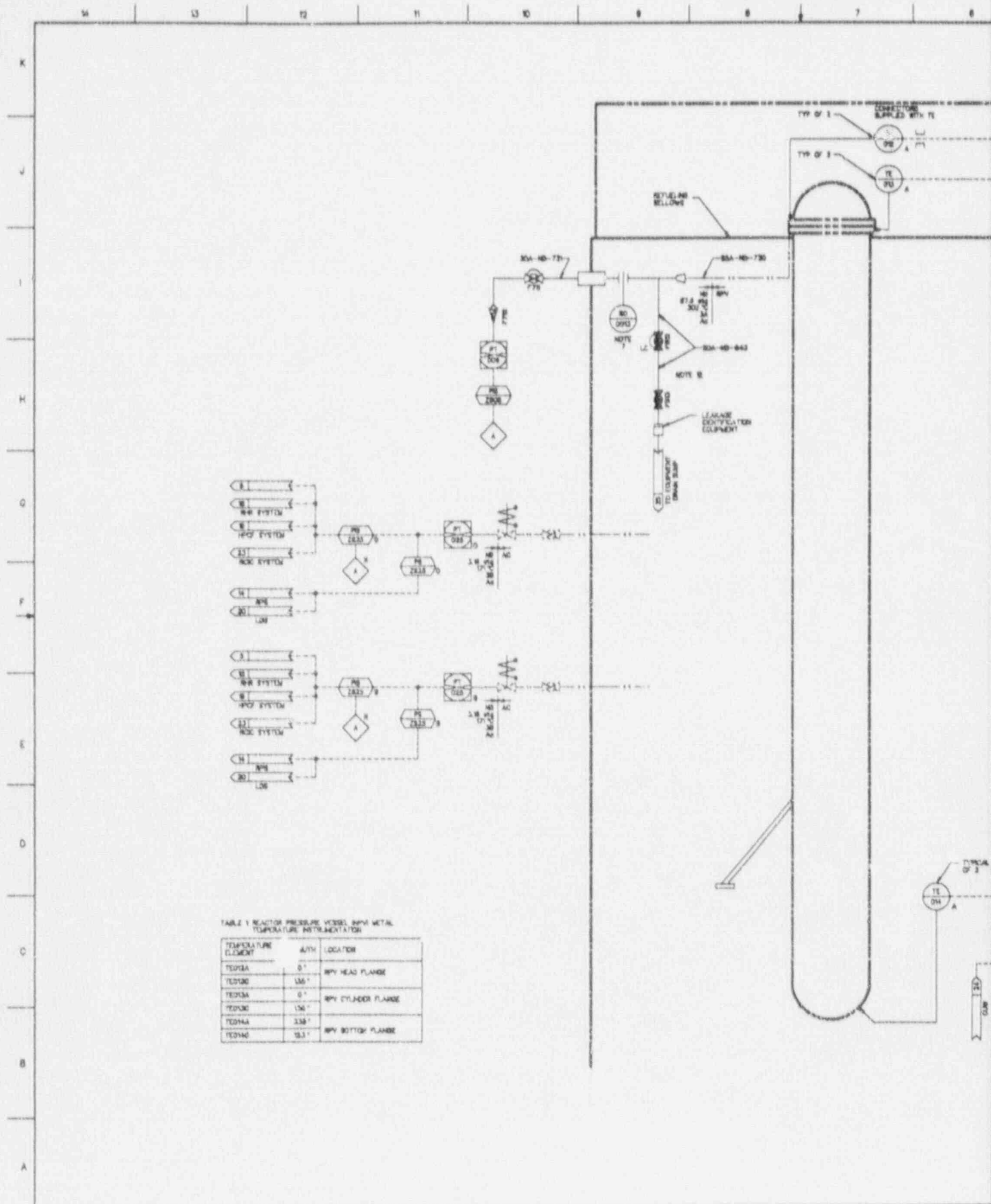
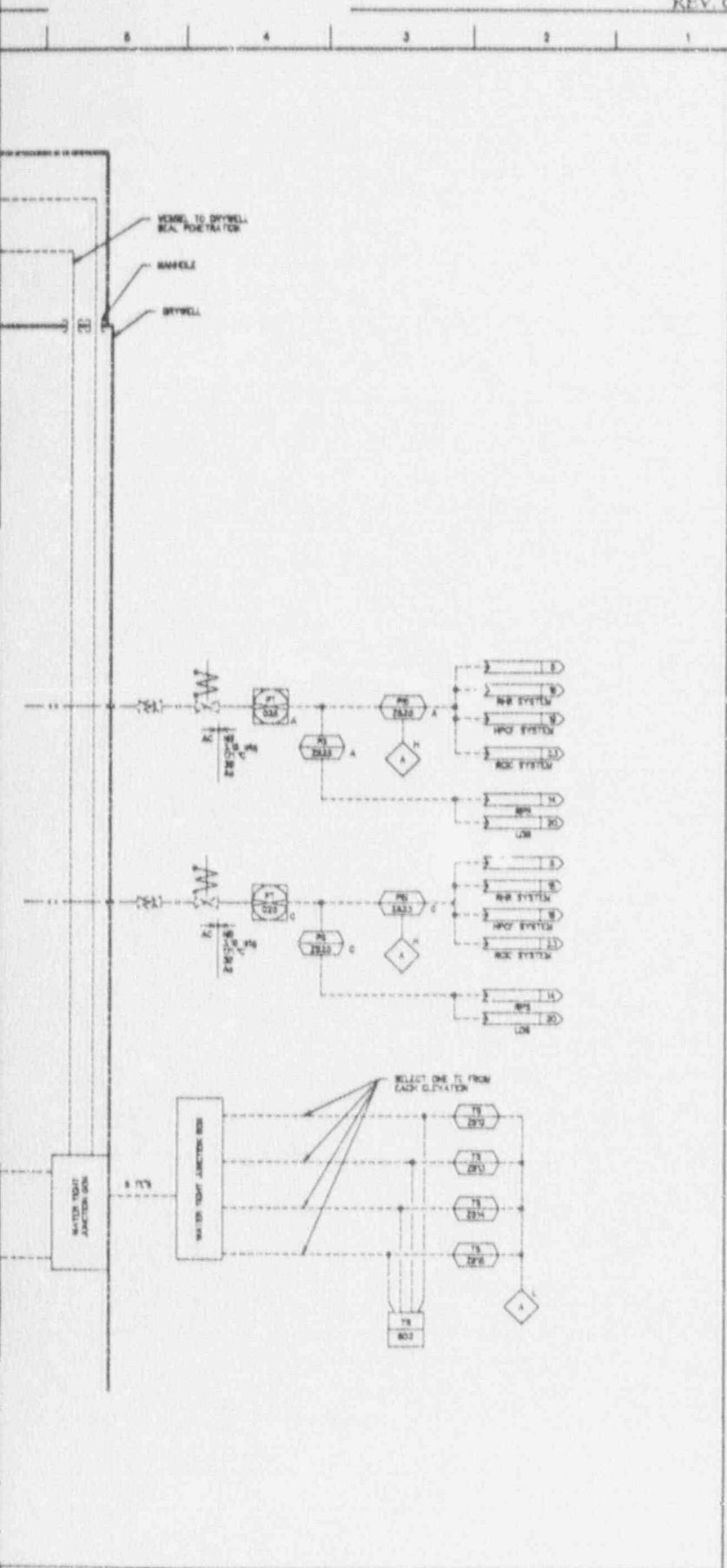


TABLE 1 REACTOR PRESSURE VESSEL RPV METAL TEMPERATURE INSTRUMENTATION

TEMPERATURE ELEMENT	DEPTH	LOCATION
TE010A	0"	RPV HEAD FLANGE
TE010B	156"	
TE010C	0"	RPV CYLINDER FLANGE
TE010D	156"	
TE010E	158"	
TE010F	163.5"	RPV BOTTOM FLANGE

Figure 5.1-3



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TABLE 3. BUYS LETTER ASSIGNMENTS FOR SAFETY/RELIEF VALVES AND ASSOCIATED EQUIPMENT

SAFETY/RELIEF VALVE TYPE	F	J	M	O	S	X	E	U	D	K
TEMPERATURE ELEMENT THERM										
BLINDING VALVE										
ACCUMULATOR										
CHECK VALVE										
BRY OPER										
MON/DRIVE										
COMPUTER INPUT										
SPRING SET PRESSURE (psia)										
RELIEF SET PRESSURE (psia)										
RELIEF BEYOND PRESSURE (psia)										
PT007A THRU D/ PS 2807A-4 THRU D-4										
PT007A THRU D/ PS 2807A-2 THRU D-8										
PT007A THRU D/ PS 2807A-4 THRU D-4										
PT007A THRU D/ PS 2807A-2 THRU D-3										
PT007A THRU D/ PS 2807A-1 THRU D-2										
PT007A THRU D/ PS 2807A-1 THRU D-1										

\* COMPUTER INPUTS FOR BRY PORTION SEE PERFORMANCE MONITORING AND CONTROL SYSTEM C81-4000

TABLE 4. ELEVATOR CORRELATION CHART

REFERENCE	EOL VESSEL NOSES ABOVE VESSEL ZERO	REFERENCE	REACTOR VESSEL WATER LEVEL (SEE TABLE 3)	CONTROL ROOM WATER LEVEL INDICATOR AND SAFEGUARDS (SEE NOTE 2)		
				PORT ACCEPT MONITOR FUEL LINE RANGE	SAFEGUARDS	
					WIDE RANGE	NARROW RANGE
				L: 808.488	W: 280.148 C.V. 7.148	US 280.148 N.C. 8.00
INSTRUMENT LINE NOZZLE	210.8 in	TOP ANKLE OF HEAD				
	183.8 in					
	155.4 in	MAIN STEAM LINE NOZZLES			180.4 in	
INSTRUMENT LINE NOZZLE (NARROW RANGE)			138.3 in			308.0 in 308.0 in
			135.8 in			181.4 in 181.4 in
		W-ALARM NORMAL WATER LEVEL LOW ALARM	130.3 in			148.8 in 148.8 in
			127.3 in			133.8 in 133.8 in
		SCFAXATOR	122.2 in			280.8 in 280.8 in
		BOTTOM OF OFFICE VENT	122.0 in			333.0 in 333.0 in
INSTRUMENT LINE NOZZLE (WIDE RANGE)			188.1 in			283.2 in
			122.0 in			
			103.0 in			18.1 in
INSTRUMENT LINE NOZZLE (WIDE RANGE)		TOP OF THE ACTIVE FUEL TRAY	83.8 in			26.7 in
			82.8 in			0 in
UPPER PUMP DECK LOWER PUMP DECK		UPPER INSTRUMENT LINE NOZZLES LOWER				28.0 in
			90.5 in			
		BOTTOM HEAD	0 in			

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821	822	823	824	825	826	827	828
829	830	831	832	833	834	835	836
837	838	839	840	841	842	843	844
845	846	847	848	849	850	851	852
853	854	855	856	857	858	859	860
861	862	863	864	865	866	867	868
869	870	871	872	873	874	875	876
877	878	879	880	881	882	883	884
885	886	887	888	889	890	891	892
893	894	895	896	897	898	899	900

SHUTDOWN	FOR IN USE
L 805	L 804
1800.0 mm	1800.0 mm
1832.0 mm	1800.0 mm
355.5 mm	355.5 mm
0 mm	0 mm

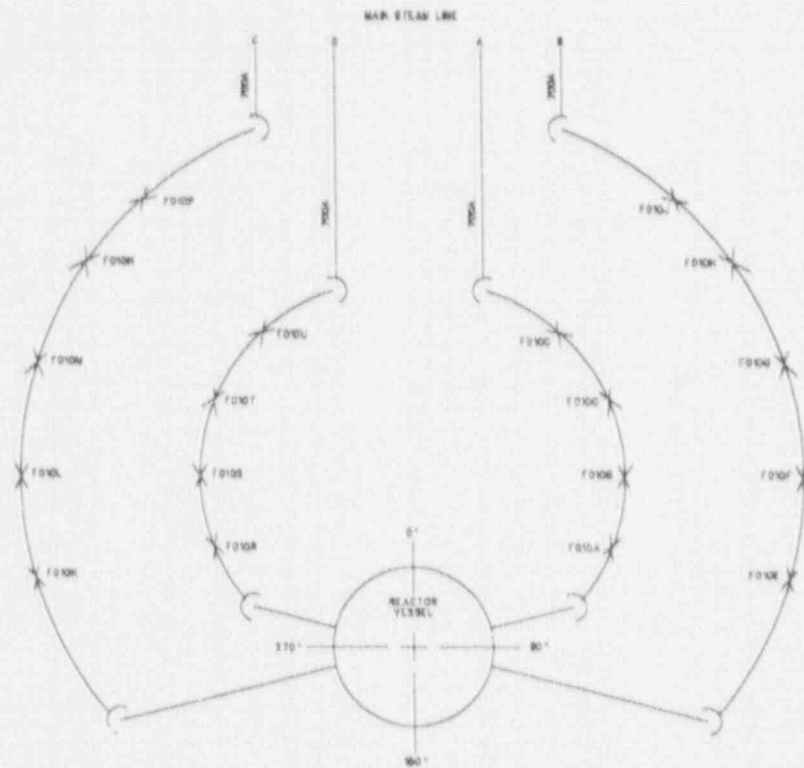


FIG. 3  
SAFETY/RELIEF VALVE ORIENTATION  
AND STEAM PIPING LINE SIZING

TABLE 3 WATER LEVEL TRIP FUNCTION

REACTOR VESSEL WATER LEVEL	DESCRIPTION OF TRIPS	INITIATE PREVENTING TRIP SIGNAL	NOTES
8	TRIPS F00 FLOWLINE TRIP HPC/ BLEEDER VALVES CLOSE MAIN TURBINE STOP VALVES TRIPS FEEDWATER PUMPS	LS-280A-1 THRU D-1 LS-280A-1 THRU D-1 SEE REFERENCE DOCUMENT 12	NARROW RANGE NARROW RANGE NARROW RANGE
7	HIGH LEVEL ALARM	SEE REFERENCE DOCUMENT 12	NARROW RANGE
4	LOW LEVEL ALARM AND FLOW ALARM BOUNDARY OF FEED PUMPS	SEE REFERENCE DOCUMENT 12	NARROW RANGE
3	SCRAMS REACTOR CLOSE REB SHUTDOWN COOLING ISOLATION VALVES CLOSE CONTAINMENT ISOL VALVES (EXCEPT ON COOLING AND GUN REEL VALVES AND WSP'S)	LS-280A THRU D LS-280A THRU D LS-280A THRU D	NARROW RANGE NARROW RANGE NARROW RANGE
2	TRIP 4 OF RBP PUMPS	SEE REFERENCE DOCUMENT 12	NARROW RANGE
1	INITIATES RBC	LS-280A-1 THRU D-1	WIDE RANGE
	TRIP REMAINING 2 RBP PUMPS CLOSE CUR ISOL VALVES	LS-280A-1 THRU D-1 LS-280A-2 THRU D-2	WIDE RANGE WIDE RANGE
	INITIATES HPC/ B & C	LS-280C-3 THRU D-3	WIDE RANGE
LS	CLOSE WSP'S & ON COOLING SYSTEM ISOL VALVES	LS-280C-4 THRU D-4	WIDE RANGE
	INITIATES ADS WITH CONCURRENT HIGH DIFFERENTIAL PRESSURE	LS-2803A THRU H	WIDE RANGE
	INITIATES RBP/LWP WIDE	LS-2803A THRU H	WIDE RANGE

Figure 5.1-3 NUCLEAR BOILER SYSTEM P&ID, Sheet 9 of 11







0 4 3 2 1

SPECIFICATIONS EDITED

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TABLE 6: PPE NUMBERS FOR THE MAIN STEAM LINES

MAIN STEAM LINE	SPV TO THE OUTBOARD SPV	OUTBOARD SPV TO SERVICE INTERFACE	M220MC INTERFACE TO MAIN STEAM SYSTEM	OUTBOARD SPV TEST LINE	
				OUTBOARD SPV TO REDUCER	DOWNSTREAM OF REDUCER
A	700A-NB-023	700A-NB-024	700A-NB-298	80A-NB-298	80A-NB-299
B	700A-NB-025	700A-NB-026	700A-NB-270	80A-NB-280	80A-NB-281
C	700A-NB-027	700A-NB-028	700A-NB-271	80A-NB-282	80A-NB-283
D	700A-NB-029	700A-NB-030	700A-NB-272	80A-NB-284	80A-NB-285

TABLE 7: PPE NUMBERS FOR THE SAFETY/RELIEF VALVE SPV DISCHARGE LINES

SPV	MSL TO SPV NOTE 2	SPV DISCHARGE LINE			YACUJAN BREAKER LINES	
		SPV TO OVERBOARD FLOOR	OVERBOARD FLOOR TO REDUCER	REDUCER TO SILENCER	UPSTREAM	DOWNSTREAM
F012A	80A-NB-031	250A-NB-031	250A-NB-033	300A-NB-034	250A-NB-033	250A-NB-034
F012B	80A-NB-030	250A-NB-036	250A-NB-037	300A-NB-038	250A-NB-036	250A-NB-038
F012C	80A-NB-038	250A-NB-040	250A-NB-041	300A-NB-042	250A-NB-037	250A-NB-038
F012D	80A-NB-043	250A-NB-044	250A-NB-045	300A-NB-048	250A-NB-038	250A-NB-040
F012E	80A-NB-047	250A-NB-048	250A-NB-053	300A-NB-050	250A-NB-041	250A-NB-043
F012F	80A-NB-081	250A-NB-081	250A-NB-083	300A-NB-084	250A-NB-043	250A-NB-044
F012G	80A-NB-086	250A-NB-088	250A-NB-087	300A-NB-088	250A-NB-045	250A-NB-048
F012H	80A-NB-088	250A-NB-080	250A-NB-081	300A-NB-082	250A-NB-047	250A-NB-048
F012J	80A-NB-083	250A-NB-084	250A-NB-086	300A-NB-088	250A-NB-048	250A-NB-050
F012K	80A-NB-087	250A-NB-088	250A-NB-089	300A-NB-070	250A-NB-051	250A-NB-052
F012L	80A-NB-071	250A-NB-073	250A-NB-073	300A-NB-074	250A-NB-053	250A-NB-054
F012M	80A-NB-078	250A-NB-078	250A-NB-077	300A-NB-078	250A-NB-056	250A-NB-058
F012N	80A-NB-079	250A-NB-080	250A-NB-081	300A-NB-082	250A-NB-057	250A-NB-058
F012P	80A-NB-083	250A-NB-084	250A-NB-080	300A-NB-088	250A-NB-059	250A-NB-060
F012R	80A-NB-087	250A-NB-084	250A-NB-088	300A-NB-090	250A-NB-061	250A-NB-062
F012S	80A-NB-081	250A-NB-082	250A-NB-083	300A-NB-084	250A-NB-063	250A-NB-064
F012T	80A-NB-085	250A-NB-088	250A-NB-087	300A-NB-088	250A-NB-063	250A-NB-068
F012U	80A-NB-088	250A-NB-090	250A-NB-091	300A-NB-092	250A-NB-067	250A-NB-068

TABLE 8: PPE NUMBERS FOR THE MAIN STEAM ISOLATION VALVE SPV PNEUMATIC LINES

SPV	OPENING-CHECK VALVE TO SPV	OPENING-FROM ACCUMULATOR	CLOSING-VALVE CONTROL PANEL TO SPV	VENT LINE
F008A	80-NB-070	80-NB-071	80-NB-072	80-NB-008
F008B	80-NB-073	80-NB-074	80-NB-075	80-NB-008
F008C	80-NB-076	80-NB-077	80-NB-078	80-NB-010
F008D	80-NB-079	80-NB-080	80-NB-081	80-NB-011
F008E	80-NB-082	80-NB-083	80-NB-084	80-NB-012
F008F	80-NB-085	80-NB-086	80-NB-087	80-NB-013
F008G	80-NB-088	80-NB-089	80-NB-090	80-NB-014
F008H	80-NB-091	80-NB-092	80-NB-093	80-NB-015

TABLE 6: PIPE NUMBERS FOR THE MAIN STEAM LINE (MSL) INSTRUMENT LINES

MAIN STEAM LINE	MSL FLOW RESTRICTOR INSTRUMENT LINES				MSL PRESSURE TEST POINT
	INSTRUMENT LINE TO LDR		INSTRUMENT LINE TO LDR & FDRD		
	MSL TO REDUCER	REDUCER TO CHECK FLOW CHECK VALVE	MSL TO REDUCER	REDUCER TO CHECK FLOW CHECK VALVE	
A	25A-NB-784	25A-NB-785	25A-NB-786	25A-NB-787	25A-NB-788
B	25A-NB-789	25A-NB-790	25A-NB-770	25A-NB-771	25A-NB-781
C	25A-NB-772	25A-NB-773	25A-NB-774	25A-NB-775	-
D	25A-NB-776	25A-NB-777	25A-NB-778	25A-NB-779	-

TABLE 8: PIPE NUMBERS FOR THE SAFETY/RELIEF VALVE (SRV) PNEUMATIC LINES

SRV	ADS PNEUMATIC LINES				PNEUMATIC LINES FOR POWER-ACTUATED RELIEF		
	CHECK VALVE TO SRV ADS 1' TO SRV	BRANCH LINE FROM ACCUMULATOR	BRANCH LINE FROM SRV ADS 1' TO SRV ADS 2'	ACCUMULATOR VENT LINE	CHECK VALVE TO SRV ADS 1' ADS SRV OR SRV NON-ADS SRV	BRANCH LINE FROM ACCUMULATOR	ACCUMULATOR VENT LINE
F06A	--NB-84	--NB-86	--NB-88	--NB-89	--NB-97	--NB-98	--NB-97
F06B					--NB-99	--NB-200	--NB-98
F06C	--NB-201	--NB-202	--NB-203	--NB-89	--NB-204	--NB-205	--NB-620
F06D					--NB-206	--NB-207	--NB-621
F06E					--NB-208	--NB-209	--NB-622
F06F	--NB-210	--NB-211	--NB-212	--NB-613	--NB-213	--NB-214	--NB-624
F06G					--NB-215	--NB-216	--NB-626
F06H	--NB-217	--NB-218	--NB-219	--NB-628	--NB-220	--NB-221	--NB-627
F06I					--NB-222	--NB-223	--NB-629
F06J					--NB-224	--NB-225	--NB-628
F06K	--NB-226	--NB-227	--NB-228	--NB-630	--NB-229	--NB-230	--NB-631
F06L					--NB-231	--NB-232	--NB-632
F06M	--NB-233	--NB-234	--NB-235	--NB-633	--NB-236	--NB-237	--NB-634
F06P					--NB-238	--NB-239	--NB-636
F06R	--NB-240	--NB-241	--NB-242	--NB-638	--NB-243	--NB-244	--NB-637
F06S					--NB-245	--NB-246	--NB-638
F06T	--NB-247	--NB-248	--NB-249	--NB-639	--NB-250	--NB-251	--NB-640
F06U					--NB-252	--NB-253	--NB-641

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Figure 5.1-3 NUCLEAR BOILER SYSTEM P&ID, Sheet 11 of 11

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NOTES:

1. ALL SIGNALS ARE TRANSMITTED AS SERIAL DATA TO EACH TLU OR SLL THROUGH EACH SERIAL OUTPUT.
2. FOUR TEMPERATURE ELEMENTS SHALL BE LOCATED AT APPROXIMATELY EQUAL INTERVALS IN THE VERTICAL DIRECTION SO AS TO MONITOR AMBIENT TEMPERATURES OVER THE FULL HEIGHT OF THE DRYWELL.
3. TEMPERATURE DETECTORS SHALL BE LOCATED OR SWELED SO THAT THE DETECTOR IS SENSITIVE TO THE AIR TEMPERATURE AND NOT THE RADIATED HEAT FROM HOT EQUIPMENT.
4. ALL INSTRUMENT LINES THAT CONNECT TO THE REACTOR COOLANT PRESSURE BOUNDARY AND PENETRATE THE CONTAINMENT WALL SHALL HAVE 6.30 MM RESTRICTING DEVICES INSIDE THE CONTAINMENT. SEE SUPPORTING DOCUMENTS 1&2 FOR ADDITIONAL REQUIREMENTS.
5. EACH INSTRUMENT LINE THROUGH THE CONTAINMENT WALL SHALL HAVE TWO ISOLATION VALVES OUTSIDE THE CONTAINMENT LOCATED AS CLOSE TO THE CONTAINMENT AS PRACTICAL.
6. SOLENOID OPERATED GLOBE VALVES MAY BE ADOPTED FOR THE AIR OPERATED VALVES.
7. THIS DOCUMENT PROVIDES A FUNCTIONAL DEFINITION OF THE REQUIRED SYSTEM LEVEL, PROCESS MONITORING AND CONTROL INSTRUMENTATION. IT DOES NOT DESCRIBE DETAILS OF THE METHODS BY WHICH SIGNALS FROM THESE COMPONENTS WILL BE PROCESSED. THIS PROCESSING MAY INVOLVE THE PLANT MULTIPLEXING SYSTEM 9122 OR MAY UTILIZE HARDWIRED SPECIFIC ELECTRICAL ISOLATION REQUIREMENTS SYSTEM ON THE SIGNALING WAY. UNNECESSARY MULTIPLEXED SIGNAL TRANSMISSION PROVIDES IMPERENT ISOLATION.
8. ALL ALARMS SHALL BE LOCATED IN THE MAIN CONTROL ROOM.
9. TEMPERATURE SWITCHES (TS) SHALL BE PROVIDED WITH TEMPERATURE INDICATOR EITHER ON THE SWITCHES OR ON CONDUIT METERS MODULES.
10. THE LOS SHALL BE DESIGNED IN ACCORDANCE WITH THE SYSTEM DESIGN SPECIFICATION 2-31-4300. OTHER PRIMARY CONTAINMENT ISOLATION (PCI) VALVES WHICH ARE PART OF OTHER SYSTEMS, ARE NOT SHOWN IN THIS 2-31-4300. THESE VALVES ARE SHOWN ON THE LOS/MSD - INTERLOCK BLOCK DIAGRAM (2-31-1630).
11. FOR REACTOR WATER LEVELS MONITORING, SEE 2-31-1010.
12. FOR DRYWELL PRESSURE MONITORING, SEE 2-31-1010.
13. FOR DETECTION OF RADIATION LEAKAGE INTO COOLING WATER SUPPLYING MICRO PUMP, RHR, AND CWR HEAT EXCHANGERS, SEE DTI-1040.
14. LETTER DESIGNATIONS FOR FOUR DIVISIONS ARE AS FOLLOWS:
  - A, E, J, N - DIVISION 1
  - B, F, K, P - DIVISION 2
  - C, G, L, R - DIVISION 3
  - D, H, M, Q - DIVISION 4
15. IF HEAT TRACING OF SAMPLE LINE IS NECESSARY TO PREVENT CONDENSATION, THE MAXIMUM ALLOWABLE SAMPLE TEMPERATURE IS LIMITED BY THE PHOTO MULTIPLIER TUBES IN THE MONITORING CHANNELS.
16. BALL VALVE MAY BE ADOPTED FOR THIS GATE VALVE.
17. DTN, TLU, SLL, OLU ARE PART OF SAFETY SYSTEM LOGIC AND CONTROL. (SEE REFERENCE DOCUMENT 12)
  - A. DTN SHOWN ON SHEETS 2 & 4 PROCESS SENSOR INPUTS FOR WSW ISOLATION TRIP LOGIC.
  - B. DTN SHOWN ON SHEETS 3 & 8 PROCESS SENSOR INPUTS FOR ECCS ISOLATION TRIP LOGIC.
  - C. DTN SHOWN ON SHEETS 7 & 10 PROCESS SENSOR INPUTS FOR AUXILIARY 2# ISOLATION TRIP LOGIC.
  - D. TLU AND OLU SHOWN ON SHEET 3 AND SHEET 4 PROCESS 2-OUT-OF-4 CONSIDENCE LOGIC FOR WSW CLOSURE TRIP.
  - E. SLL SHOWN ON SHEETS 3 & 8 PROCESS 2-OUT-OF-4 CONSIDENCE LOGIC FOR ECCS ISOLATION.
  - F. SLL SHOWN ON SHEETS 7 & 10 PROCESS 2-OUT-OF-4 CONSIDENCE LOGIC FOR AUXILIARY 2# ISOLATION.
18. SWS, WUX, EWS AND NEMS ARE PART OF THE MULTIPLEXING SYSTEM 9122.
19. ALL INSTRUMENT LINES ARE 304-SS (STAINLESS STEEL) SCHEDULE 40. SAMPLING LINES ARE 316-SS SCHEDULE 40.

8 7 6 5 4 3 2 1

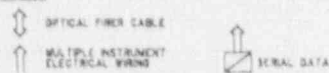
REFERENCES

	WP. NO.
1. REACTOR CORE ISOLATION COOLING SYSTEM P&ID	181-1010
2. NUCLEAR SOLDER SYSTEM P&ID	82-1010
3. LIQUID WASTE SYSTEM P&ID	417-1010
4. SAMPLING SYSTEM P&ID	781-1010
5. PROCESS RADIATION MONITORING SYSTEM SD	071-1040
6. VALVE BLAND LEAKAGE TREATMENT SYSTEM	771-1010
7. INSTRUMENT AIR SYSTEM P&ID	783-1010
8. ESSENTIAL MULTIPLEXING SYSTEM	163-4015
9. LEAK DETECTION & ISOLATION SYSTEM ID	131-1030
10. REACTOR PROTECTION SYSTEM ID	171-1040
11. STANDBY LIQUID CONTROL SYSTEM ID	141-1030
12. SAFETY SYSTEM & LOGIC CONTROL DS	123-8280
13. PERFORMANCE MONITORING & CONTROL SYSTEM DS	151-4010
14. NEUTRON MONITORING SYSTEM ID	131-1010
15. SUPPRESSION POOL CLEAN-UP SYSTEM P&ID	141-1010
16. REACTOR BUILDUP WATER CLEAN-UP SYSTEM P&ID	131-1010
17. HYDRO NORMAL COOLING WATER SYSTEM P&ID	131-1010
18. STANDBY GAS TREATMENT SYSTEM P&ID	132-1010
19. ATMOSPHERIC CONTROL SYSTEM P&ID	131-1010
20. FLAMMABILITY CONTROL SYSTEM P&ID	148-1010
21. HEATING, VENTILATING & AIR CONDITIONING P&ID	141-1010
22. REACTOR WATER CLEANUP SYSTEM P&ID	131-1010
23. RADIOACTIVE DRAIN TRANSFER SYSTEM P&ID	171-1010

SUPPORTING DOCUMENTS

	WP. NO.
1. PIPING AND INSTRUMENT DIAGRAM SYMBOLS	410-3010
2. PROCESS INSTRUMENTATION	410-3030
3. GROUP CLASSIFICATION & CONTAINMENT ISOLATION D&D	410-1030

LEGEND



ABBREVIATIONS

- OLU - OUTPUT LOGIC UNIT
- TLU - TRIP LOGIC UNIT
- DTM - DIGITAL TRIP MODULE
- RMU - REMOTE MULTIPLEXING UNIT
- BLU - SAFETY SYSTEM LOGIC UNIT
- ESMS - ESSENTIAL MULTIPLEXING SYSTEM
- HEMS - NON-ESSENTIAL MULTIPLEXING SYSTEM

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WP. NO. 131-1040

Figure 5.2-8 LEAK DETECTION AND ISOLATION SYSTEM IED, Sheet 1 of 10



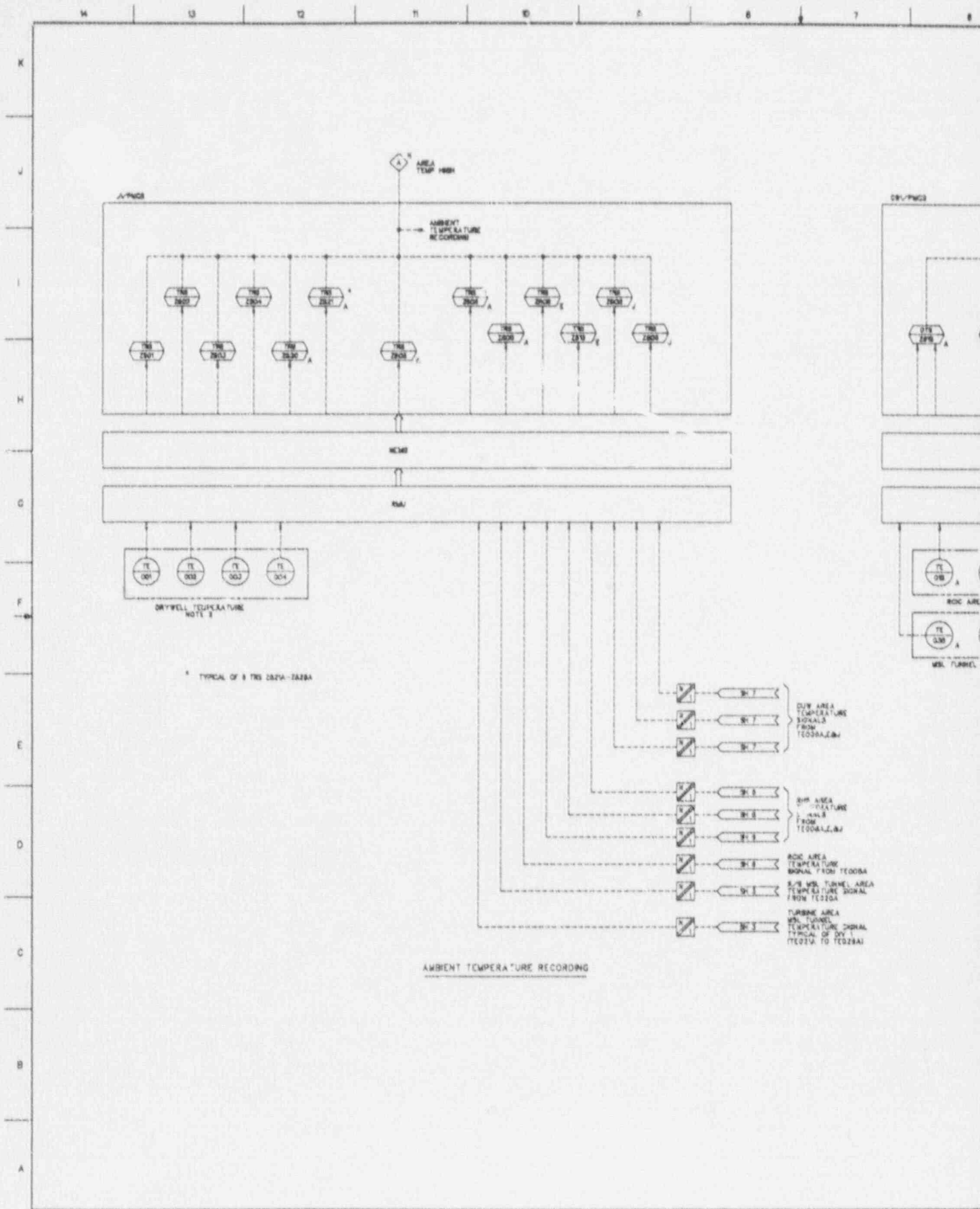
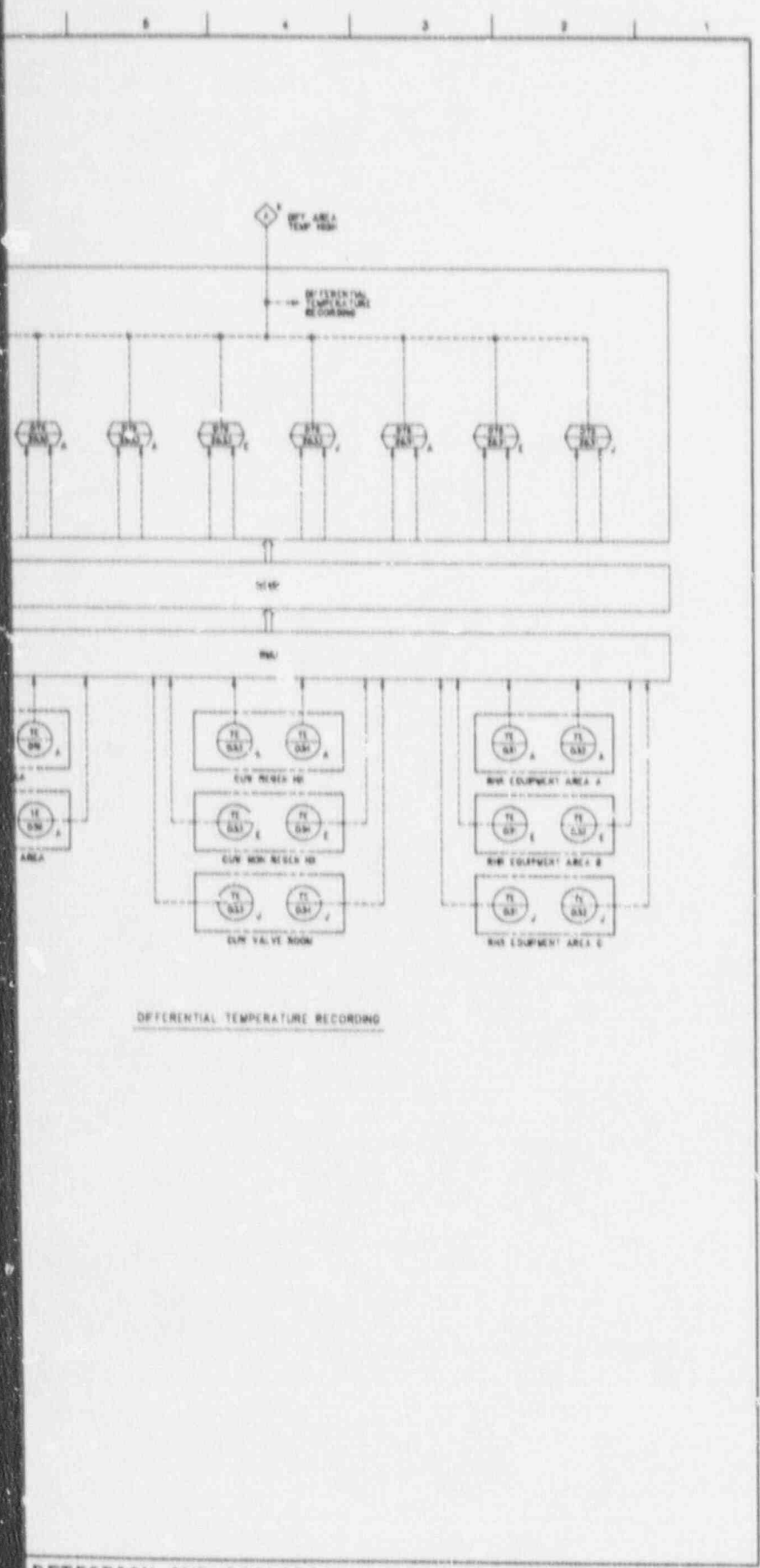


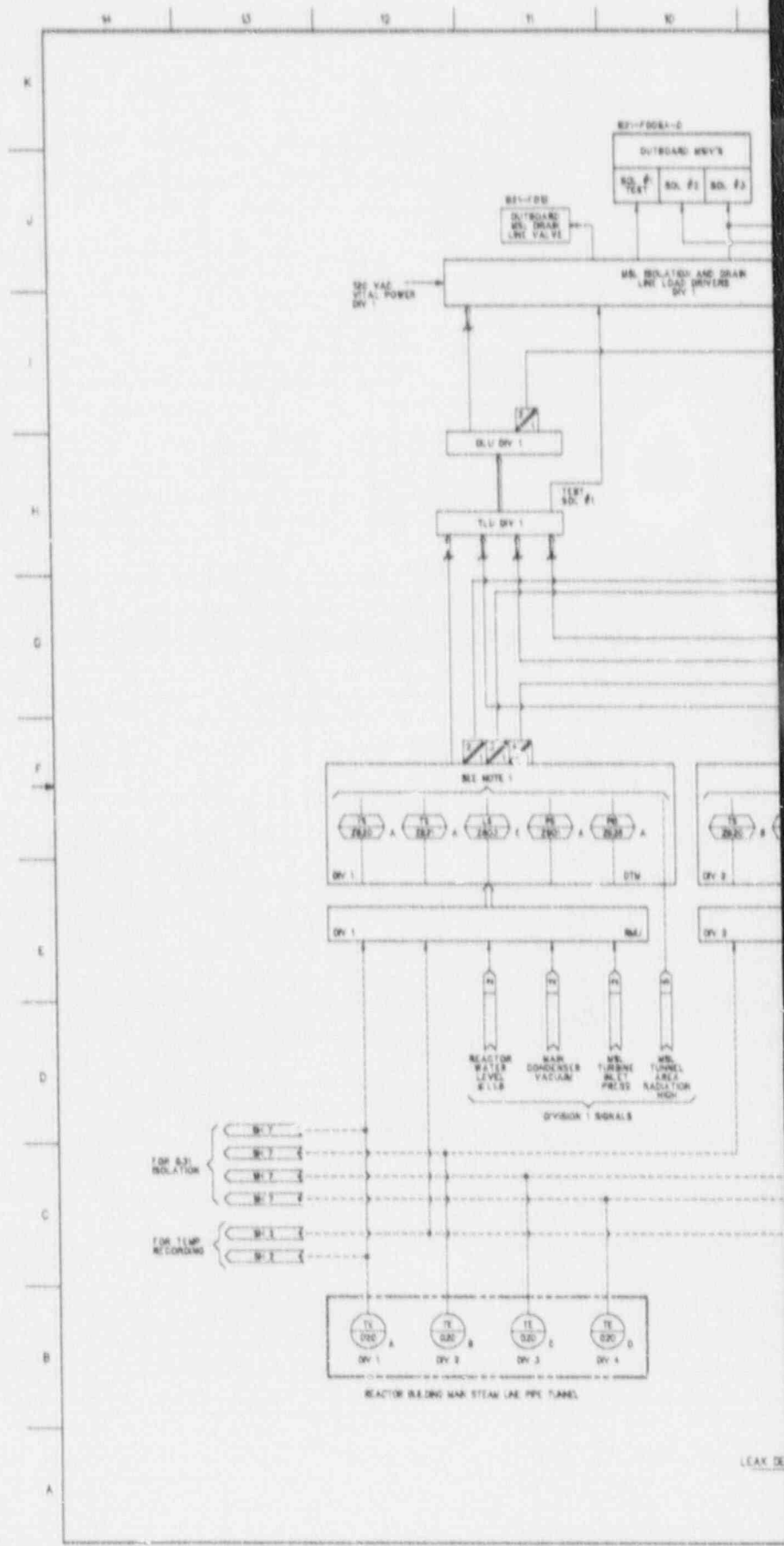
Figure 5.2-8 LEAK





DIFFERENTIAL TEMPERATURE RECORDING

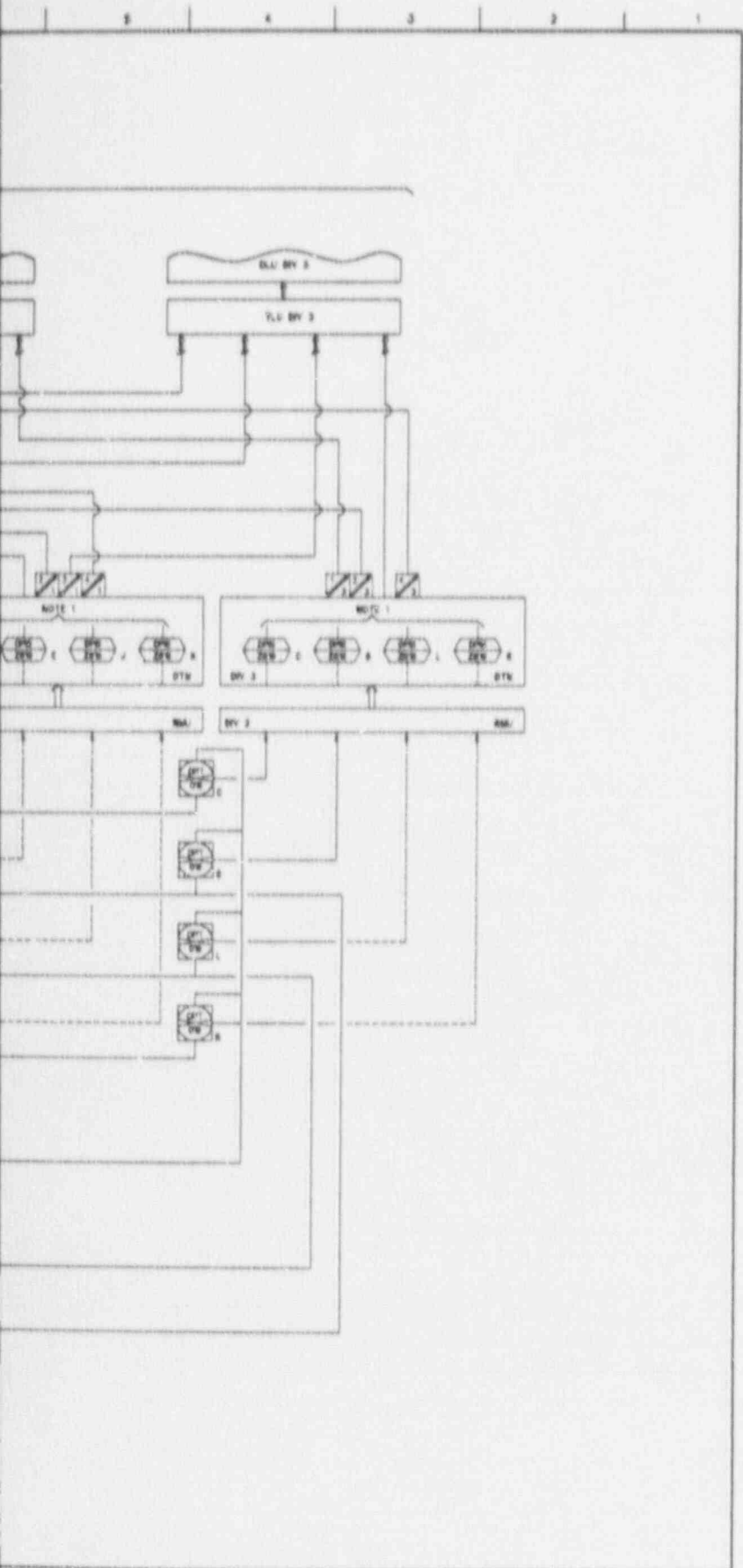
APPROVED  
DATE  
BY



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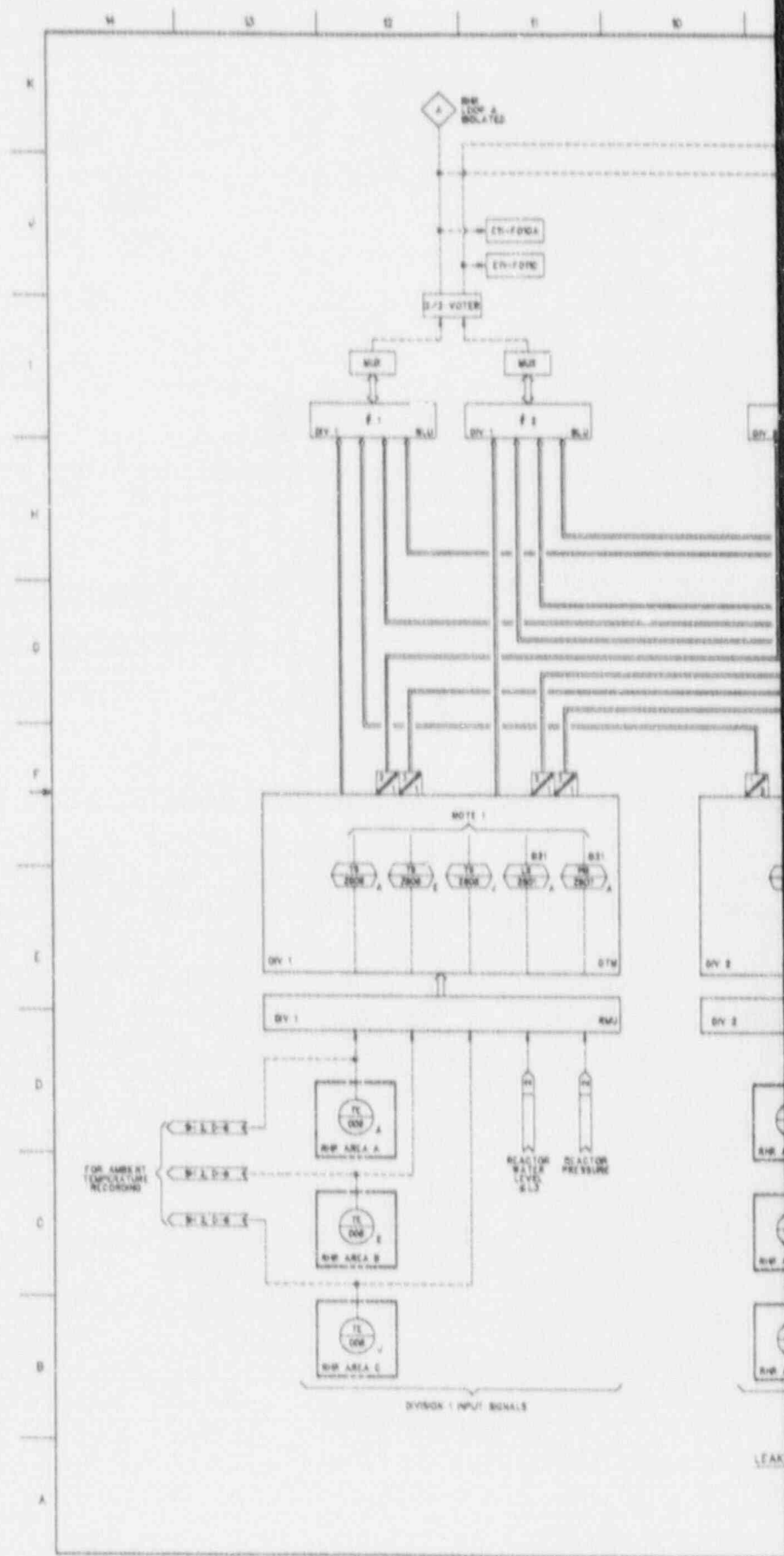




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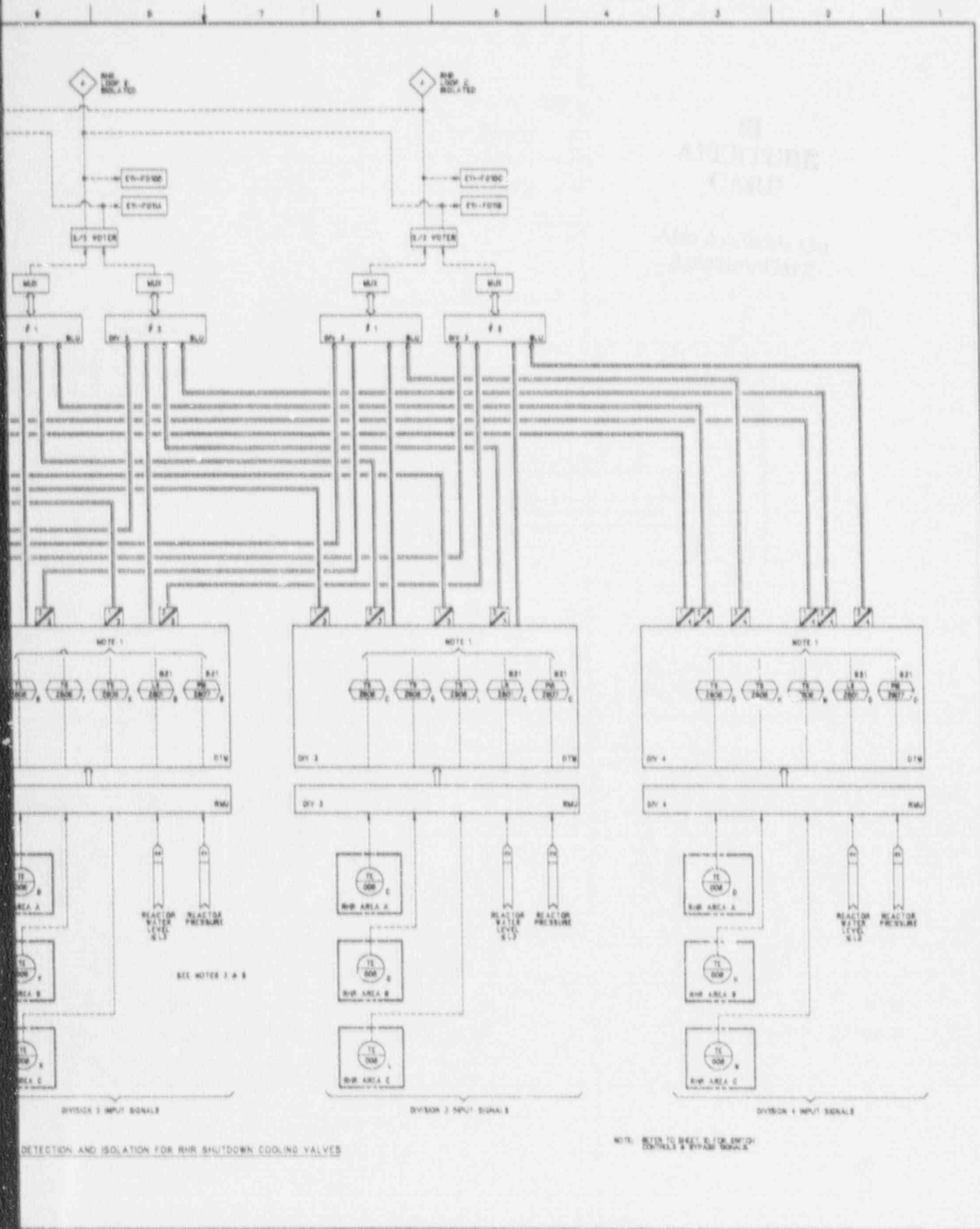
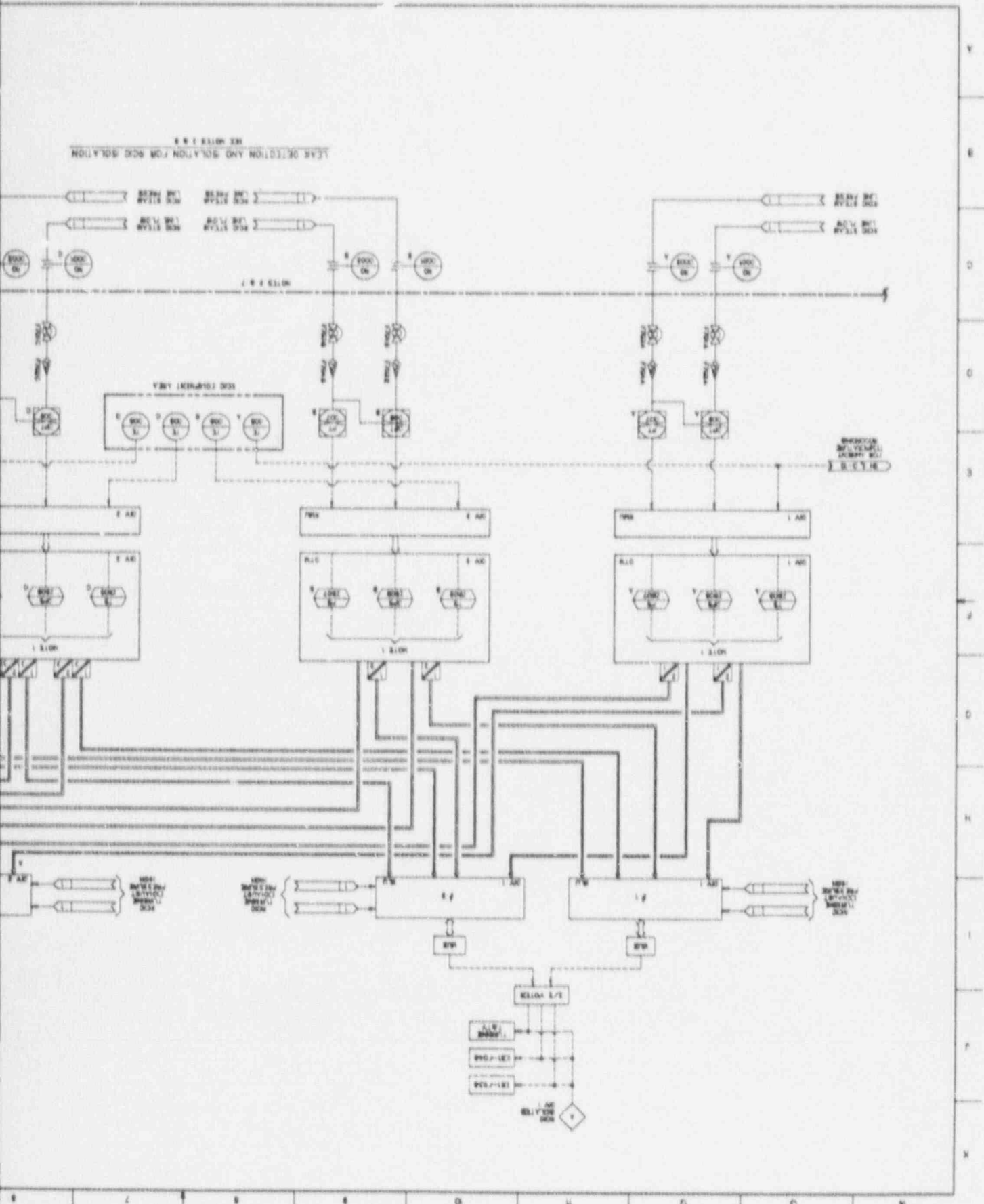
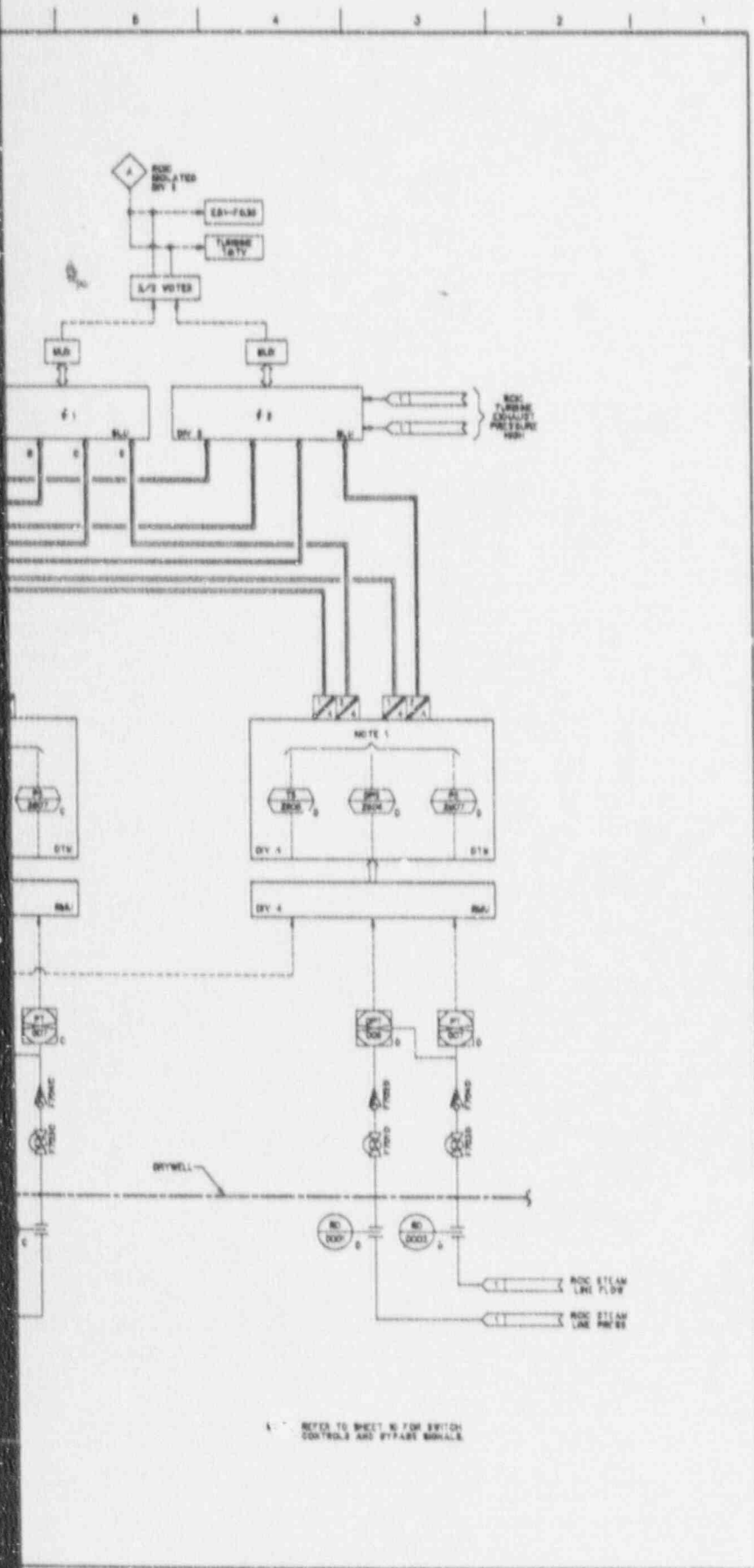


Figure 5.2-8 LEAK DETECTION AND ISOLATION SYSTEM IED, Sheet 5 of 10

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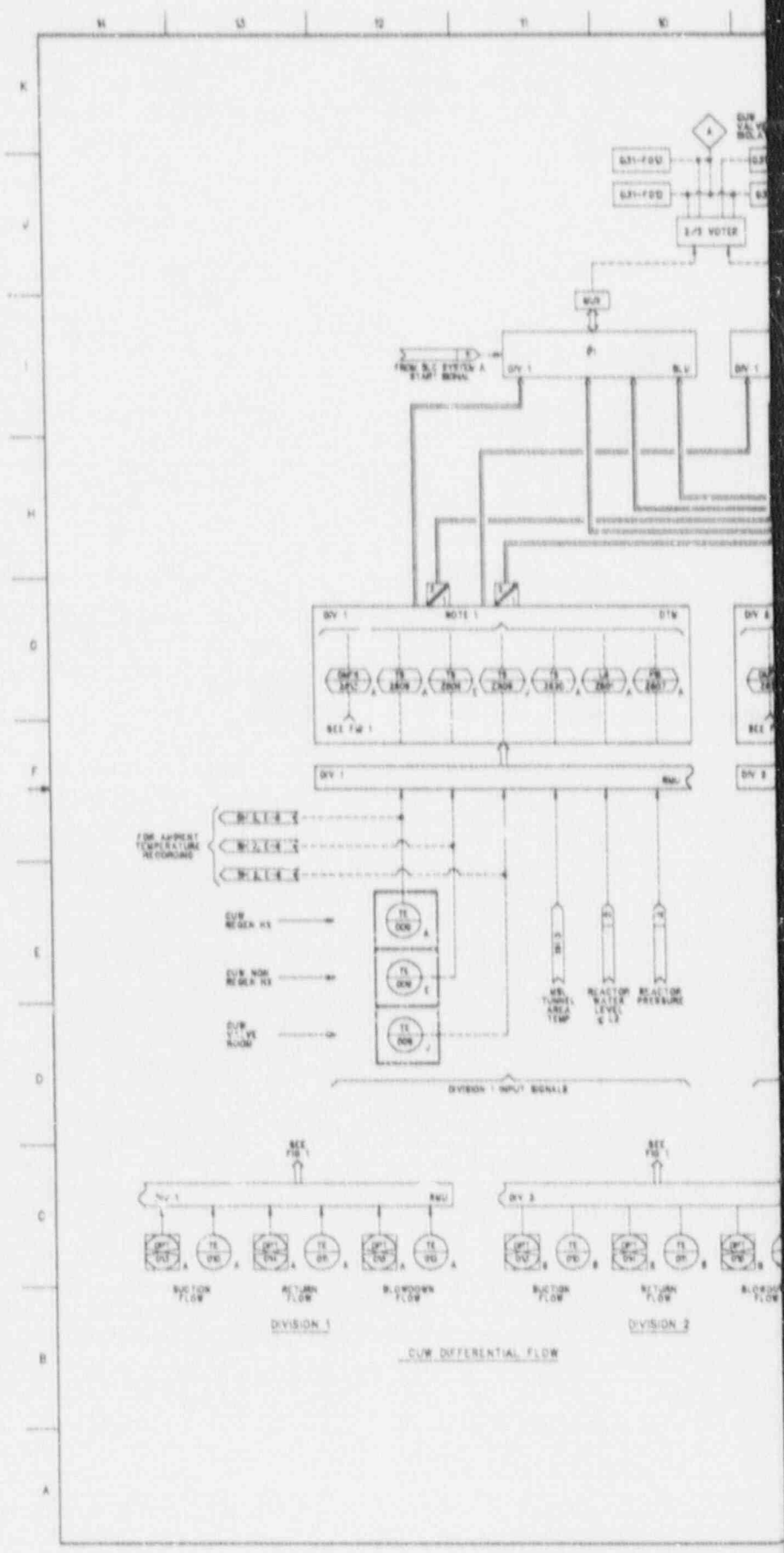
Figure 5.2-8 LEAK



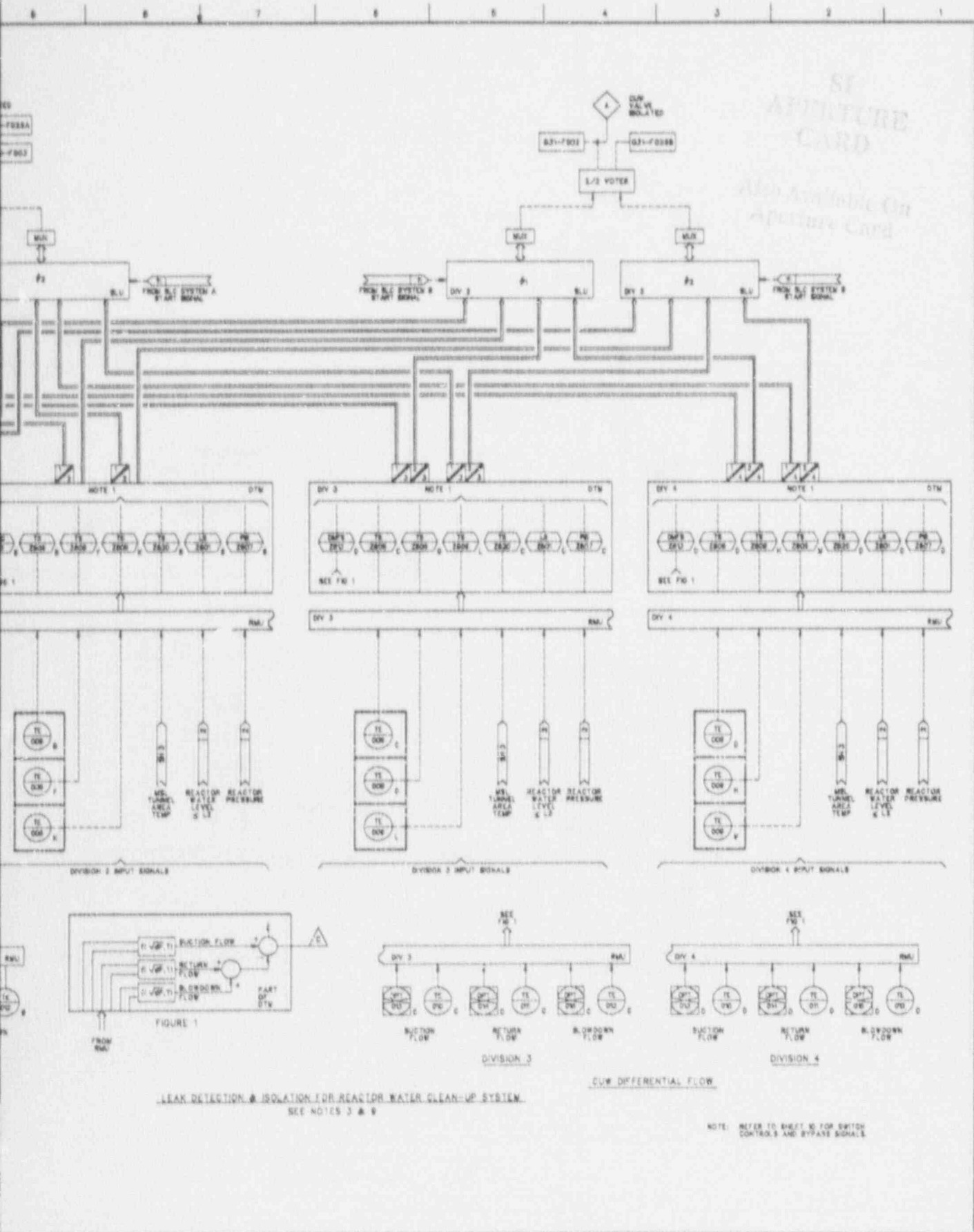


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LEAK DETECTION & ISOLATION FOR REACTOR WATER CLEAN-UP SYSTEM  
SEE NOTES 3 & 9

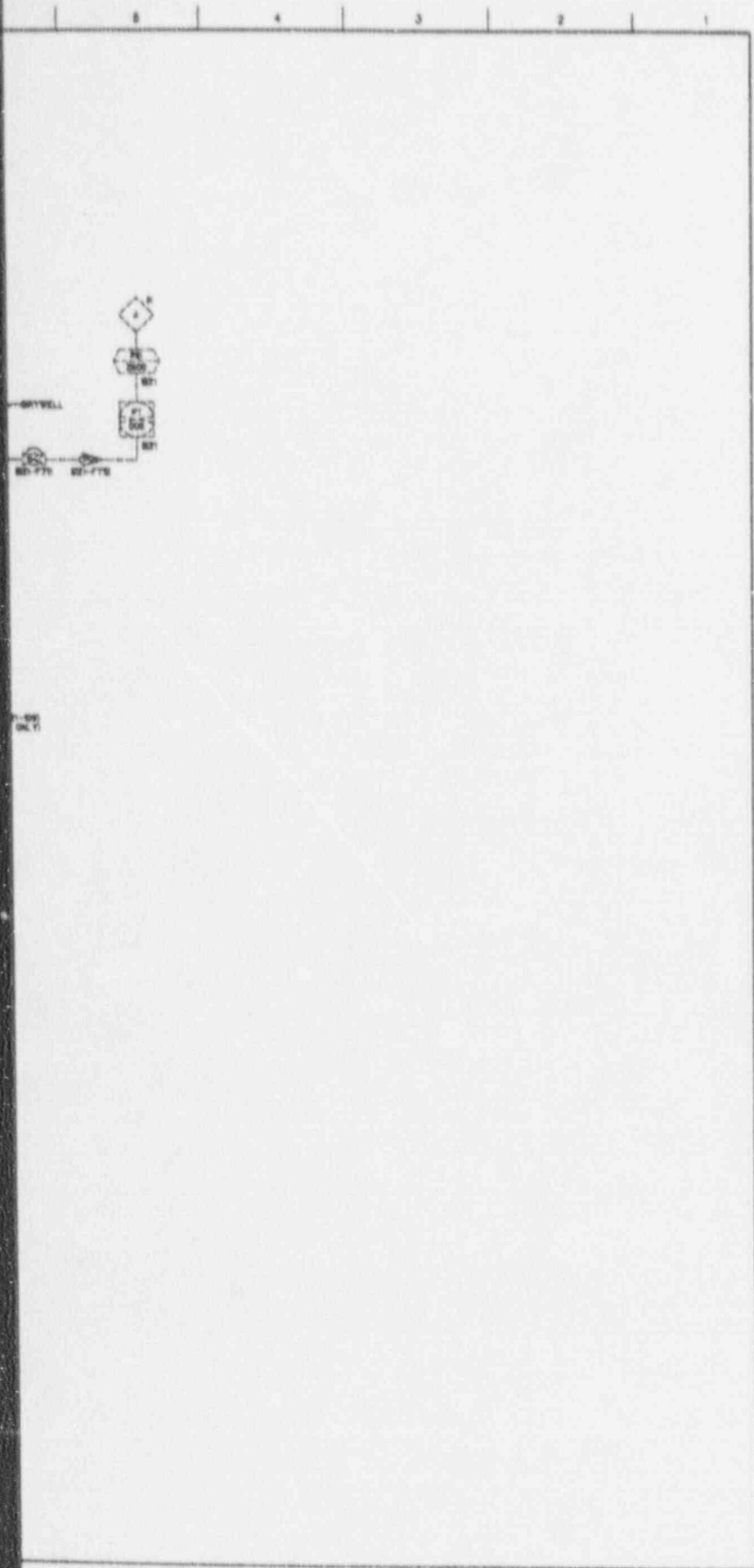
CUV DIFFERENTIAL FLOW

NOTE: REFER TO SHEET 10 FOR SWITCH CONTROLS AND BYPASS SIGNALS

Figure 5.2-8 LEAK DETECTION AND ISOLATION SYSTEM IED, Sheet 7 of 10

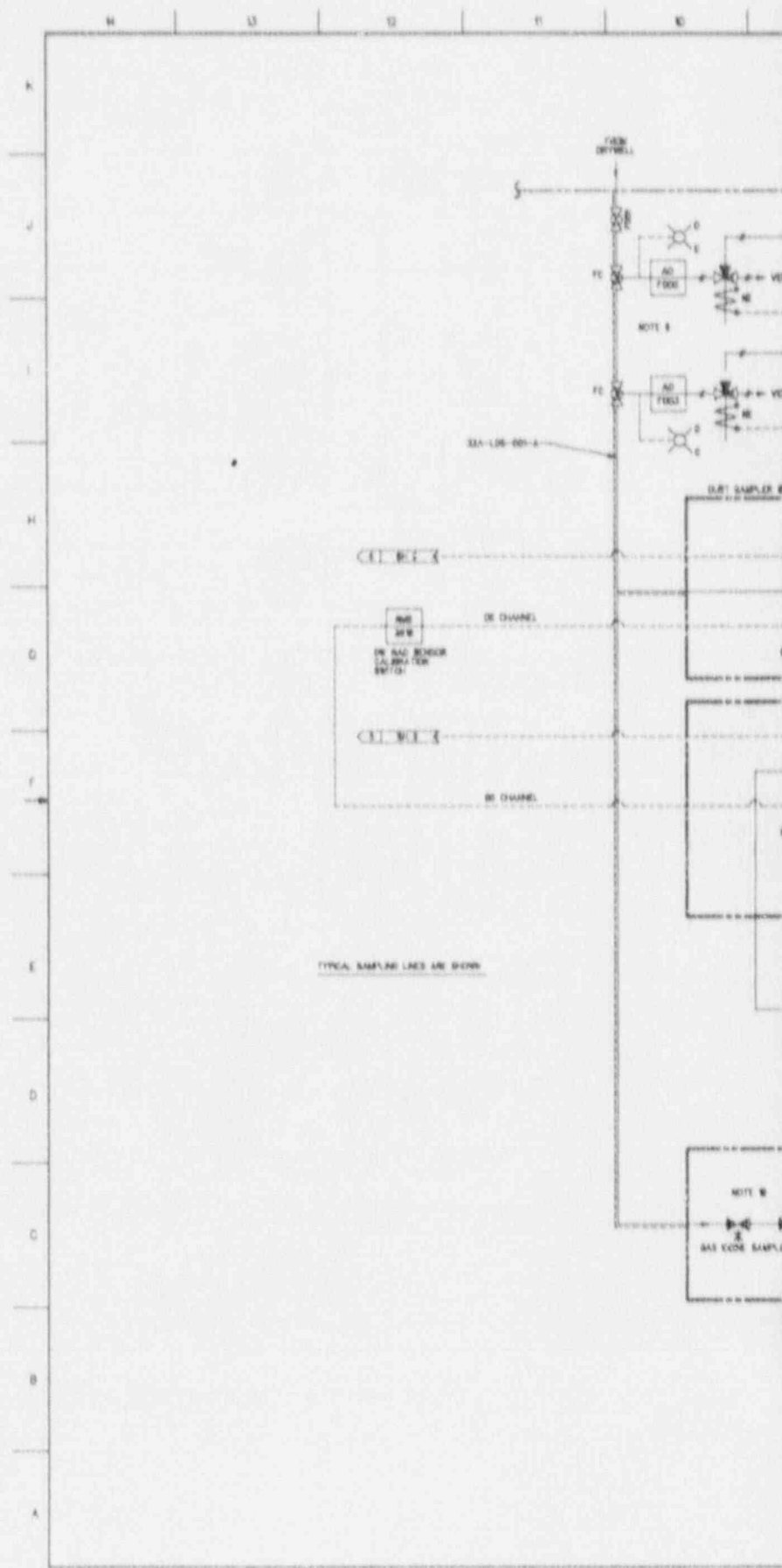
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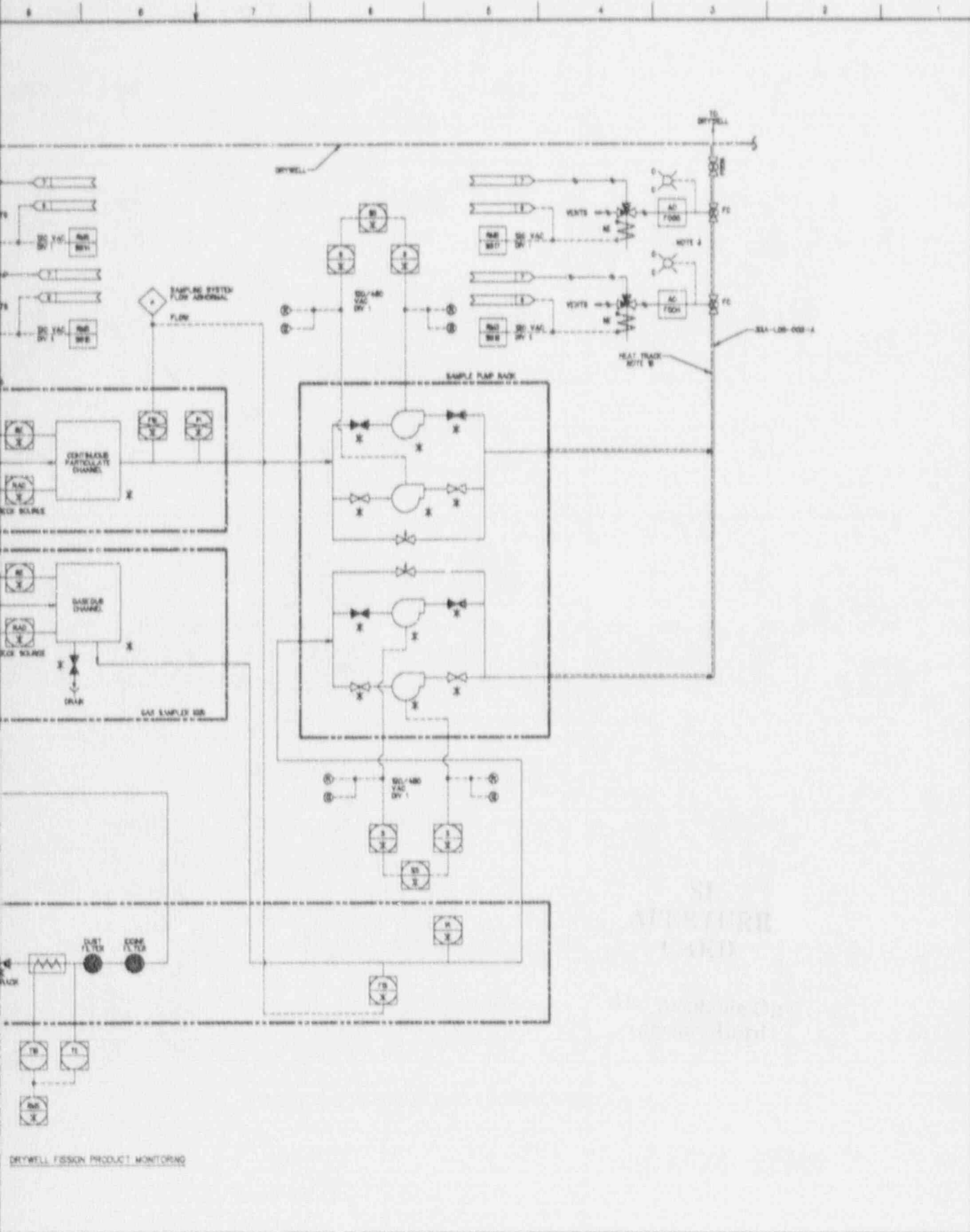


Figure 5.2-8 LEAK DETECTION AND ISOLATION SYSTEM IED, Sheet 9 of 10



# ABWR Standard Plant

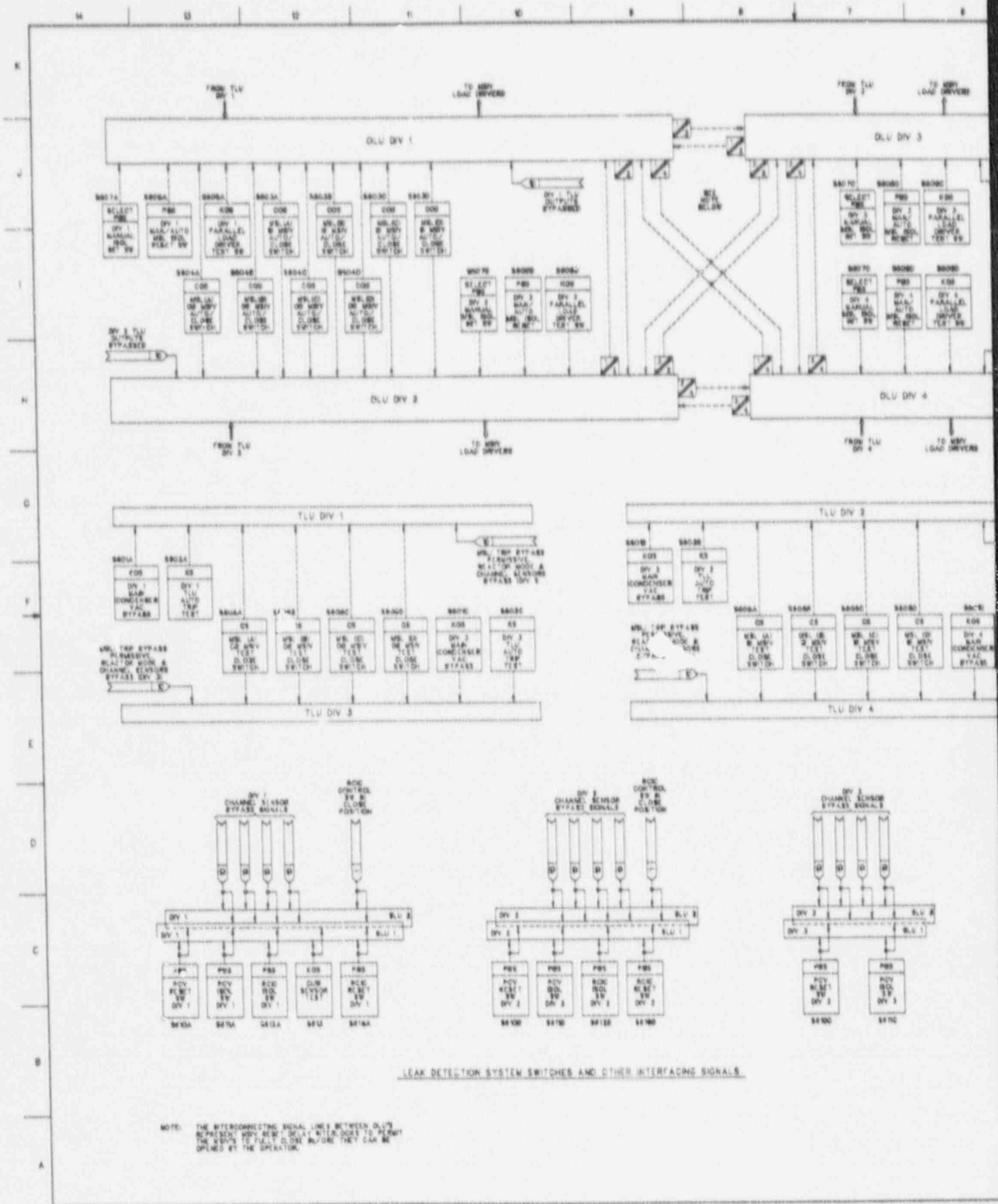
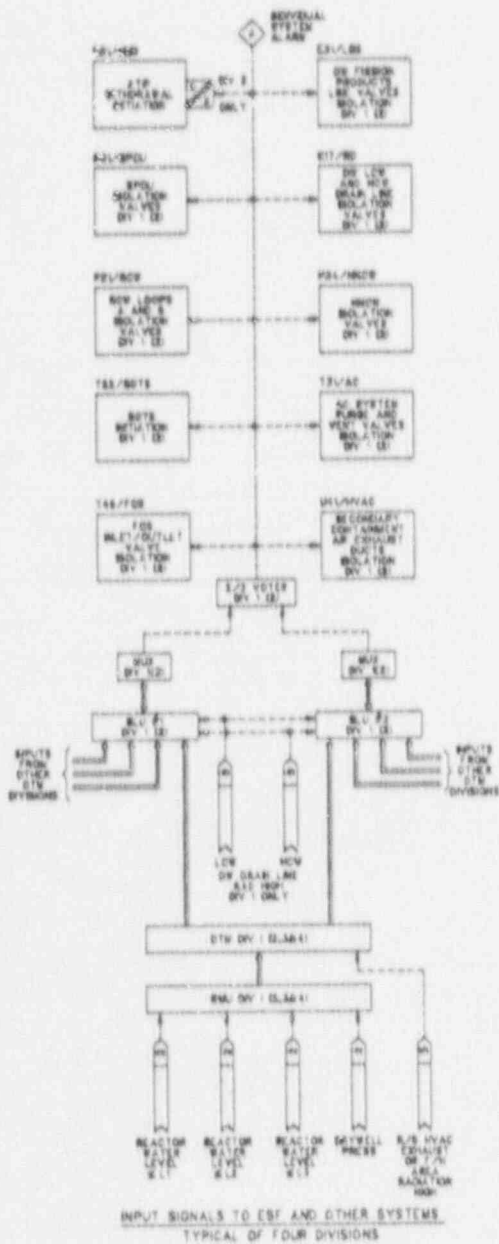
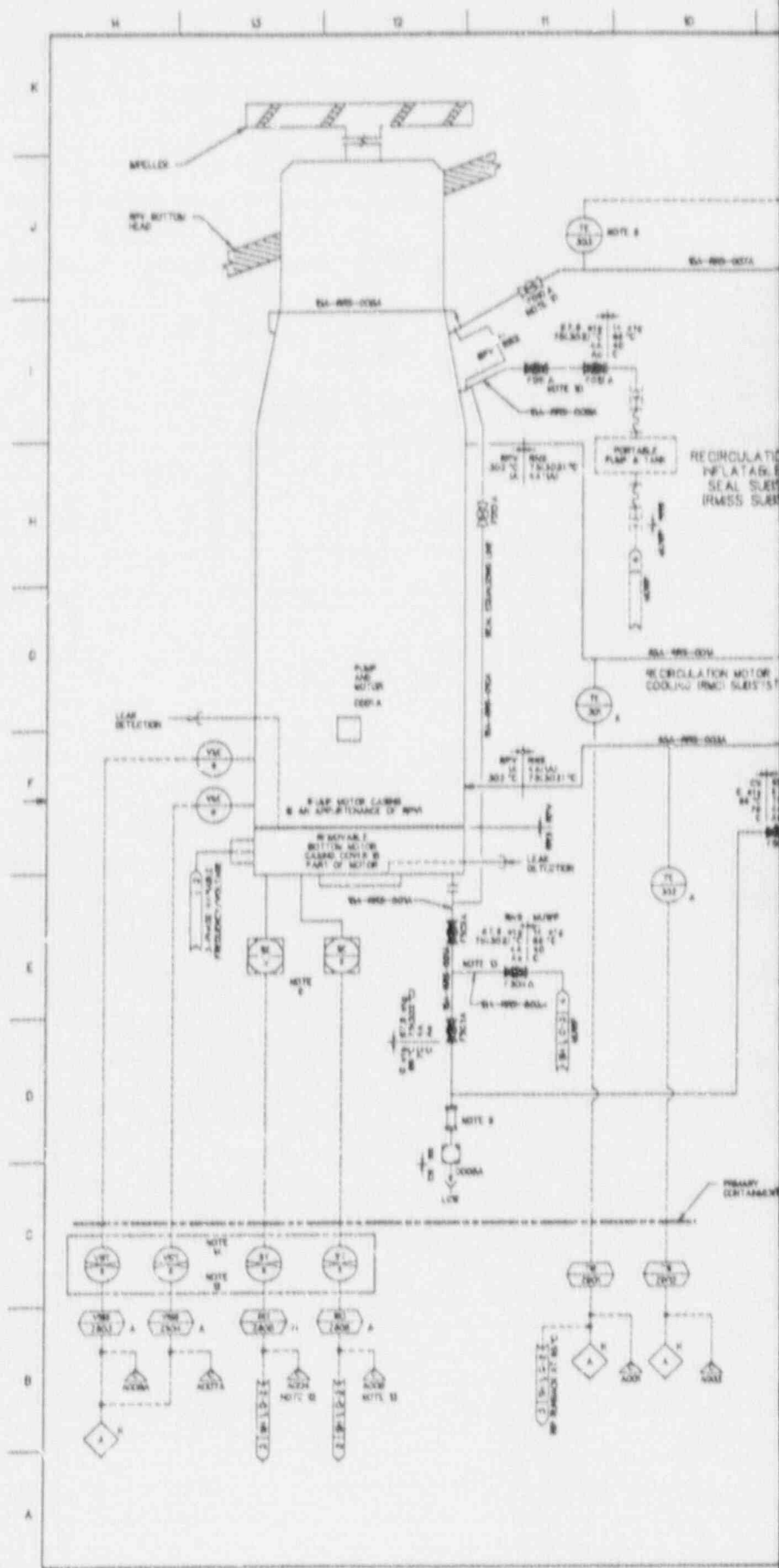


Figure 5.2-8 LEAK



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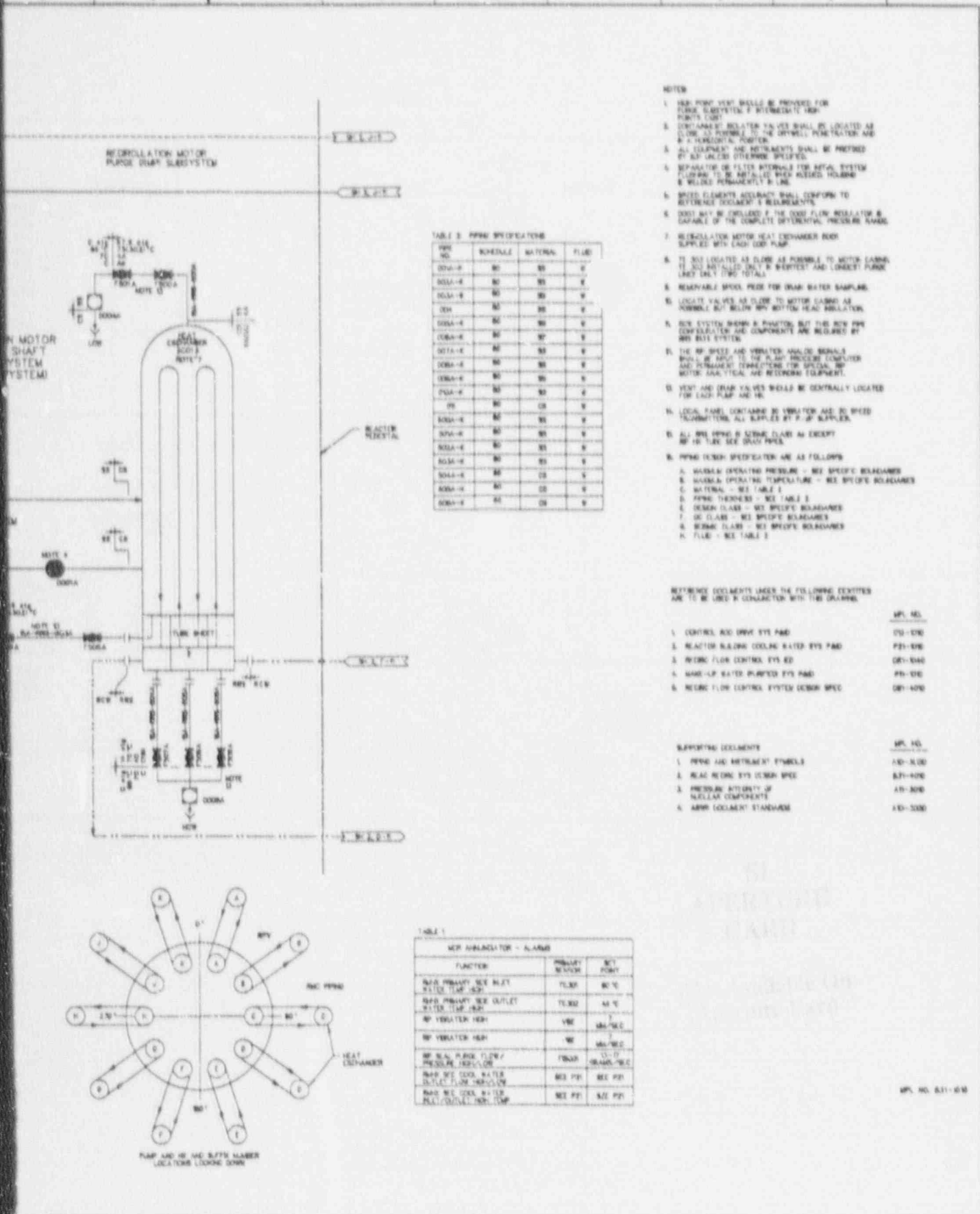


TABLE 2: PFRM SPECIFICATIONS

PFRM NO.	SCHEDULE	MATERIAL	FLUID
001A-F	80	80	0
002A-F	80	80	0
003A-F	80	80	0
004A-F	80	80	0
005A-F	80	80	0
006A-F	80	80	0
007A-F	80	80	0
008A-F	80	80	0
009A-F	80	80	0
010A-F	80	80	0
011A-F	80	80	0
012A-F	80	80	0
013A-F	80	80	0
014A-F	80	80	0
015A-F	80	80	0
016A-F	80	80	0
017A-F	80	80	0
018A-F	80	80	0
019A-F	80	80	0
020A-F	80	80	0
021A-F	80	80	0
022A-F	80	80	0
023A-F	80	80	0
024A-F	80	80	0
025A-F	80	80	0
026A-F	80	80	0
027A-F	80	80	0
028A-F	80	80	0
029A-F	80	80	0
030A-F	80	80	0
031A-F	80	80	0
032A-F	80	80	0
033A-F	80	80	0
034A-F	80	80	0
035A-F	80	80	0
036A-F	80	80	0
037A-F	80	80	0
038A-F	80	80	0
039A-F	80	80	0
040A-F	80	80	0

- NOTES
1. VENT PUMP VENT SHOULD BE PROVIDED FOR FLUID SUBSYSTEM 2 INTERMEDIATE VENT POINTS ONLY.
  2. CONTAINMENT ISOLATION VALVES SHALL BE LOCATED AS CLOSE AS POSSIBLE TO THE DRYWELL PENETRATOR AND # 2 HORIZONTAL POSITION.
  3. ALL EQUIPMENT AND INSTRUMENTS SHALL BE PROVIDED BY A3 UNLESS OTHERWISE SPECIFIED.
  4. SEPARATOR OR FILTER INTERNALS FOR REACTOR SYSTEM FLOWMAY TO BE INSTALLED WHEN NEEDED. HOLDING & WELDING PERMANENTLY IN LINE.
  5. SPEED ELEMENTS ACCURACY SHALL CONFORM TO REFERENCED DOCUMENT & REQUIREMENTS.
  6. DONT MAY BE CHALLENGED IF THE DONT FLOW INDICATOR IS CAPABLE OF THE COMPLETE DIFFERENTIAL PRESSURE RANGE.
  7. RECIRCULATION MOTOR HEAT EXCHANGER BEING SUPPLIED WITH CASH COOL PUMP.
  8. TO BE LOCATED AS CLOSE AS POSSIBLE TO MOTOR CABINETS. TO BE INSTALLED ONLY IN WERTEST AND LOWEST FLOOR LAYER ONLY (TWO TOTAL).
  9. REMOVABLE SPEED PUMP FOR DRINK WATER SAMPLING.
  10. LOCATE VALVES AS CLOSE TO MOTOR CABINETS AS POSSIBLE BUT WITHIN ANY BOTTOM HEAD ISOLATION.
  11. REACTOR SYSTEM SHOWN AS PHANTOM BUT THIS REACTOR PIPE CONNECTIONS AND COMPONENTS ARE REQUIRED BY THE REACTOR SYSTEM.
  12. THE REACTOR SPEED AND VIBRATION ANALOG SIGNALS SHALL BE SUPPLIED TO THE PLANT PROCESS COMPUTER AND PERMANENT TEMPLATIONS FOR SPECIAL REACTOR MOTOR ANALYTICAL AND RECORDING EQUIPMENT.
  13. VENT AND DRAIN VALVES SHOULD BE CENTRALLY LOCATED FOR EACH FLOOR AND VENT.
  14. LOCAL PANEL CONTAINS 30 SWITCHES AND 20 SPEED TRANSDUCERS. ALL SUPPLIED BY P. 2P SUPPLIES.
  15. ALL PFRM PIPING IS DESIGN CLASS A EXCEPT REACTOR TUBE SHEET SEE DRUM PIPING.
  16. PFRM DESIGN SPECIFICATIONS ARE AS FOLLOWS:
    - A. WALKER OPERATING PRESSURE - SEE SPECIFIC BOLTS
    - B. WALKER OPERATING TEMPERATURE - SEE SPECIFIC BOLTS
    - C. MATERIAL - SEE TABLE 2
    - D. PFRM THICKNESS - SEE TABLE 2
    - E. DESIGN CLASS - SEE SPECIFIC BOLTS
    - F. DR CLASS - SEE SPECIFIC BOLTS
    - G. WERMA CLASS - SEE SPECIFIC BOLTS
    - H. FLUID - SEE TABLE 2

- REFERENCE DOCUMENTS UNDER THE FOLLOWING CATEGORIES ARE TO BE USED IN CONNECTION WITH THIS DRAWING.
- | REF. NO. | DESCRIPTION                             |
|----------|---|
| 1        | CONTROL AND DRIVE SYS P&ID              |
| 2        | REACTOR WAZING COOLING WATER SYS P&ID   |
| 3        | REACTOR FLOW CONTROL SYS P&ID           |
| 4        | WAZING WATER PLANTED SYS P&ID           |
| 5        | REACTOR FLOW CONTROL SYSTEM DESIGN SPEC |

- SUPPORTING DOCUMENTS
- | REF. NO. | DESCRIPTION                            |
|----------|--|
| 1        | PFRM AND INSTRUMENT PIPING             |
| 2        | REACTOR REACTOR SYS DESIGN SPEC        |
| 3        | REACTOR INTEGRITY OF SELLER COMPONENTS |
| 4        | ABWR ISOLATION STANDARDS               |

TABLE 1: REACTOR ALARMS

FUNCTION	INITIAL	SET POINT
REACTOR PRIMARY INLET WATER TEMP. HIGH	TL30	80 °C
REACTOR PRIMARY INLET WATER TEMP. HIGH	TL30	80 °C
REACTOR HEAT EXCHANGER	VBE	184 °C
REACTOR HEAT EXCHANGER	WE	184 °C
REACTOR PRIMARY FLOW PRESSURE REDUCTION	TR30	13.0 GRADES/SEC
REACTOR COOL WATER INLET FLOW REDUCTION	WIS PFI	SEE PFI
REACTOR COOL WATER INLET FLOW REDUCTION	WIS PFI	SEE PFI

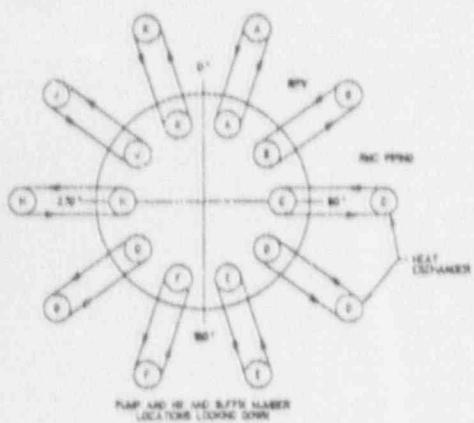


Figure 5.4-4 REACTOR RECIRCULATION SYSTEM P&ID, Sheet 1 of 2

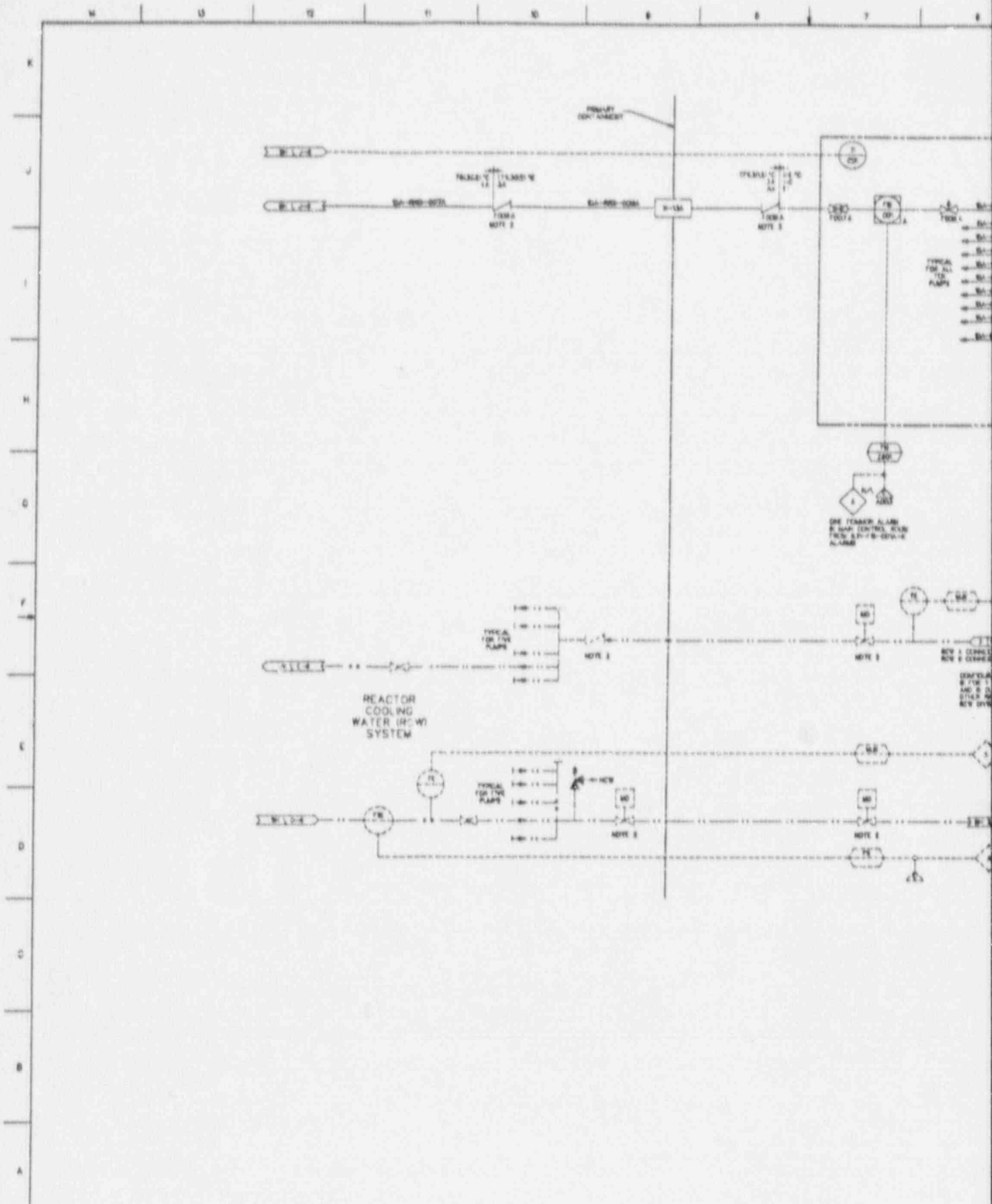
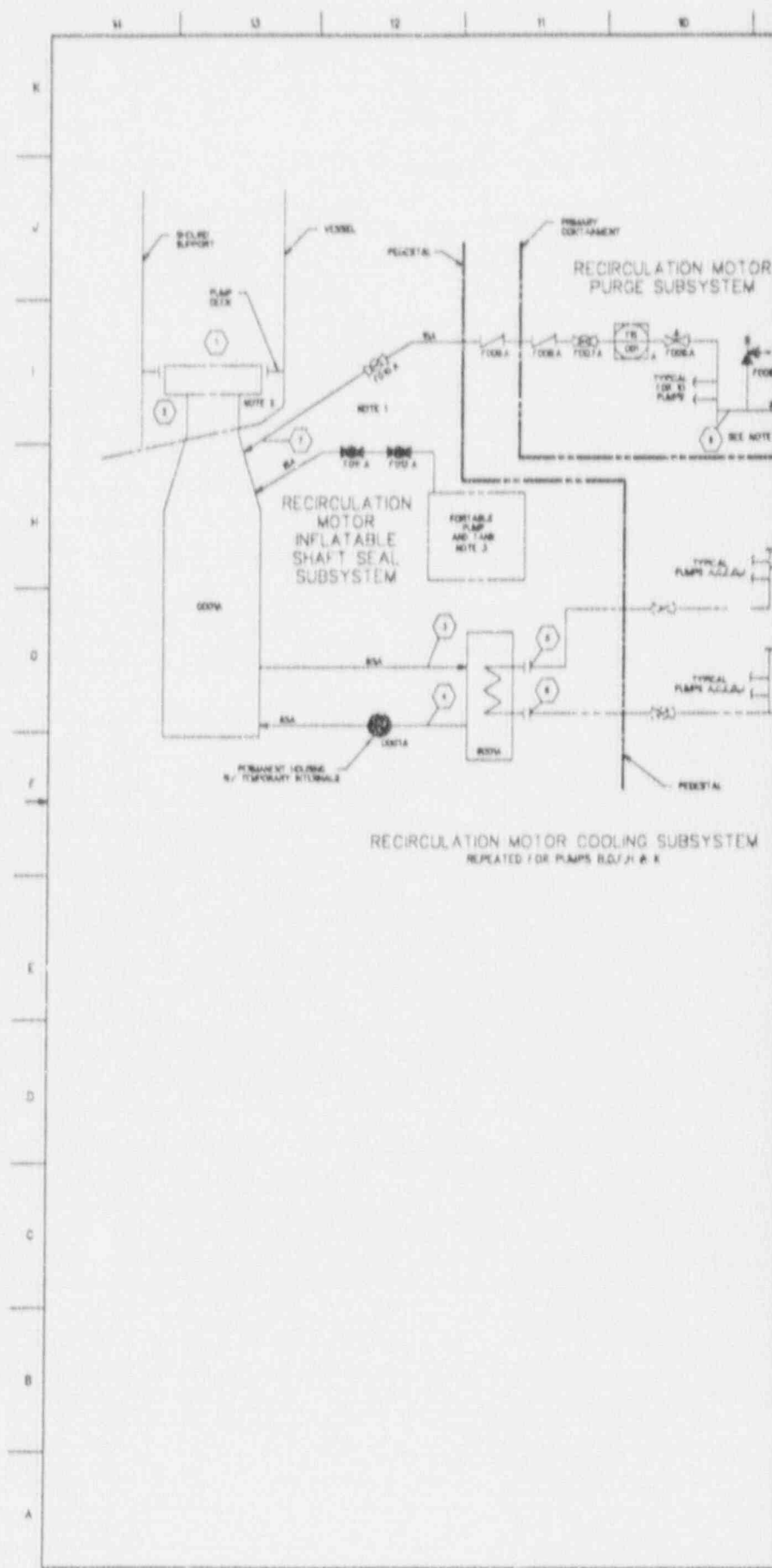
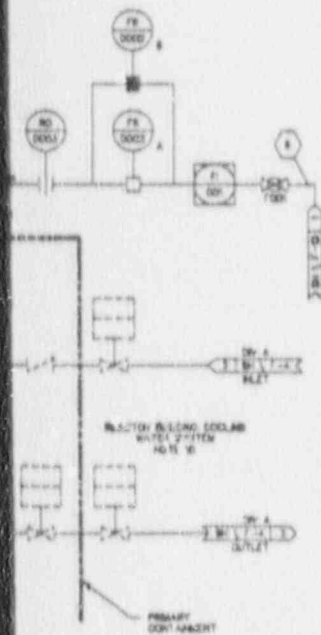


Figure 5.4-4 REA









MODE "A" REACTOR NORMAL OPERATOR TO PUMPS OPERATING  
ALL PUMPS ARE AT SAME SPEED  
CORE FLOW = 750 --- RATED DESIGN  
PUMP SPEED = 60% OF RATED NOTE 8

POSITION	1	2	3	4	5	6	7	8
FLOW g/HR	750	750	8	8	30	30	15/70	150/70
TEMP °C	270	270	60	30	30		30/70	40/80
PRESS	75.8	75.8	77.6	77.2	Δ P = 10 BAR			80
Sp/rev <sup>2</sup>	20/1000.2							
AVAILABLE	80	80						
MPN								

MODE "B" REACTOR NORMAL OPERATOR TO PUMPS OPERATING  
ALL PUMPS ARE AT SAME SPEED  
CORE FLOW = 1000 --- RATED DESIGN  
PUMP SPEED = 80% OF RATED NOTE 8

POSITION	1	2	3	4	5	6	7	8
FLOW g/HR	1000	1000	7	7	30	30	15/70	150/70
TEMP °C	270	270	60	30	30		30/70	40/80
PRESS	75.8	75.8	77.6	77.2	Δ P = 10 BAR			80
Sp/rev <sup>2</sup>	20/1000.2							
AVAILABLE	80	80						
MPN								

MODE "C" REACTOR NORMAL OPERATOR TO PUMPS OPERATING  
ONE PUMP OUT OF SERVICE  
CORE FLOW = 600 --- RATED DESIGN  
PUMP SPEED = 60% OF RATED NOTE 8

POSITION	1	2	3	4	5	6	7	8
FLOW g/HR	600	600	8	8	30	30	15/70	150/70
TEMP °C	270	270	60	30	30		30/70	40/80
PRESS	75.8	75.8	77.6	77.2	Δ P = 10 BAR			80
Sp/rev <sup>2</sup>	20/1000.2							
AVAILABLE	80	80						
MPN								

MODE "D" REACTOR NORMAL OPERATOR TO PUMPS OPERATING  
ALL PUMPS ARE AT SAME SPEED  
CORE FLOW = 600  
PUMP SPEED = 60% OF RATED

POSITION	1	2	3	4	5	6	7	8
FLOW g/HR	670	670	NAT. DRG.		30	30	15/70	150/70
TEMP °C	270	270			30		30/70	40/80
PRESS	75.8	75.8	77.6	77.2	Δ P = 10 BAR			80
Sp/rev <sup>2</sup>	20/1000.2							
AVAILABLE	80	80						
MPN								

MODE "E" REACTOR NOT STANDBY --- NO PUMPS OPERATING  
ALL PUMPS ARE AT SAME SPEED  
CORE FLOW = 1100  
PUMP SPEED = 40% MPN

POSITION	1	2	3	4	5	6	7	8
FLOW g/HR	1100	1100	1	1	30	30	15/70	150/70
TEMP °C	280	280	30	30	30		30/70	40/80
PRESS	80.1	80.1	76.3	76.0	Δ P = 10 BAR			80
Sp/rev <sup>2</sup>	20/1000.2							
AVAILABLE	80	80						
MPN								

MODE "F" REACTOR COLD STARTUP --- NO PUMPS OPERATING  
ALL PUMPS ARE AT SAME SPEED  
CORE FLOW = 1100  
PUMP SPEED = 40% MPN

POSITION	1	2	3	4	5	6	7	8
FLOW g/HR	1100	1100	1	1	30	30	15/70	150/70
TEMP °C	100	100	30	30	30		30/70	40/80
PRESS	18	18	18	18	Δ P = 10 BAR			80
Sp/rev <sup>2</sup>	20/1000.2							
AVAILABLE	80	80						
MPN								

### NOTES

- ALL VALUES SHOWN IN THEIR NORMAL PLANT OPERATING POSITION AND ARE IN THE SAME POSITION FOR ALL OPERATING MODES.
- THE MIXER FLOW RATE FLOWS IN TO THE REACTOR AT 1.
- THE RECIRCULATION MOTOR REPELLABLE SHAFT SEAL SUBSTITUTION IS USED ONLY DURING SHUTDOWNS.
- EXCITATION WEAR AND TEAR CONDITIONS.
- VALUES GIVEN FOR POSITION 1-7 ARE FOR LOADS ONE OF THE PUMPS OPERATING IN THE DESIGN ROOM CONDITION IN OR IN PUMPS OPERATING.
- PUMP RATED SPEED ASSUMED AT 1800 RPM.
- THIS VALUE IS MAXIMUM REVERSE FLOW.
- TOP VALUES INCLUDE 1 PERCENT MARGIN.
- POSITION "8" CREATED EXCLUSIVELY FOR DESIGNING DESIGN PROBLEMS/TEMPERATURE CONDITIONS THAT PRESENT CONDITIONS.
- HOW EVER FOR RPS 8.5/A/J, B SIMILAR TO RPS 8.5/A/J.
- POSITIONS 7 & 8 FLOW IS GRAVITY-DRIVEN FOR ALL MODES.

### REFERENCE DOCUMENTS UNDER THE FOLLOWING IDENTIFIERS ARE TO BE USED IN CONSULTATION WITH THE DESIGNER

- | IDENTIFIER                                  | MPN NO.  |
|---|----------|
| 1. CONTROL ROD DRIVE SYS. PFD               | 101-0300 |
| 2. REACTOR BUILDUP COOLANT WATER SYSTEM PFD | 101-0300 |

### SUPPORTING DOCUMENTS

- | IDENTIFIER                               | MPN NO. |
|--|---------|
| 1. Pumps and Motor Symbol                | 40-3030 |
| 2. Steady State Performance Requirements | 40-4000 |
| 3. Reactor Core Sys. PFD                 | 67-0300 |
| 4. Reactor Core Sys. Design Spec         | 67-4000 |
| 5. ABWR Document Stagger                 | 40-3000 |

Figure 5.4-5 REACTOR RECIRCULATION SYSTEM PFD







# ABWR Standard Plant

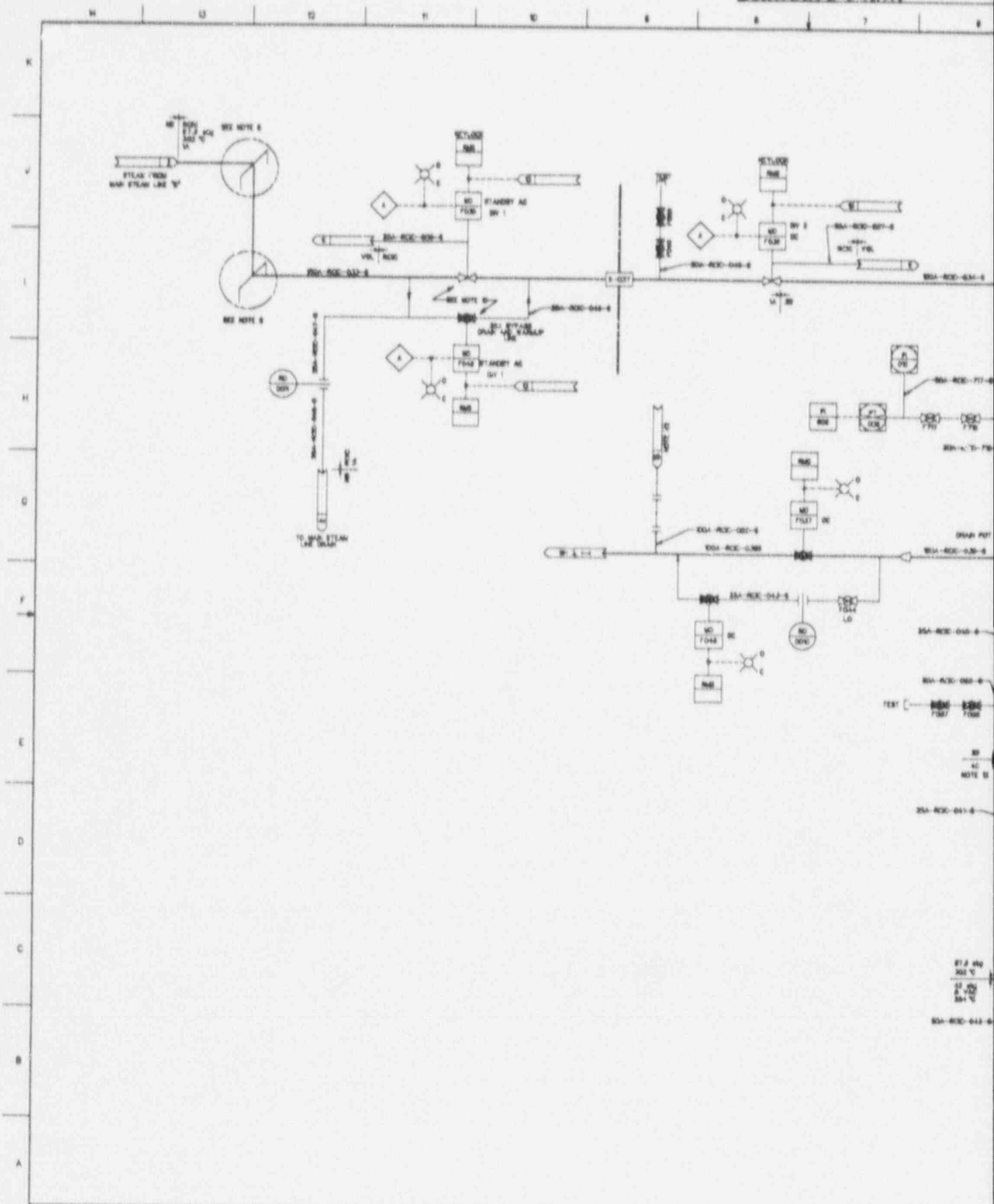
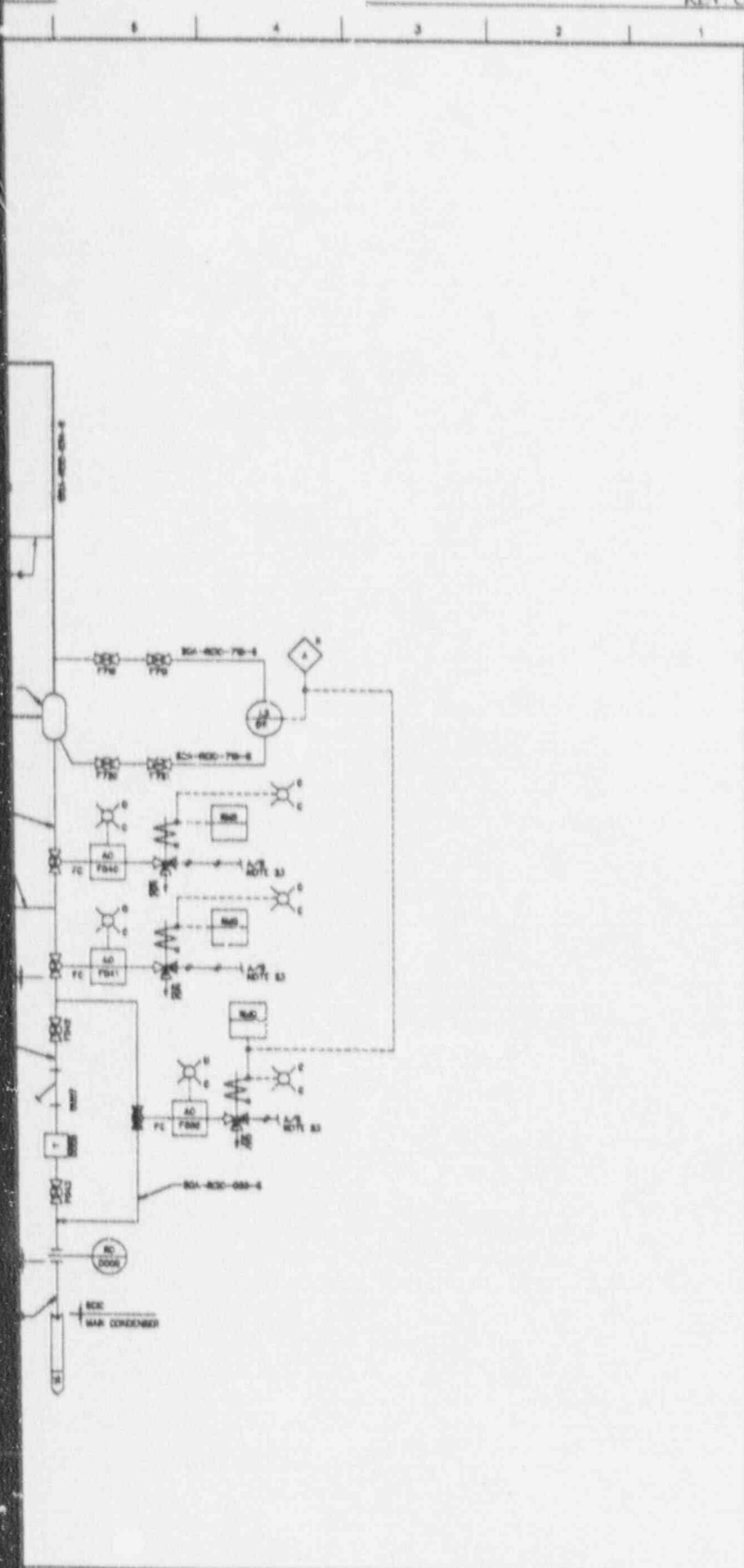


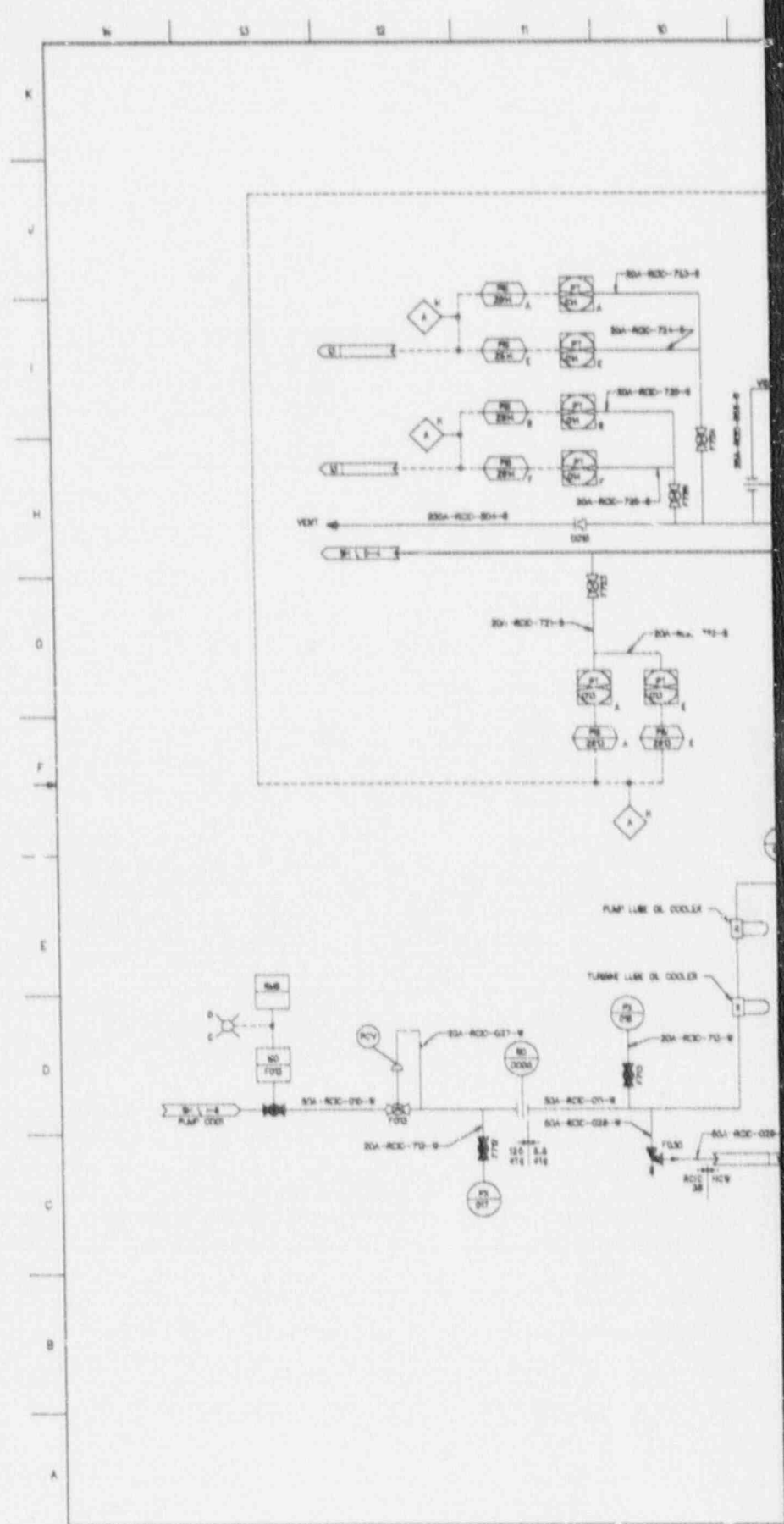
Figure 5.4-8 REACTOR

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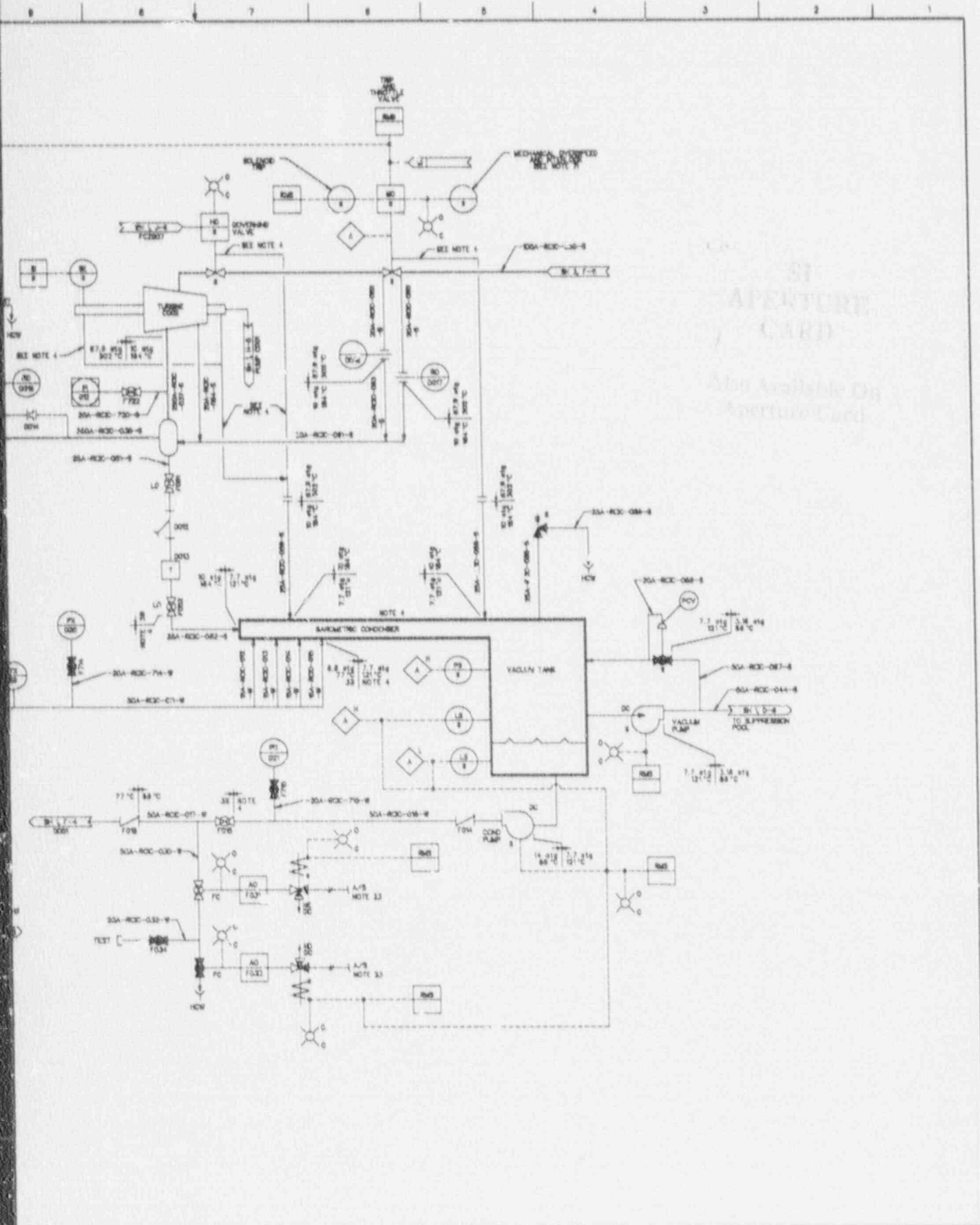
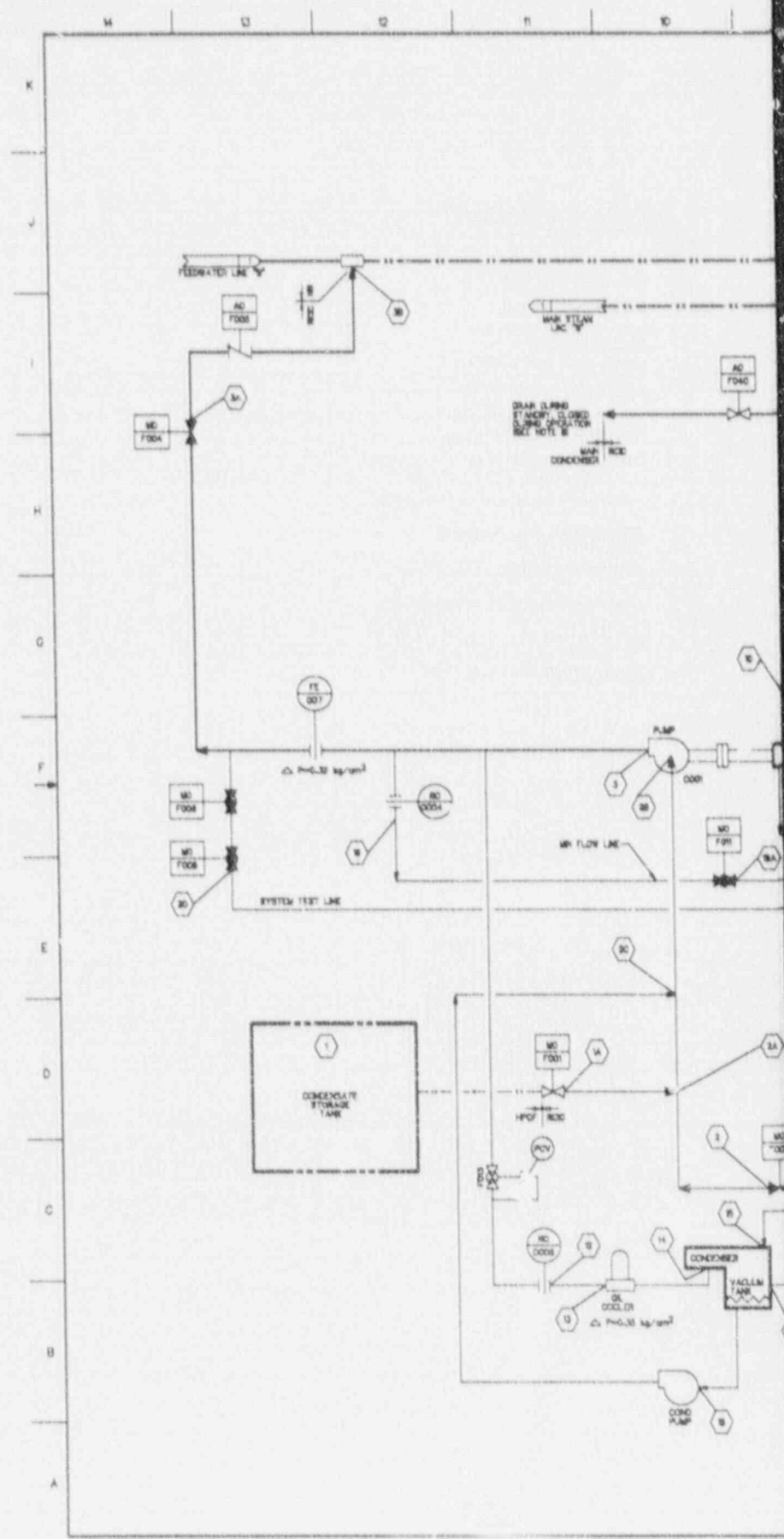
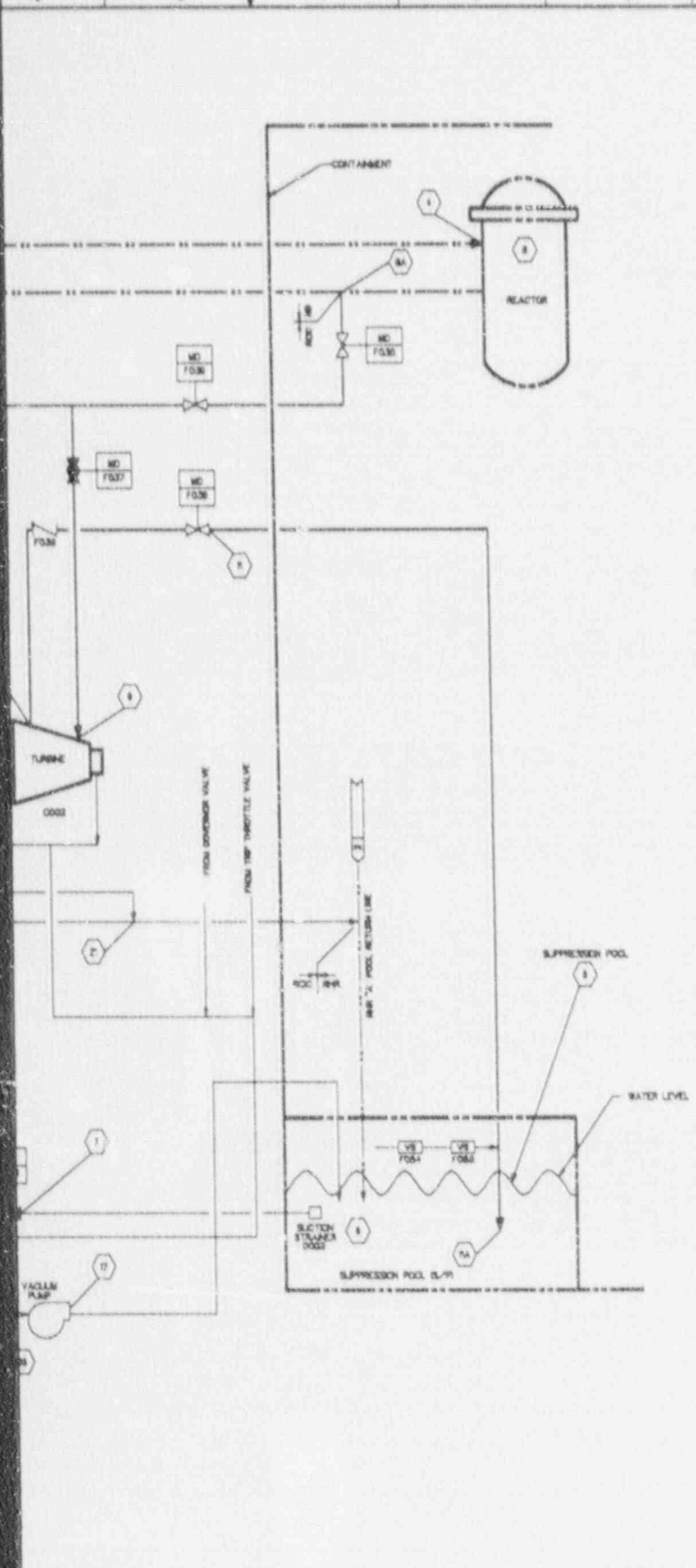


Figure 5.4-8 REACTOR CORE ISOLATION COOLING SYSTEM P&ID, Sheet 3 of 3







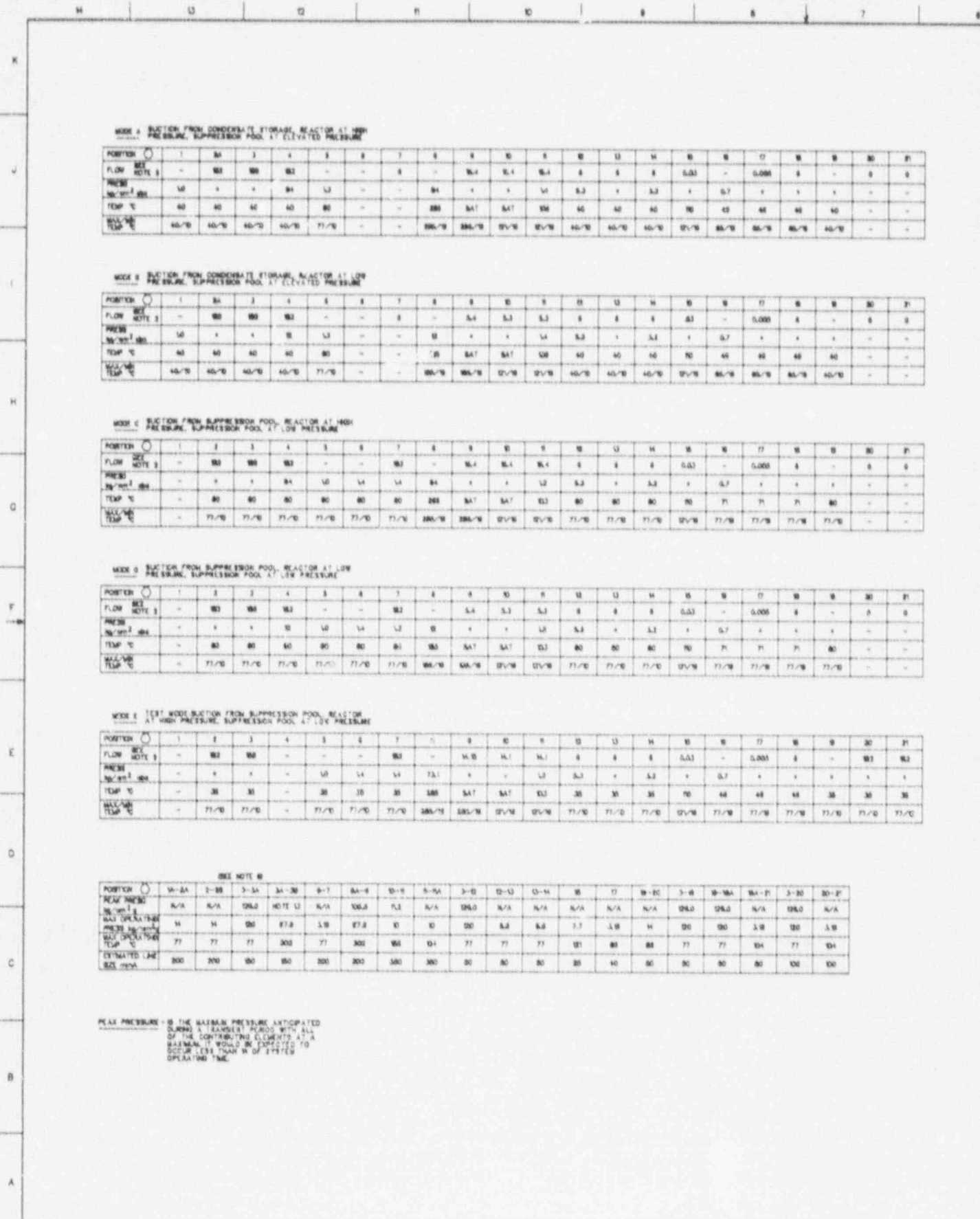
- NOTES
1. ATMOSPHERIC PRESSURE OF  $101.3 \text{ kPa}$  WAS USED IN CALCULATIONS.
  2. WATER FLOWS ARE SHOWN IN  $\text{m}^3/\text{hr}$ . STEAM FLOWS IN  $\text{kg}/\text{hr}$ .
  3. THE UNCOVERED FLOW ORIFICE PRESSURE DROP OF  $0.2 \text{ kg}/\text{cm}^2$  PER ORIFICE IS A FIXED LOSS BETWEEN PORTIONS (3) AND (4).
  4. THE LOSS OF COOLER PRESSURE DROP OF  $0.35 \text{ kg}/\text{cm}^2$  IS A FIXED LOSS BETWEEN PORTIONS (1) AND (2).
  5. THE CONTROLLING MODES FOR LINE BEING AND ARRANGEMENT ARE:  
 SLURRY FROM CONDENSATE STORAGE MODE A & B  
 SLURRY FROM SUPPRESSION POOL MODE C & D  
 FLAP DISCHARGE MODE C & D  
 STEAM SUPPLY MODE A & B  
 TURBINE EXHAUST MODE A & C  
 TEST LINE MODE E  
 COOLING SYSTEM MODE A
  6. SYSTEM OPERATION IS POSSIBLE WITH INTERMEDIATE PRESSURES IN THE REACTOR VESSEL AND THE SUPPRESSION POOL. HOWEVER, THESE CONDITIONS DO NOT CONTROL PIPE OR VALVE SIZES OR SPECIFICATIONS, AND NO DATA IS SHOWN.
  7. FLAP MINIMUM FLOW REQUIREMENT MAY OCCUR DURING ANY OPERATING MODE. FLOW REQUIREMENT IS  $254 \text{ kg}/\text{hr}$  MINIMUM WITH TURBINE FLAP AT MAXIMUM SPEED (SEE DOC. A).
  8. DURING SYSTEM STANDBY EQUIPMENT IS NOT OPERATING. INTERMITTENT FLOW OCCURS THROUGH THE STEAM SUPPLY LINE DRAIN TRAP SYSTEM AT  $10 \text{ kg}/\text{hr}$  AND  $25.7^\circ\text{C}$ .
  9. THE TABLE B FOR REFERENCE ONLY. SEE DOC. P&ID REF. DOC. 4 FOR REQUIRED VALUES.
  10. FLOW VALUES SHOWN IN MODES C & D ARE BASED UPON SLURRY FLOW DESIGN PERMITTING THE MINIMUM REQUIRED FLOW TO CONTINUE TO BE PROVIDED TO THE ROCE FLAP WHEN THE SUPPRESSION POOL SLURRY STRAINED IS BEING RESTART FLOWING.
  11. STEAM FLOWS FOR TEST MODE AT PORTION (1) (2) (3) AND (4) ARE BASED UPON A FLAP TOP OF  $700 \text{ Pa}$ .
  12. DURING ROCE SYSTEM OPERATION A FLOW OF  $0.25 \text{ m}^3/\text{hr}$  OCCURS THROUGH THE TURBINE EXHAUST LINE DRAIN POT SYSTEM AT  $10 \text{ kg}/\text{cm}^2$  AND  $10^\circ\text{C}$ .
  13. SEE REF. DOC. 1 FOR PEAK PRESSURE.

REFERENCE DOCUMENTS UNDER THE FOLLOWING IDENTITIES SHALL BE USED IN CONJUNCTION WITH THIS DRAWING:

1. NUCLEAR BOILER SYSTEM PFD	MS-365
2. REBEHALI HEAT REMOVAL SYSTEM PFD	RT-1030
3. HIGH PRESSURE CORE FLOODED SYS PFD	EE3-6100
4. ROCE SYSTEM P&ID	EE-1010
5. PIPING AND INSTRUMENTATION DIAGRAM SYMBOLS	AS-3030
6. ASME DOCUMENT STANDARDS	AS-3030

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Figure 5.4-9 RCIC SYSTEM PFD, Sheet 1 of 2



Figure

\* THE PRESSURE AT THIS POSITION DEPENDS ON PPMW ARRANGEMENT AND MAY BE VARYING WITHIN THE FOLLOWING LIMITS.

POSITION

- (1) INLET TO PUMP FROM CONDENSATE STORAGE TANK
- (2) MINIMUM SFD = 7.5 D 2 METER ABOVE PUMP FLOOR
- (3) MAXIMUM PUMP TOTAL DYNAMIC HEAD 800 m FOR MODES A & C 900 m FOR MODES E & D
- (4) MAXIMUM PRESSURE DROP BETWEEN POSITION (5) AND (6) = 1.1 kg/cm<sup>2</sup> (SEE NOTE 5)
- (5) MAXIMUM PRESSURE ALLOWED FOR RATED SYSTEM PERFORMANCE = 1.8 kg/cm<sup>2</sup> abs
- (6) MAXIMUM PRESSURE ALLOWED = 6.8 kg/cm<sup>2</sup> abs 2M
- (7) PRESSURE IS 0.35 kg/cm<sup>2</sup> LESS THAN POSITION (6)
- (8) SUFFICIENT VACUUM TO PREVENT TURBINE BRANT-OUT-LAGURE TO BE SPECIFIED ON TURBINE VENDOR DRAWINGS.
- (9) MAXIMUM PRESSURE AVAILABLE = 2.1 kg/cm<sup>2</sup> abs
- (10) MAXIMUM PRESSURE AVAILABLE = 4.2 kg/cm<sup>2</sup> abs
- (11) SUFFICIENT PRESSURE TO RETURN TO SUPPRESSION POOL
- (12) SUFFICIENT PRESSURE TO RETURN TO SUPPRESSION POOL

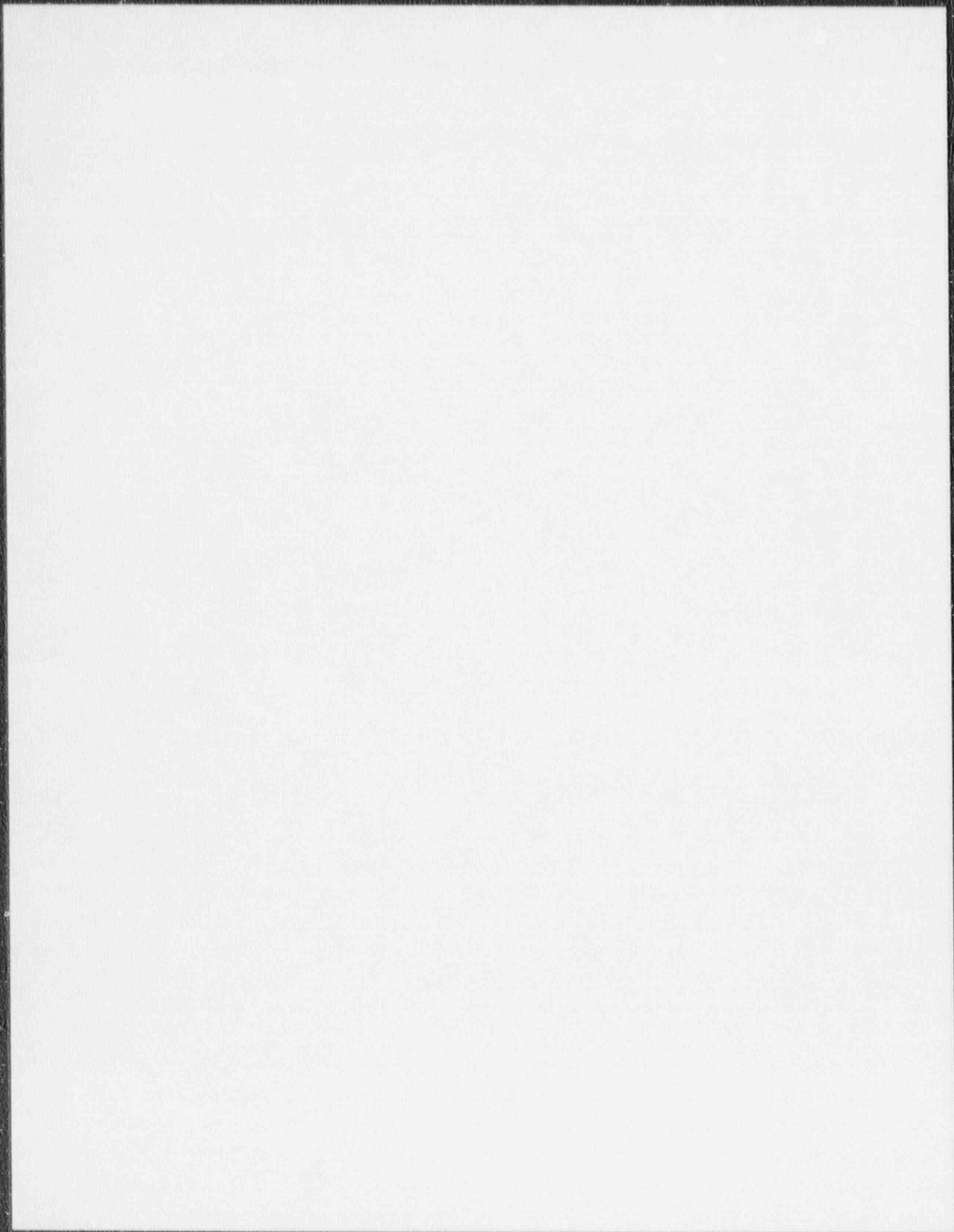
TABLE 1 VALVE POSITION CHART

VALVE	FEH	FEH	FEH	FEH	FEH	FEH	FEH	FEH	FEH	FEH
MODE A	0	C	0	0	0	0	0	0	C	C
MODE B	0	C	0	0	0	0	0	0	C	C
MODE C	0	0	0	0	0	0	0	0	C	C
MODE D	0	0	C	0	0	0	0	0	C	C
MODE E	0	0	C	0	0	0	0	0	C	T

0 = OPEN C = CLOSE T = THROTTLE

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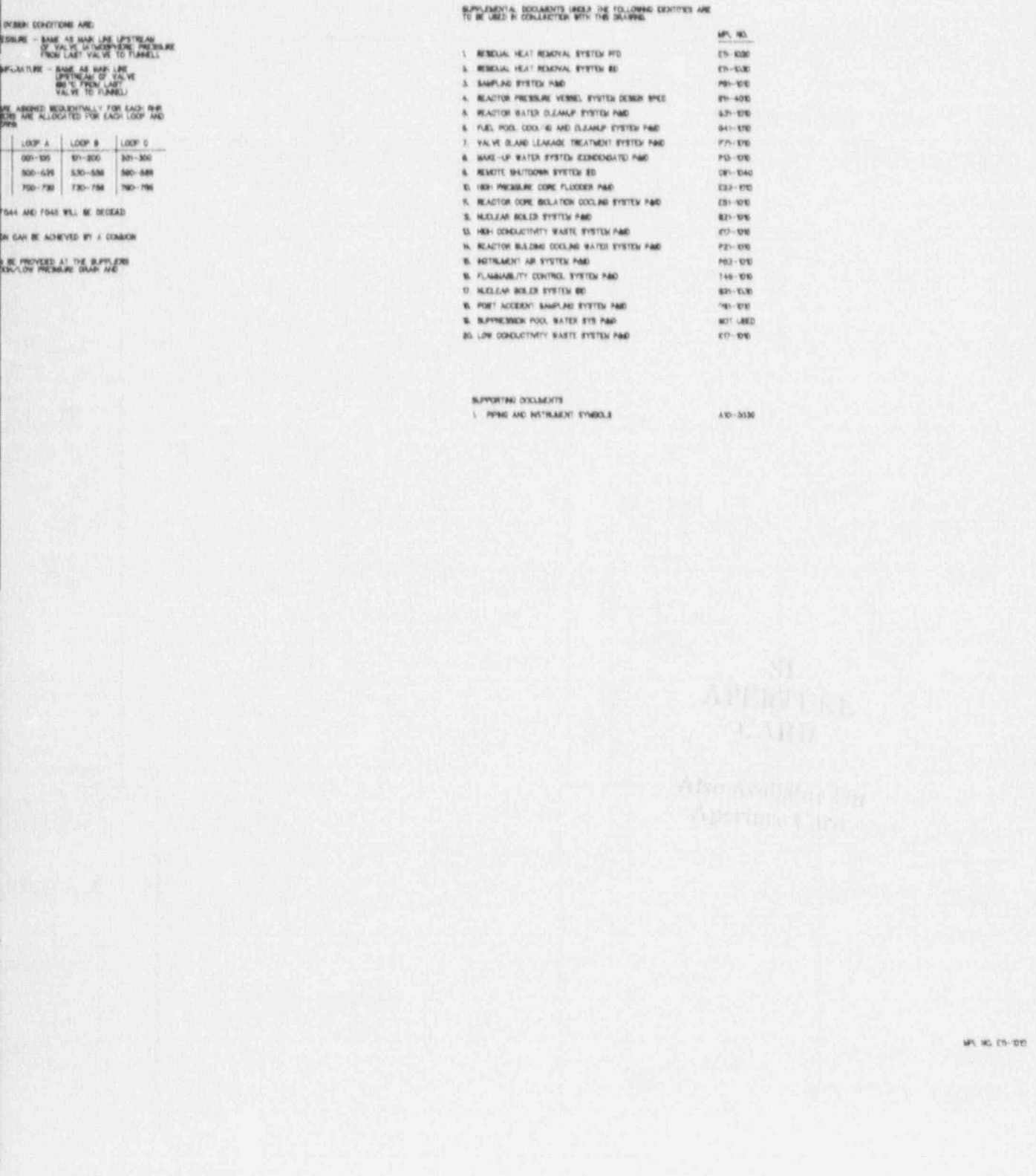
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D  
C  
B  
A

NOTES

1. PPRG HIGH POINT VENTS AND LOW POINT DRAINS ARE TO BE SIZED AS NECESSARY.
2. FITTINGS AND DESIGN ARE VALVES SHALL BE IN ACCORDANCE WITH INSTRUMENT PPRG SPECIFICATION AT-500.
3. VALVE FORM IS REQUIRED IF THERE IS POTENTIAL FOR OVERPRESSURE.
4. FOR ADDITIONAL CONTROL, REED LEADS, SYSTEM ALARMS AND REMOTE MANUAL SWITCHES, SEE THE PPRG BU TO E-300.
5. PROVISIONS FOR CONTAINMENT ISOLATION SHALL BE IN ACCORDANCE WITH CURRENT LICENSE REQUIREMENTS.
6. VALVE FORM SHALL BE LOCATED AT AN ELEVATION LOWER THAN THE SUPPRESSION POOL MINIMUM WATER LEVEL.
7. PUMP COOLING WATER, IF REQUIRED, IS SPECIFIED IN P21-000.
8. VALVES FORM FORM & FIT SHALL BE LOCATED AS CLOSE AS POSSIBLE TO THE CONTAINMENT PENETRATION.
9. ALL MOTOR OPERATED VALVES ARE AS OPERATED UNLESS OTHERWISE NOTED.
10. EQUIPMENT IN SUBSYSTEMS A, B AND C SHALL HAVE THE SUFFIX LETTER A, B AND C RESPECTIVELY AFTER THE EQUIPMENT NUMBER.
11. FLOWING CONNECTIONS AND TEMPORARY B-RAMMED CONNECTIONS ON THE REACTOR SIDE OF ALL DRAINS SHALL BE PROVIDED IN ACCORDANCE WITH A70-500.
12. ALL MOTOR OPERATED VALVES AND OPERATED VALVES AND ALL PUMPS SHALL HAVE A LOCAL NAME, REMOTE MANUAL SWITCH AND STATUS LIGHT THAT WILL BE HARD WIRED. IN ADDITION, THERE WILL BE A REDUNDANT REMOTE MANUAL SWITCH AND STATUS LIGHT CONNECTED BY MEANS OF MULTIPLEXING AND LOCATED IN THE MAIN CONTROL ROOM.
13. DRYWELL PPRG PUMPS SHALL BE HORIZONTAL OR VERTICAL UPWARD FROM THE DRYWELL, SHALL TO THE POINT OF ATTACHMENT WITH THE REACTOR VESSEL.
14. THE HIGH POINT VENT SHALL BE LOCATED AT THE HIGHEST POINT IN THE PPRG OUTSIDE THE DRYWELL BETWEEN VALVES F001 AND F002.
15. SUBSYSTEM A RETURN TO RPV THROUGH FEEDWATER LINE A.
16. DISCHARGE LINES FOR COOLING WATER TO BE ROUTED UPSTREAM OF SERVICE WATER RADIATION MONITORS.
17. VALVE FORM SHALL BE AS CLOSE AS POSSIBLE TO THE CONNECTIONS TO THE MAIN LINE.
18. DESIGN LINE SIZE WILL BE FINALIZED AT THE DETAILED DESIGN PHASE. ACTUAL LINE SIZES DETERMINED BY THE PPRG DESIGNER SHALL MEET THE PROCESS DATA HYDRAULIC REQUIREMENTS.
19. CHECK VALVE FORM SHALL BE LOCATED AS CLOSE AS PRACTICAL TO THE REACTOR VESSEL NOZZLE.
20. VALVES FORM FORM AND FORM ARE IN ELECTRICAL DIVISIONS B, C AND A RESPECTIVELY. THE MANUAL CONTROL SWITCHES FOR VALVES FORM FORM AND FORM ARE IN ELECTRICAL DIVISIONS A, B AND C RESPECTIVELY.
21. PPRG DESIGN SPECIFICATIONS ARE AS FOLLOWS:
  - A. MAXIMUM OPERATING PRESSURE - SEE SPECIFIC BOUNDARY SYMBOL.
  - B. MAXIMUM OPERATING TEMPERATURE - SEE SPECIFIC BOUNDARY SYMBOL.
  - C. MATERIAL - CARBON STEEL.
  - D. PPRG SCHEDULE - INTERFACE.
  - E. DESIGN CLASS - SEE SPECIFIC BOUNDARY SYMBOL.
  - F. QC CLASS - SEE SPECIFIC BOUNDARY SYMBOL.
  - G. SERVICE CLASS - PWR - AN.
  - H. FILL - WATER.
  - I. INTERFACE.
22. AIR SUPPLY AND NITROGEN SUPPLY SHOWN IN SUPPLEMENTAL DOCUMENT B.
23. STRAINER TYPE AS SUPPLIED WITH PUMP COOL.
24. FLOWING CONNECTION USED FOR OCCASIONAL SUPPRESSION POOL DRAINING.

25. DRAIN AND VENT PPRG  
 MAXIMUM OPERATING PRESSURE  
 MAXIMUM OPERATING TEMPERATURE  
 26. INSTR. PPRG NUMBER  
 LOOP NUMBER OF ALARM  
 TYPE OF PPRG AS FOLLOWS  
 PROCESS PPRG  
 DRAIN AND VENT PPRG  
 INSTRUMENT PPRG  
 27. THE VALVE TYPE FOR  
 B. THE FINAL DESIGN.  
 28. THE RECORDING FLUCTUATION  
 RECORDING DEVICE.  
 29. TWO ROOT VALVES CAN  
 OPEN ON EACH RADIATION  
 VENT LINE.





DESIGN CONDITIONS ARE:  
DESIGNURE - NAME AS MARK LINE UPSTREAM  
OF VALVE UNLESS OTHERWISE SPECIFIED  
FROM LAST VALVE TO TUNNELL

MANUFACTURE - NAME AS MARK LINE  
UPSTREAM OF VALVE  
UNLESS OTHERWISE SPECIFIED  
FROM LAST VALVE TO TUNNELL

DESIGN ARE ABRIDGED INDICATIVELY FOR EACH P&ID  
WHERE ARE ALLOCATED FOR EACH LOOP AND  
DESIGN

LOOP A	LOOP B	LOOP C
001-105	011-200	021-300
500-625	530-585	580-685
700-730	730-795	780-795

SUPPLEMENTAL DOCUMENTS UNDER THE FOLLOWING IDENTITIES ARE  
TO BE USED IN CONNECTION WITH THIS DRAWING.

	REF. NO.
1. RESIDUAL HEAT REMOVAL SYSTEM P&ID	05-000
2. RESIDUAL HEAT REMOVAL SYSTEM SD	05-000
3. SAMPLING SYSTEM P&ID	05-010
4. REACTOR PRESSURE VESSEL SYSTEM DESIGN SPEC	05-400
5. REACTOR WATER CLEANUP SYSTEM P&ID	05-010
6. FUEL POOL COOLING AND CLEANUP SYSTEM P&ID	041-010
7. VALVE ISLAND LEAKAGE TREATMENT SYSTEM P&ID	071-010
8. MAKE-UP WATER SYSTEM CONDENSATE P&ID	05-010
9. REMOTE SHUTDOWN SYSTEM SD	05-040
10. HIGH PRESSURE CORE FLOODER P&ID	023-010
11. REACTOR CORE ISOLATION COOLING SYSTEM P&ID	051-010
12. NUCLEAR BOILER SYSTEM P&ID	021-010
13. HIGH CONDUCTIVITY WASTE SYSTEM P&ID	07-010
14. REACTOR BUILDING COOLING WATER SYSTEM P&ID	021-010
15. INSTRUMENT AIR SYSTEM P&ID	052-010
16. FLAMMABILITY CONTROL SYSTEM P&ID	145-010
17. NUCLEAR BOILER SYSTEM SD	021-010
18. POST ACCIDENT SAMPLING SYSTEM P&ID	051-010
19. SUPPRESSOR POOL WATER SYS P&ID	NOT USED
20. LOW CONDUCTIVITY WASTE SYSTEM P&ID	07-010

SUPPORTING DOCUMENTS  
1. P&ID AND INSTRUMENT SYMBOLS  
A10-000

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APERTURE CARD  
A10  
Also Available on  
Aperture Card

REF. NO. 05-000

Figure 5.4-10 RESIDUAL HEAT REMOVAL SYSTEM P&ID, Sheet 1 of 7



# ABWR Standard Plant

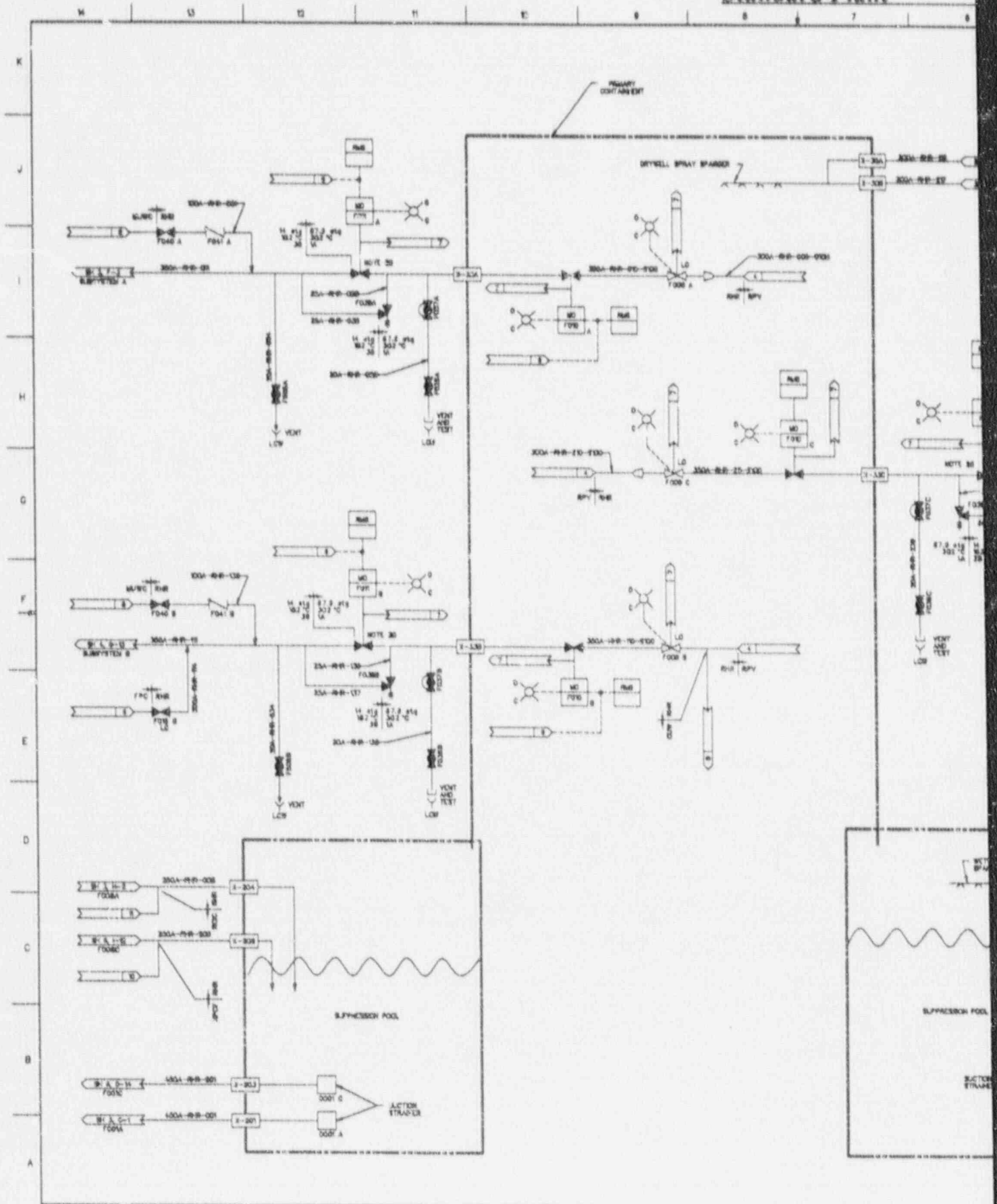
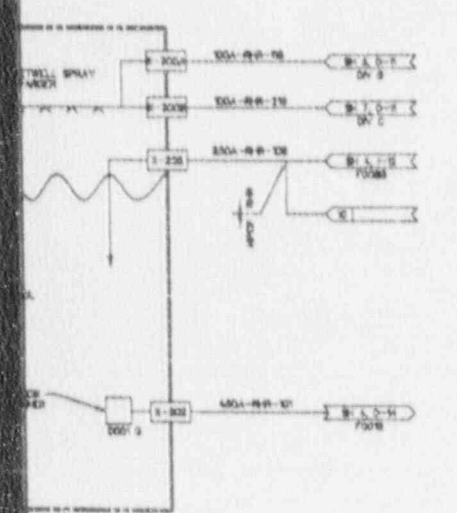
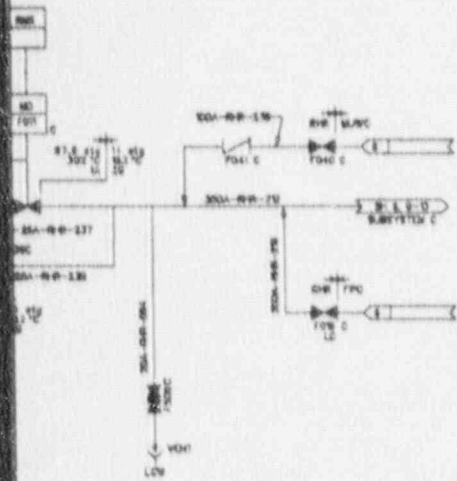
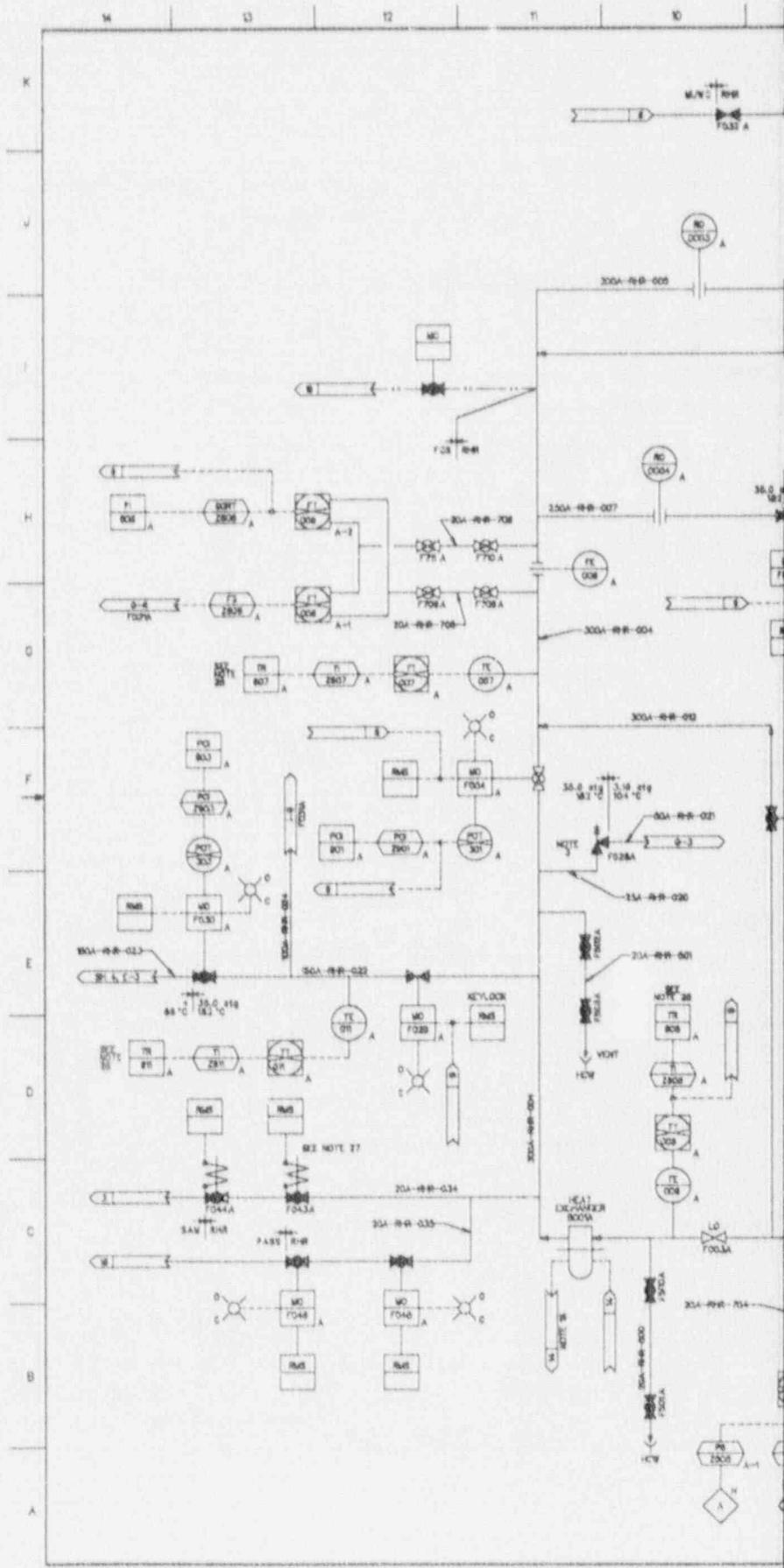


Figure 5.4-10 R

WATER  
LEVEL



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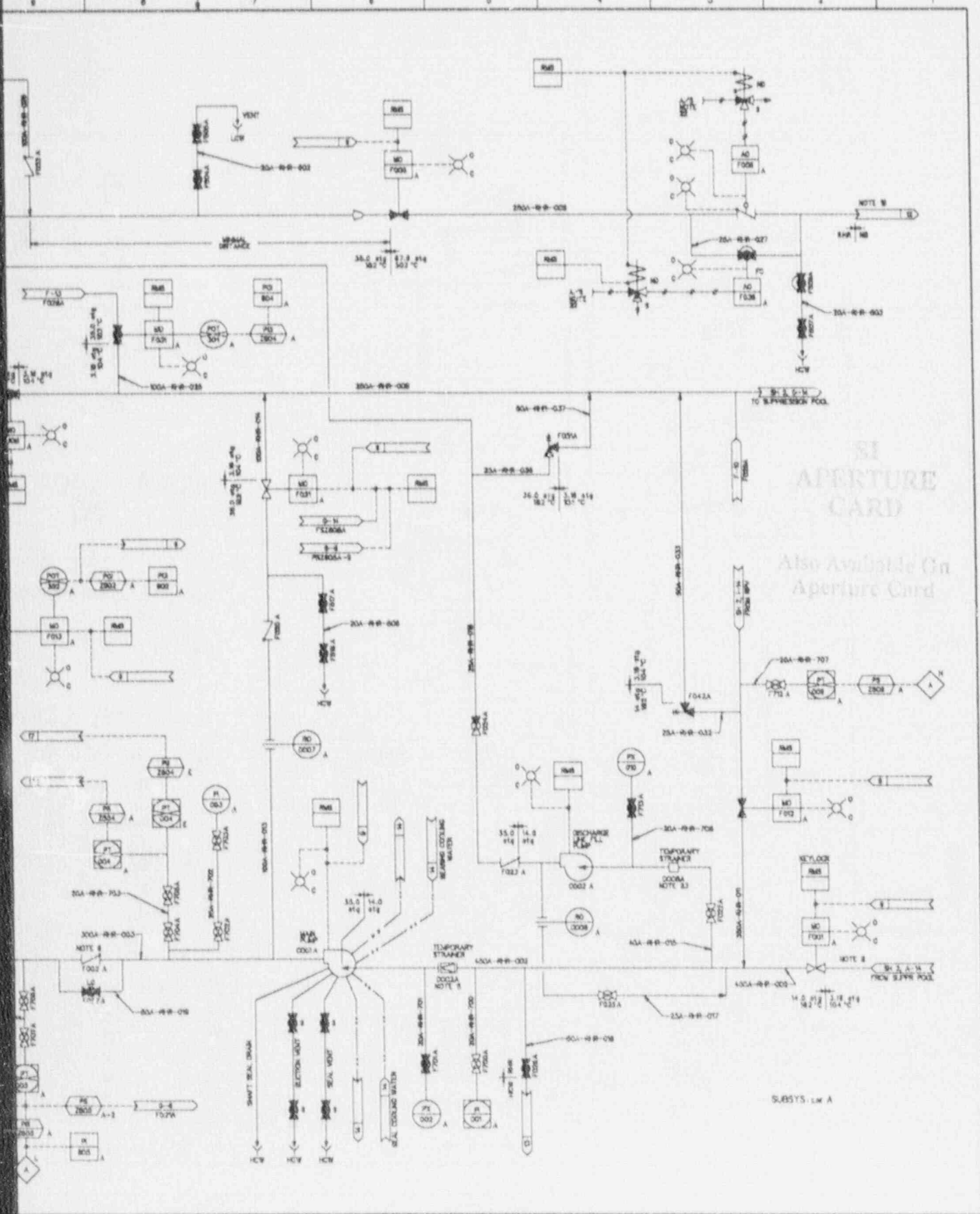


Figure 5.4-10 RESIDUAL HEAT REMOVAL SYSTEM P&ID, Sheet 3 of 7

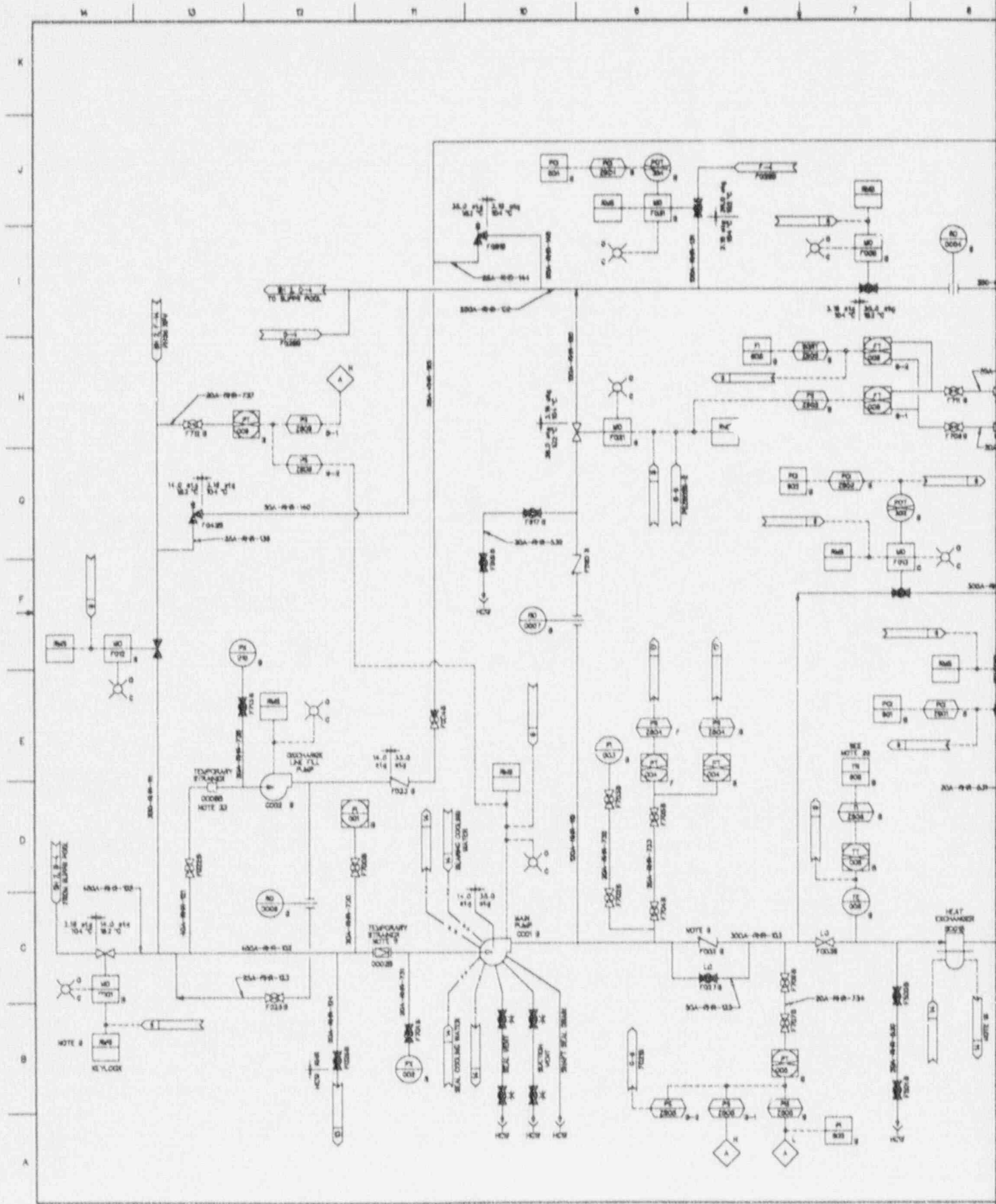
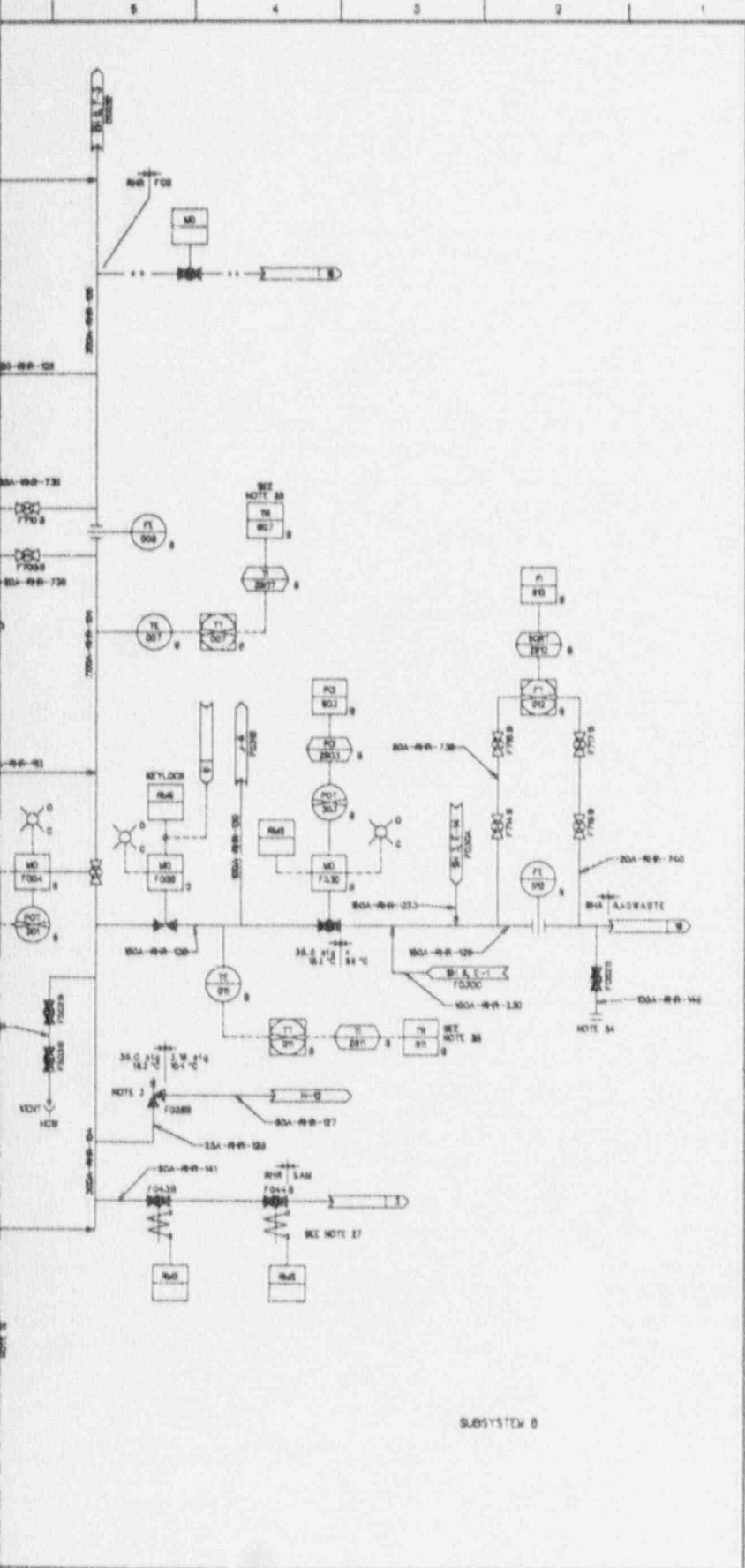


Figure 5.4-10 RE

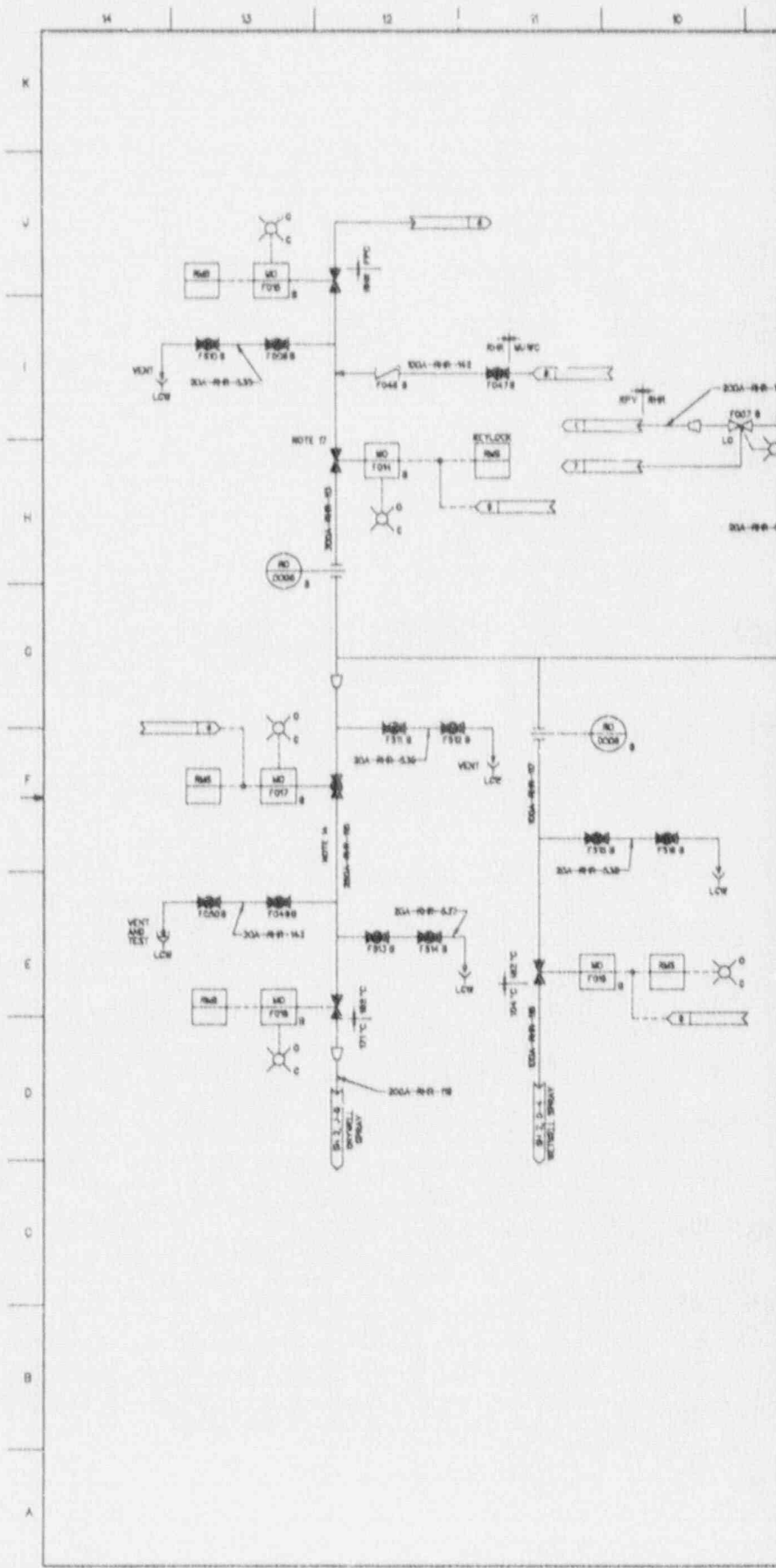




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CARD  
Also Available On  
Aperture Card

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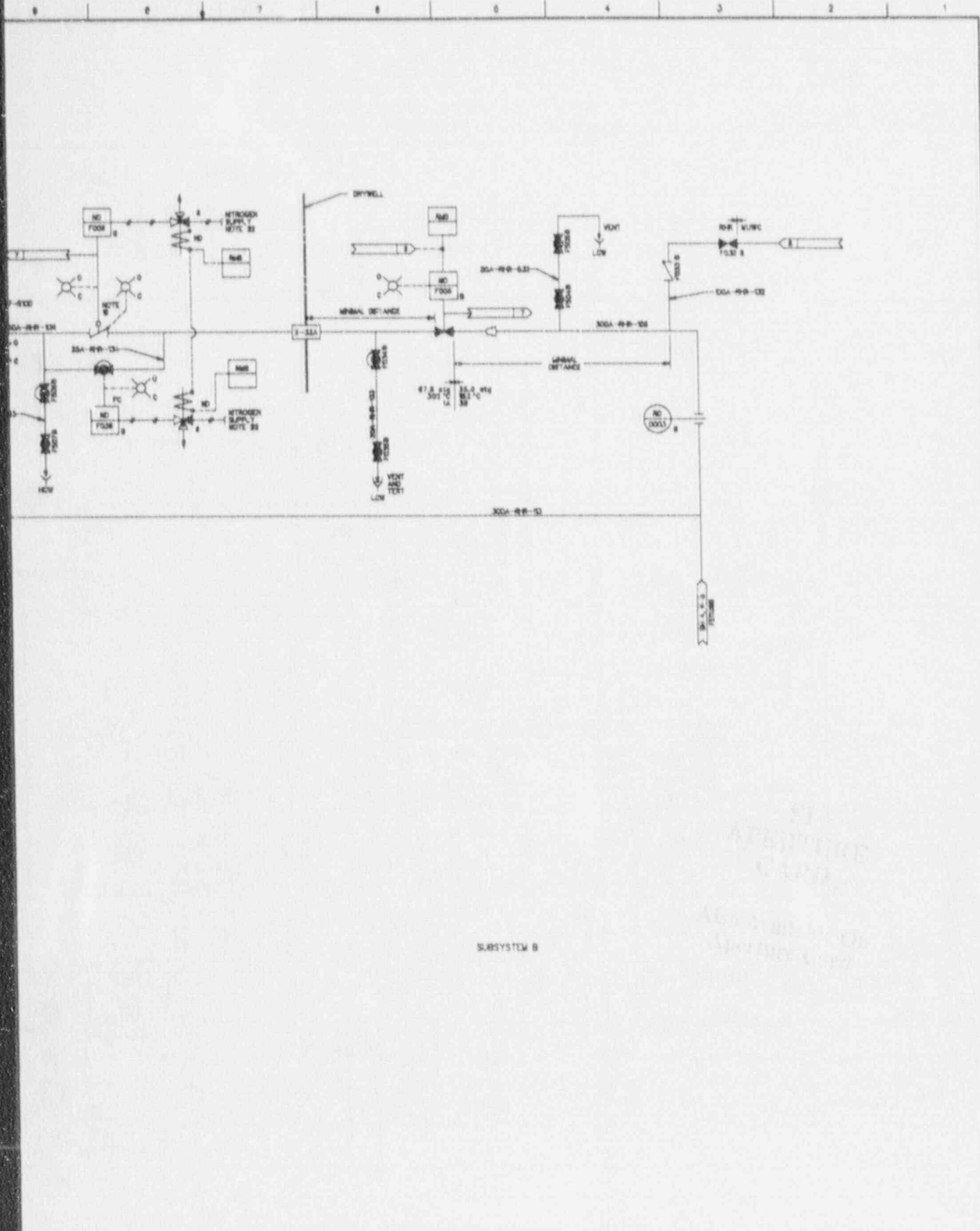


Figure 5.4-10 RESIDUAL HEAT REMOVAL SYSTEM P&ID, Sheet 5 of 7

# ABWR Standard Plant

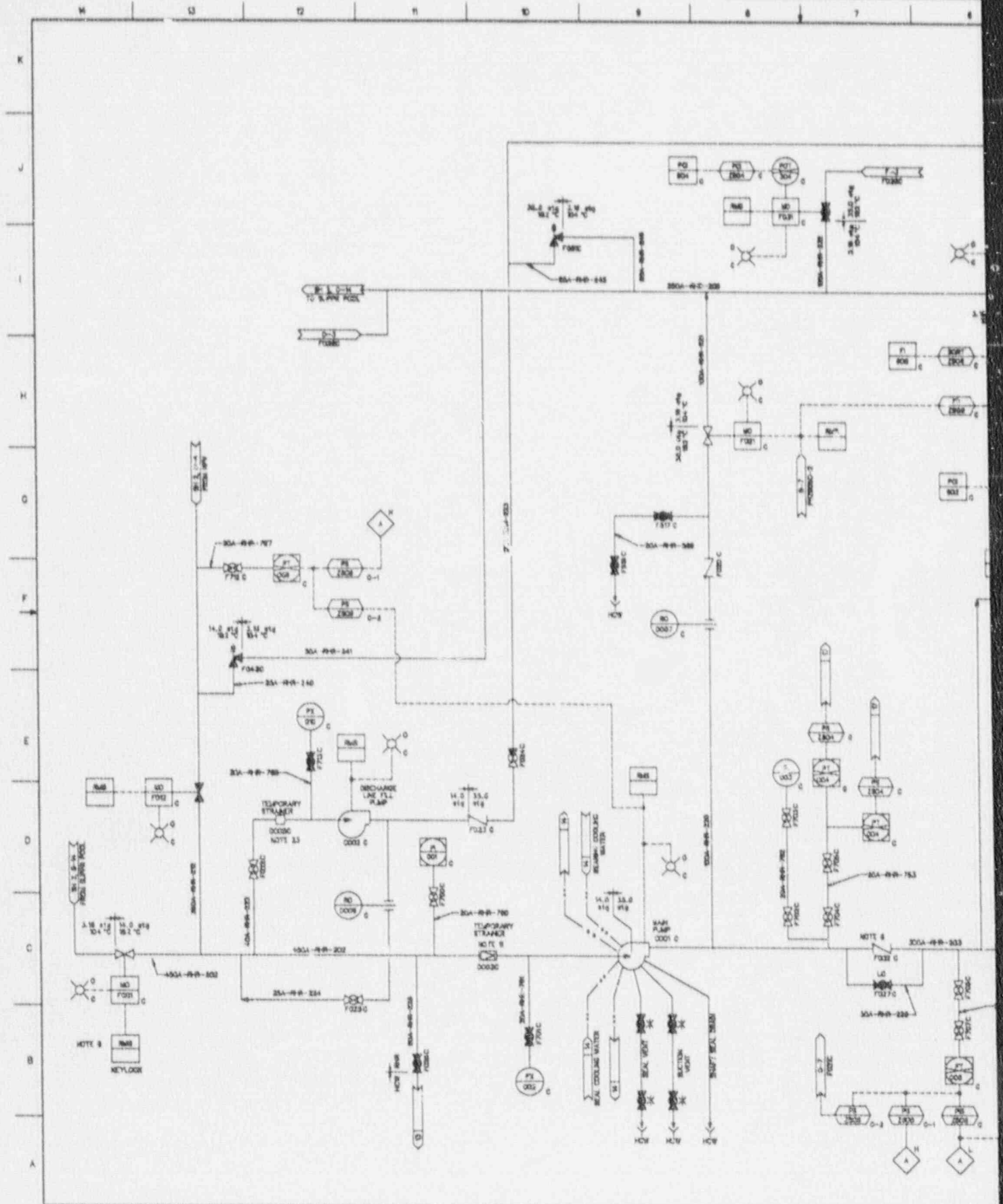
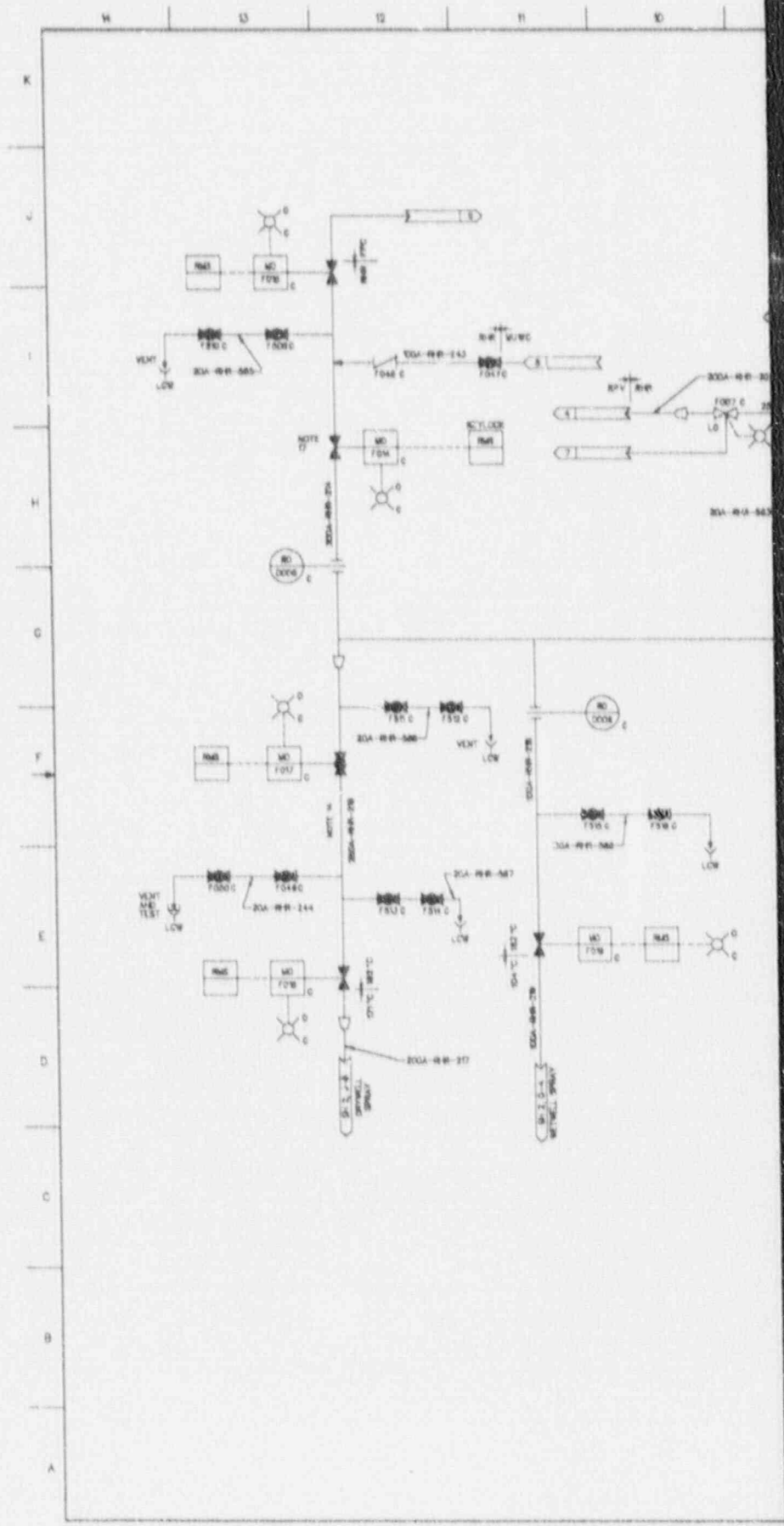


Figure 5.4-10 RE







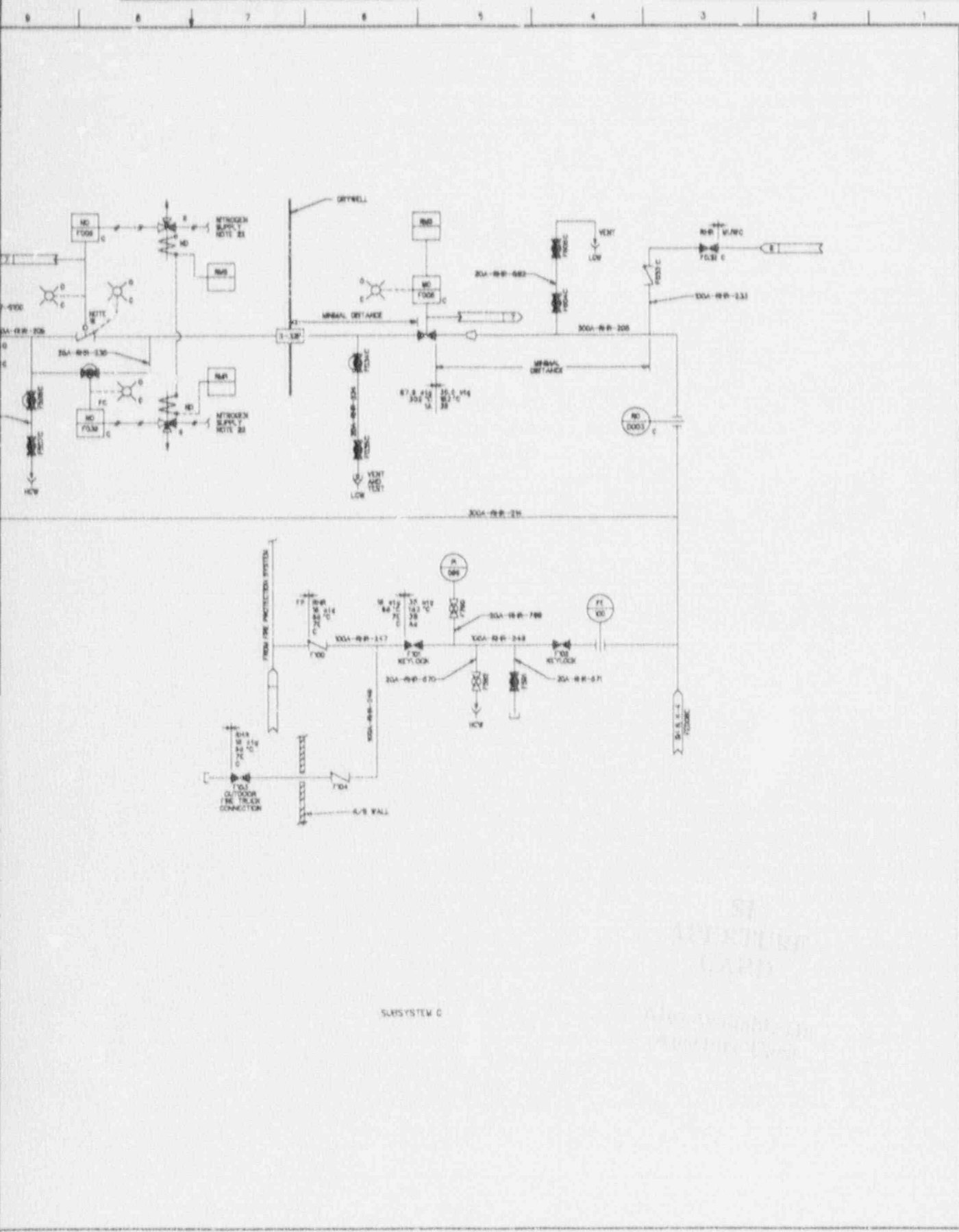


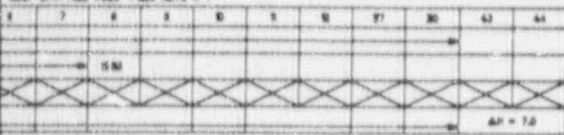
Figure 5.4-10 RESIDUAL HEAT REMOVAL SYSTEM P&ID, Sheet 7 of 7



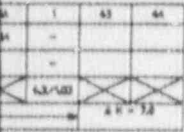




LOOP WITH FUEL POOL (SEE NOTE 1)



VIEW B OF C



MODES

- A. POST ACCIDENT CORE COOLING WITH STRAINER BOX PLUGGED.
- B-1. POST ACCIDENT SUPPRESSION POOL COOLING WITH STRAINER BOX PLUGGED.
- B-2. SUPPRESSION POOL COOLING DURING HOT STANDBY OPERATION.
- C-1. INITIATION OF SHUTDOWN COOLING AFTER SHUTDOWN TO MAIN CONDENSER AT FOUR HOURS.
- C-2. CONTINUATION OF SHUTDOWN COOLING AT 20 HOURS.
- C-3. CONTINUATION OF SHUTDOWN COOLING WITH RETURN TO UPPER CONTAINMENT POOL AT GREATER THAN 20 HOURS.
- D-1. FUEL POOL COOLING - CLOSED LOOP WITH FUEL POOL.
- E. RHR TEST DURING PLANT OPERATION.
- F. POST ACCIDENT CONTAINMENT SPRAY WITH HEAT REJECTION AND SCREEN BOX PLUGGED.
- F. MINIMAL FLOW BYPASS MODE.
- G. SYSTEM ON STANDBY DUTY.

DUTY



TABLE 1 VALVE POSITION CHART (SEE NOTE 2)

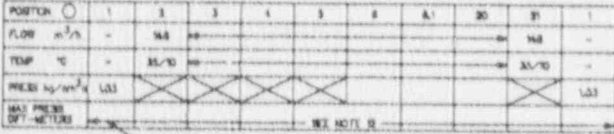
MODE	1	2	3	4	5	6	7	8	9	10	11	12	13	14	15	16	17	18	19	20	21	22	23	24	25	26	27	28	29	30	31	STRAINER
A	0	0	0																												BOX P	
B-1	0	0	0	0																											BOX P	
B-2	0	0	0	0																											BOX P	
C-1	0-1	0		0	0	0	0	0	0	0	0	0	0	0	0	0	0	0	0	0	0	0	0	0	0	0	0	0	0	0	CLEAR	
C-2	0-1	0		0	0	0	0	0	0	0	0	0	0	0	0	0	0	0	0	0	0	0	0	0	0	0	0	0	0	0	CLEAR	
C-3	0-1			0	0	0	0	0	0	0	0	0	0	0	0	0	0	0	0	0	0	0	0	0	0	0	0	0	0	0	CLEAR	
C-4	0-1				0	0	0	0	0	0	0	0	0	0	0	0	0	0	0	0	0	0	0	0	0	0	0	0	0	0	CLEAR	
D	0	0	0	0-1																											CLEAR	
E	0	0	0																												BOX P	
F - POOL	0	0-1																													CLEAR	
G	0	0	0																												CLEAR	

0 = VALVE OPEN  
 T = VALVE THROTTLED  
 1-C = VALVE THROTTLED OR CLOSED  
 0-T = VALVE OPEN OR THROTTLED  
 P = STRAINER PLUGGED  
 BLANK SPACE INDICATE VALVE IS CLOSED

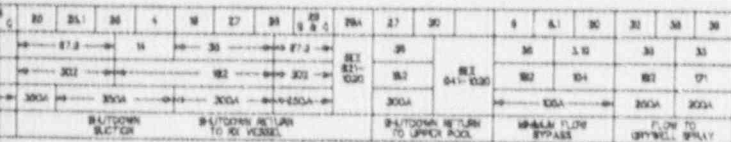
LEGEND

RP PRESS ----- REACTOR VESSEL PRESSURE  
 TOR ----- TOTAL DYNAMIC HEAD  
 SCF ----- SAFETY HEAD  
 ΔH ----- HEAD LOSS

MODE F MINIMAL FLOW BYPASS MODE



ALL SUB SYSTEMS

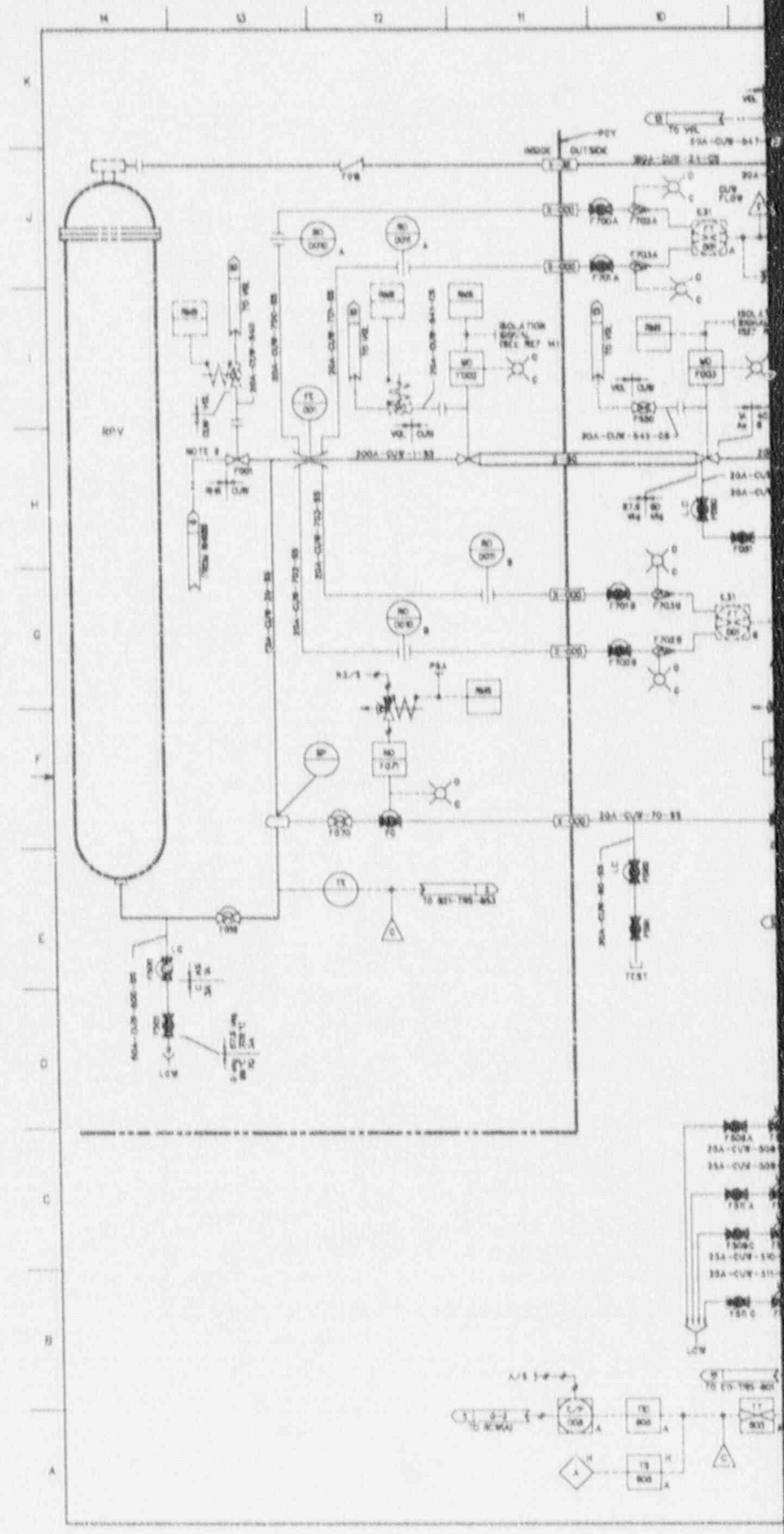


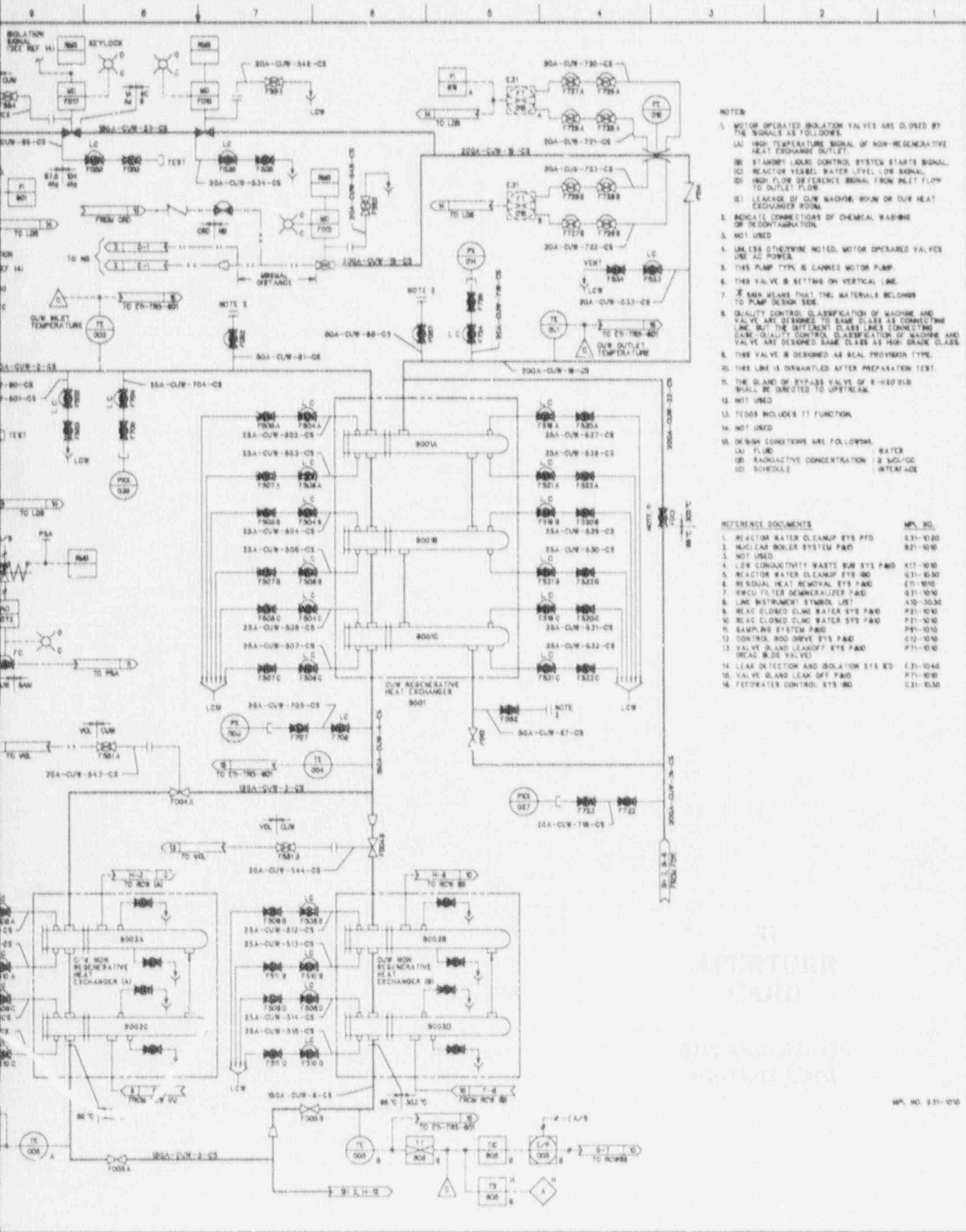
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- NOTES:
- MOTOR OPERATED ISOLATION VALVES ARE CLOSED BY THE SIGNALS AS FOLLOWS:
    - (A) HIGH TEMPERATURE SIGNAL OF NON-REGENERATIVE HEAT EXCHANGER OUTLET.
    - (B) STANDBY LIQUID CONTROL SYSTEM STARTS SIGNAL.
    - (C) REACTOR YESSEL WATER LEVEL LOW SIGNAL.
    - (D) HIGH FLOW DIFFERENCE SIGNAL FROM INLET FLOW TO OUTLET FLOW.
    - (E) LEAKAGE OF CUW WASHIE ROOM OR CUW HEAT EXCHANGER ROOM.
  - INDICATE CONNECTIONS OF CHEMICAL WASHING OR DECONTAMINATION.
  - NOT USED.
  - UNLESS OTHERWISE NOTED, MOTOR OPERATED VALVES USE AC POWER.
  - THIS PUMP TYPE IS GEARED MOTOR PUMP.
  - THIS VALVE IS SETTING ON VERTICAL LINE.
  - ✕ S&W MEANS THAT THE MATERIALS BELONGS TO PUMP DESIGN SIDE.
  - QUALITY CONTROL CLASSIFICATION OF WASHING AND VALVE ARE DESIGNED TO SAME CLASS AS CONNECTING LINE, BUT THE DIFFERENT CLASS LINES CONNECTING QUALITY CONTROL CLASSIFICATION OF WASHING AND VALVE ARE DESIGNED SAME CLASS AS HIGH DRAIN CLASS.
  - THIS VALVE IS DESIGNED AS SEAL PROVISION TYPE.
  - THIS LINE IS DEGAIRATED AFTER PREPARATION TEST.
  - THE BRAND OF BY-PASS VALVE OF R-HEU DIB SHALL BE DIRECTED TO UPSTREAM.
  - NOT USED.
  - TEGOS INCLUDES TF FUNCTION.
  - NOT USED.
  - DESIGN CONDITIONS ARE FOLLOWS:
    - (A) FLUID WATER
    - (B) RADIOACTIVE CONCENTRATION  $\leq 340/CC$
    - (C) SCHEDULE INTERFACIE

REFERENCE DOCUMENTS

NO.	DESCRIPTION	REV. NO.
1	REACTOR WATER CLEANUP SYS PFD	031-1030
2	NUCLEAR BOILER SYSTEM P&ID	021-1010
3	NOT USED	
4	LOW CONDUCTIVITY WASTE SYS P&ID	017-1030
5	REACTOR WATER CLEANUP SYS I&D	031-0330
6	RESIDUAL HEAT REMOVAL SYS P&ID	011-1010
7	RFCU FILTER DEMINERALIZER P&ID	031-1010
8	LINE INSTRUMENT SYMBOL LIST	130-3030
9	HEAT EXCHANGER WATER SYS P&ID	021-1030
10	HEAT EXCHANGER WATER SYS P&ID	021-1030
11	SAMPLING SYSTEM P&ID	021-1010
12	CONTROL ROD DRIVE SYS P&ID	012-1010
13	VALVE BRAND LEAKOFF SYS P&ID	021-1010
14	LEAK DETECTION AND ISOLATION SYS I&D	031-1040
15	VALVE BRAND LEAK OFF P&ID	021-1010
16	FRESHWATER CONTROL SYS I&D	031-0330

Figure 5.4-12 REACTOR WATER CLEANUP SYSTEM P&ID, Sheet 1 of 2

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# ABWR Standard Plant

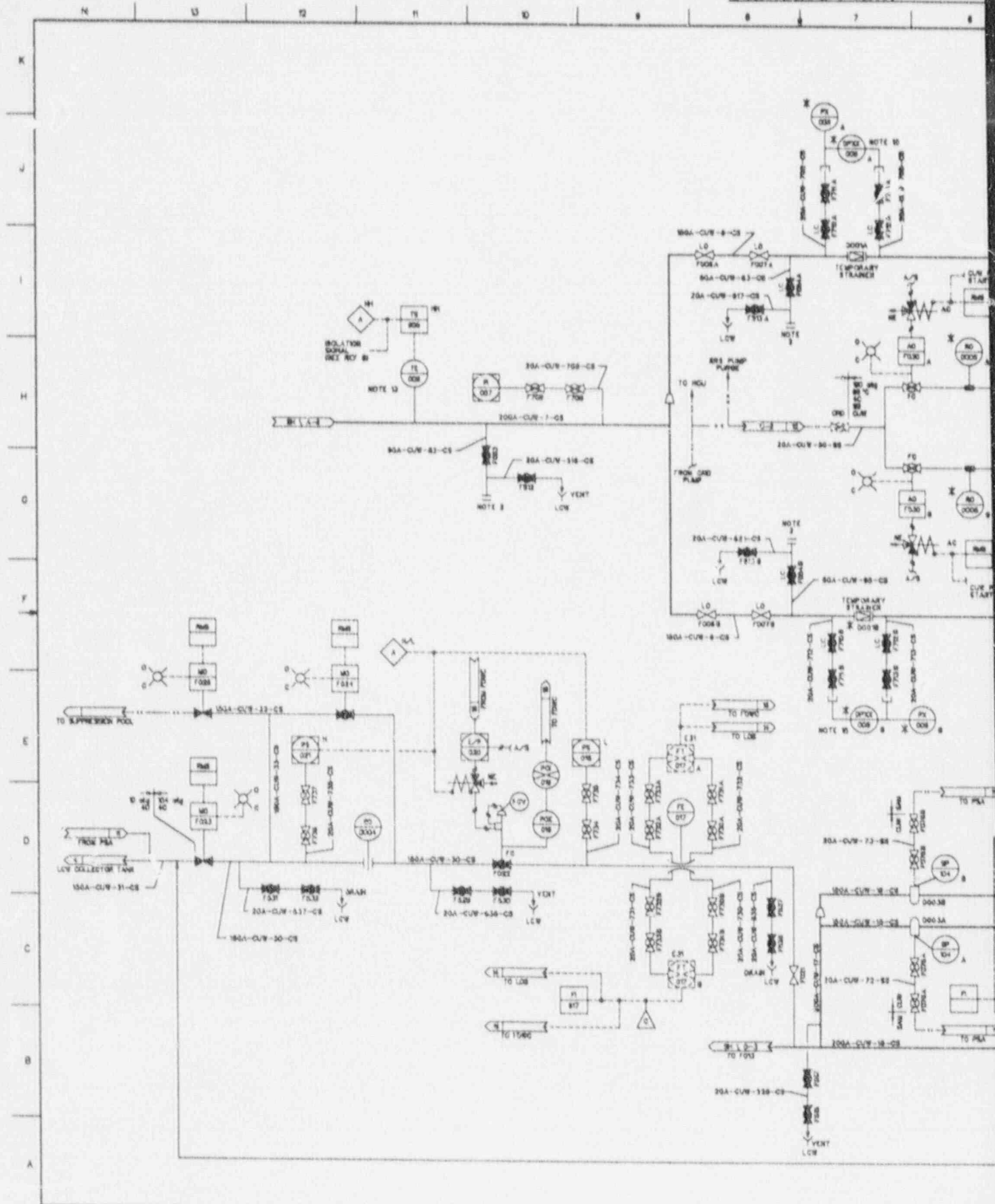
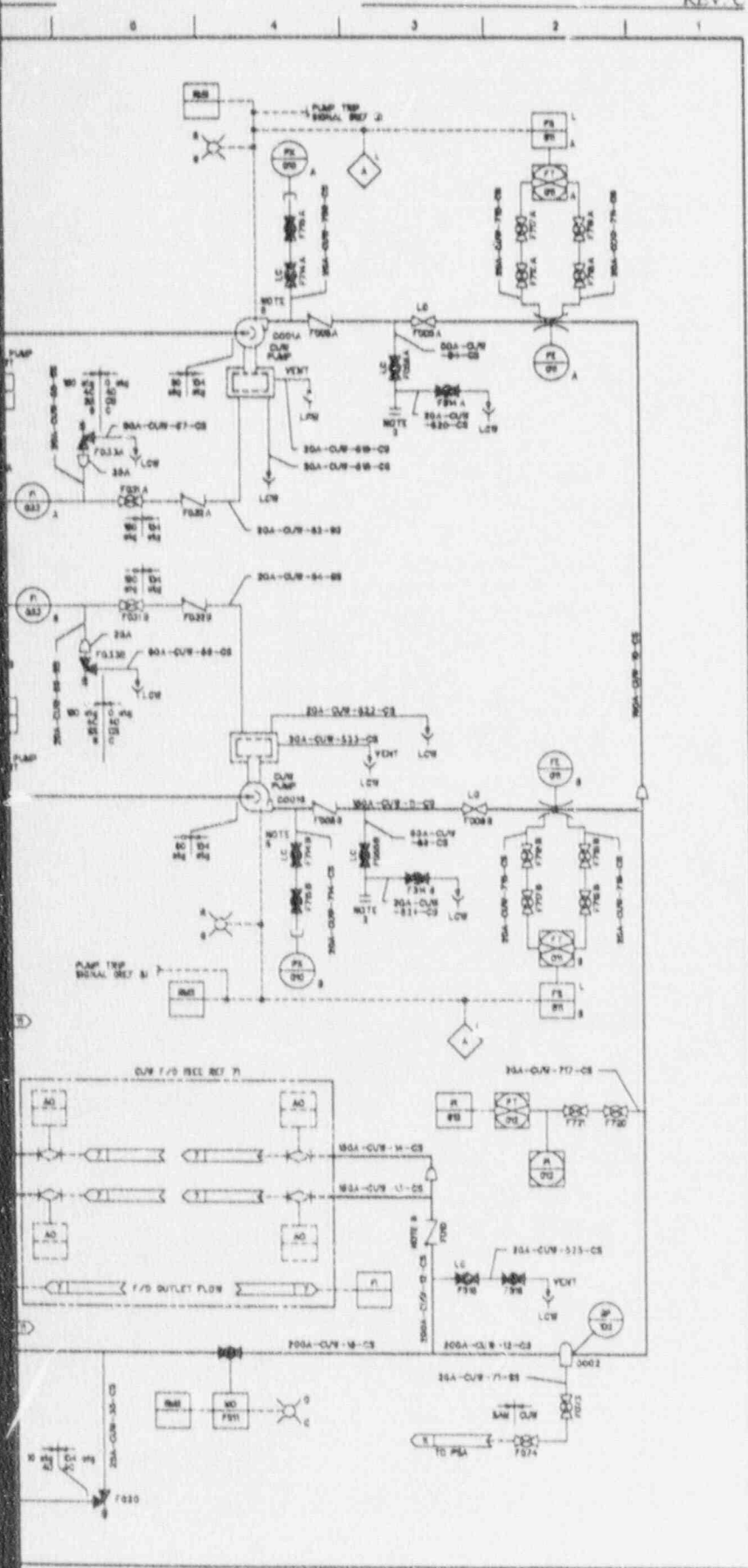


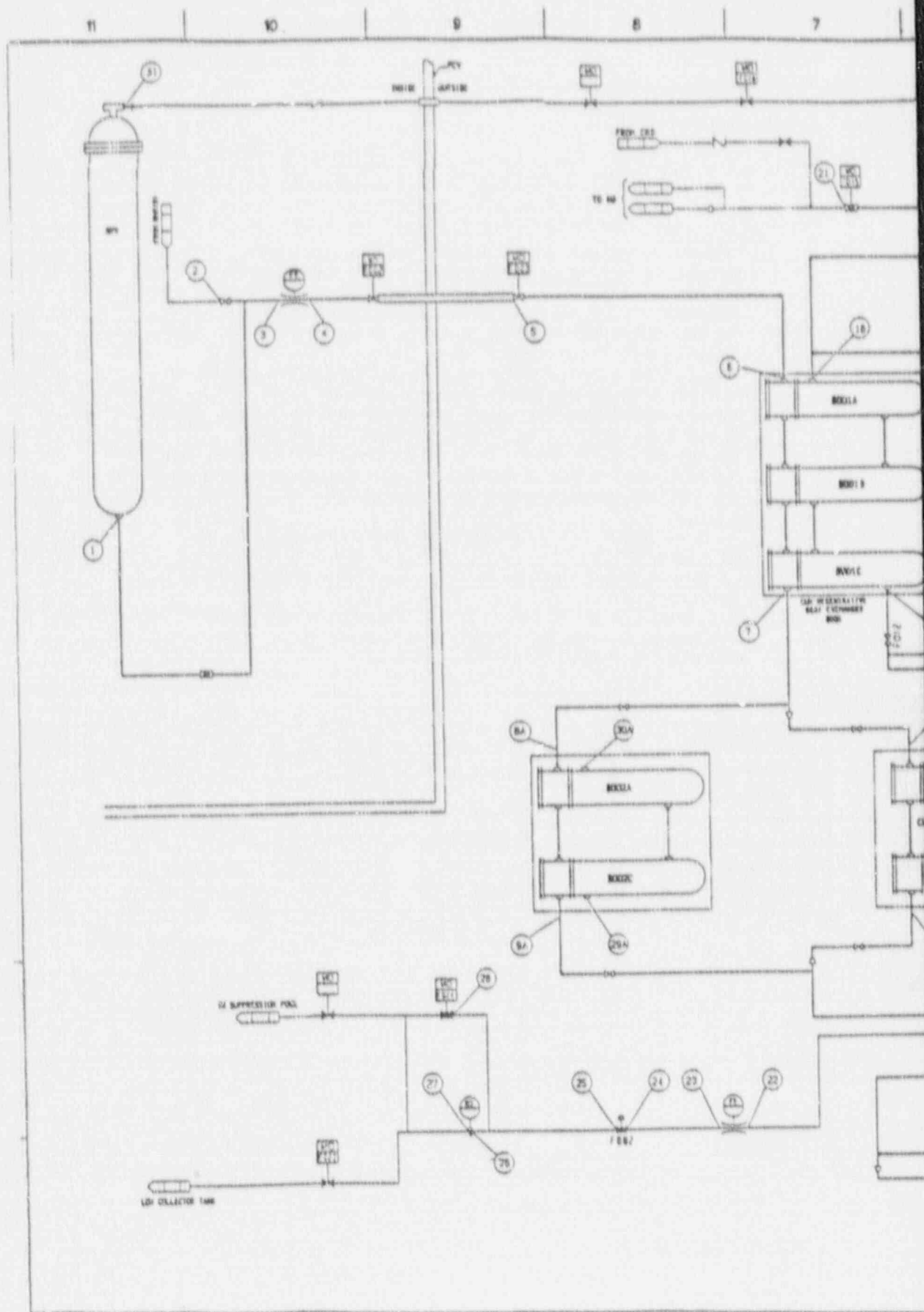
Figure 5.4-12 RE



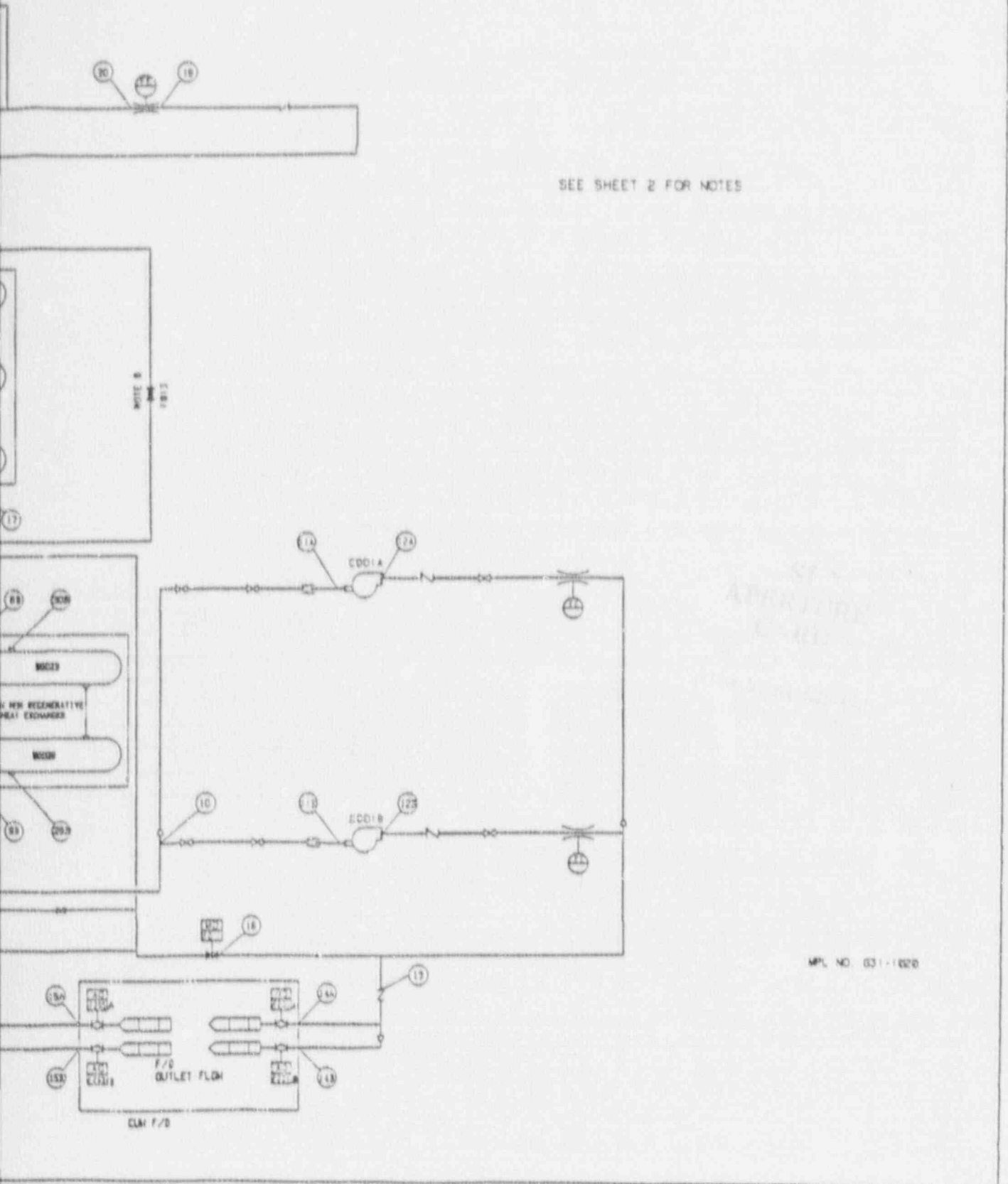
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Also Available  
Approved

REACTOR WATER CLEANUP SYSTEM P&ID, Sheet 2 of 2









SEE SHEET 2 FOR NOTES

ST  
APPROVAL  
L-01

MPL NO. 031-1020

Figure 5.4-13 REACTOR WATER CLEANUP SYSTEM PFD, Sheet 1 of 2

5                      4                      3                      2

GENERAL NOTES THE VALVE (FO24) AND

1. SHOWS THE PART WHERE THE FLUID DOES NOT FLOW.
2. SHOWS THE VALVE WHICH IS NOT NEEDED IN BASIC PLANNING OF THIS SYSTEM.
3. CUM PUMP, CUM NON-REGENERATIVE HEAT EXCHANGER AND CUM REGENERATIVE HEAT EXCHANGER SHALL BE INSTALLED TO HAVE ADEQUATE STATIC PRESSURE. THE MOST SEVERE OPERATING MODE IS OFF GAS OPERATING MODE AT START-UP OPERATION WHEN REACTOR PRESSURE IS 0.1 kg/cm<sup>2</sup>.
4. MODE A IS THE BASIC DESIGN CONDITION OF HEAT EXCHANGER (REGENERATIVE HEAT EXCHANGER AND NON-REGENERATIVE HEAT EXCHANGER).
5. DURING A STARTUP OPERATION F/D MAY BE BYPASSED WHEN F/D IS OUT OF SERVICE.
6. AT MODE B AND MODE F THE VALVE (FO22, FO23, FO24) SHALL BE OPEN AND THEN THE FLUID IS TRANSFERRED TO THE LOW CONDUCTIVITY COLLECTOR TANK.
7. THE TOTAL PRESSURE LOSS OF REGENERATIVE HEAT EXCHANGE (SHELL SIDE AND TUBE SIDE) AND NON-REGENERATIVE HEAT EXCHANGER (TUBE SIDE) IS UNDER 2.1 kg/cm<sup>2</sup>.
8. ALL OF THE SYSTEM FLOW IS SPRAYED FROM RPV SPRAY HEADER AT THE RPV SPRAY MODE.
9. THE TOTAL FLOW RATE FROM RHR AND FROM RPV BOTTOM HEAD DRAIN LINE IS 152.5 x 10<sup>3</sup> kg/h.
10. THIS BYPASS LINE MAY BE APPLIED WITH FO13 OPENED AND FO12 CLOSED CONDITION SO THAT THE DECAY HEAT IS REMOVED BY NON REGENERATIVE HEAT EXCHANGER DURING REFUELING OUTAGE IF REQUIRED.

CLOSING CONDITION

FO15	FO16	FO22	FO23	FO24	F201	F205
0	C	C	C	C	0	0
0	C	T	0	C	0	0
0	C	C	C	C	C	C
0	C	C	C	C	C	C
0	C	C	C	C	0	0
C	C	0	0	0	0	0
C	0	C	C	C	0	0

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APPROVED  
DATE  
  
APPROVED  
DATE

Figure 5.4-13 REACTOR WATER CLEANUP SYSTEM PFD, Sheet 2 of 2

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MODE C SHUT DOWN (REACTOR PWRG. 10.5% PWR) NOTES

DATE	TIME	TEMP (°C)	FLOW (M <sup>3</sup> /H)	REACTOR PWRG. (%)
1	2	3	4	5
6	7	8	9	10
11	12	13	14	15
16	17	18	19	20
21	22	23	24	25
26	27	28	29	30
31	32	33	34	35

MODE E AFTER RESTARTING (REACTOR PWRG. ATHERMATIC PWRG) NOTES

DATE	TIME	TEMP (°C)	FLOW (M <sup>3</sup> /H)	REACTOR PWRG. (%)
1	2	3	4	5
6	7	8	9	10
11	12	13	14	15
16	17	18	19	20
21	22	23	24	25
26	27	28	29	30
31	32	33	34	35

MODE E RESTARTING (REACTOR PWRG. ATHERMATIC PWRG)

DATE	TIME	TEMP (°C)	FLOW (M <sup>3</sup> /H)	REACTOR PWRG. (%)
1	2	3	4	5
6	7	8	9	10
11	12	13	14	15
16	17	18	19	20
21	22	23	24	25
26	27	28	29	30
31	32	33	34	35

MODE D HOT STANDBY OPERATION (AVAILABLE REACTOR PWRG. 77.5% PWR)

DATE	TIME	TEMP (°C)	FLOW (M <sup>3</sup> /H)	REACTOR PWRG. (%)
1	2	3	4	5
6	7	8	9	10
11	12	13	14	15
16	17	18	19	20
21	22	23	24	25
26	27	28	29	30
31	32	33	34	35

MODE C HOT STANDBY OPERATION (AVAILABLE REACTOR PWRG. 73.1% PWR)

DATE	TIME	TEMP (°C)	FLOW (M <sup>3</sup> /H)	REACTOR PWRG. (%)
1	2	3	4	5
6	7	8	9	10
11	12	13	14	15
16	17	18	19	20
21	22	23	24	25
26	27	28	29	30
31	32	33	34	35

MODE B START-UP OPERATION (REACTOR PWRG. 11.0% PWR) NOTES

DATE	TIME	TEMP (°C)	FLOW (M <sup>3</sup> /H)	REACTOR PWRG. (%)
1	2	3	4	5
6	7	8	9	10
11	12	13	14	15
16	17	18	19	20
21	22	23	24	25
26	27	28	29	30
31	32	33	34	35

MODE A NORMAL OPERATION (REACTOR PWRG. 73.1% PWR)

DATE	TIME	TEMP (°C)	FLOW (M <sup>3</sup> /H)	REACTOR PWRG. (%)
1	2	3	4	5
6	7	8	9	10
11	12	13	14	15
16	17	18	19	20
21	22	23	24	25
26	27	28	29	30
31	32	33	34	35

TABLE 1 VALVE OPENING

MODE	POD	POD	POD	POD	POD	POD	POD	POD	POD
A	0	0	0	0	0	0	0	0	0
B	0	0	0	0	0	0	0	0	0
C	0	0	0	0	0	0	0	0	0
D	0	0	0	0	0	0	0	0	0
E	0	0	0	0	0	0	0	0	0
F	0	0	0	0	0	0	0	0	0
G	0	0	0	0	0	0	0	0	0
C	0	0	0	0	0	0	0	0	0
C	0	0	0	0	0	0	0	0	0
C	0	0	0	0	0	0	0	0	0

0 : OPENING  
C : CLOSED  
T : THROTTLE