



UNITED STATES  
NUCLEAR REGULATORY COMMISSION  
WASHINGTON, D. C. 20555

TPS-1P

SAFETY EVALUATION BY THE OFFICE OF NUCLEAR REACTOR REGULATION

RELATED TO AMENDMENT NOS. 91 AND 81 TO

FACILITY OPERATING LICENSE NOS. NPF-10 AND NPF-15

SOUTHERN CALIFORNIA EDISON COMPANY

SAN DIEGO GAS AND ELECTRIC COMPANY

THE CITY OF RIVERSIDE, CALIFORNIA

THE CITY OF ANAHEIM, CALIFORNIA

SAN ONOFRE NUCLEAR GENERATING STATION, UNIT NOS. 2 AND 3

DOCKET NOS. 50-361 AND 50-362

1.0 INTRODUCTION

By letter dated November 8, 1990, Southern California Edison Company, et al. (SCE or the licensee) requested changes to the Technical Specifications (TS) for Facility Operating License Nos. NPF-10 and NPF-15 that authorize operation of San Onofre Nuclear Generating Station, Unit Nos. 2 and 3 in San Diego County, California. The licensee requested to revise TS 3/4.7.1.1, "Safety Valves," TS Table 3.7-1, "Steam Line Safety Valves Per Loop," TS Table 3.7-2, "Maximum Allowable Linear Power Level-High Trip Setpoint With Inoperable Steam Line Safety Valves During Operation With Both Steam Generators," and the corresponding Bases section.

2.0 EVALUATION

The licensee requested these amendments due to a change in the allowable value for Linear Power Level-High in TS Table 2.2-1, "Reactor Protective Instrumentation Trip Setpoint Limits," as granted in Amendment Nos. 88 and 78 on June 8, 1990 by the NRC. In this case, the aforementioned amendments were granted to account for an increase in the allowed tolerance for trip bistable functional testing for certain instrument surveillances. This change reduced the allowable value for linear power from 111.3% rated thermal power to 111.0% rated thermal power. This value affects the equation used to reduce the core operating power level in case of inoperable main steam line safety valves as found in the current Bases, which was not recognized at the time of the aforementioned amendments. This change would ensure consistency within the Technical Specifications. Additionally, the licensee requested these amendments to make editorial revisions, such as removing ambiguous statements, clarify design requirements, and correct typographical and numerical errors, which are primarily administrative in nature.

While the licensee states there has been no safety impact as a result of the lower allowable value on plant operation, and administrative controls are in place to ensure this, the Bases equation to lower reactor rated thermal power is affected such that the values for reduced reactor power with inoperable main steam line safety valves as found in TS Table 3.7-2 are reduced. The licensee states that this change does not affect the overpressure protection analysis as found in the Updated Final Safety Analysis Report, Appendix 5.2A. This analysis assumes that high pressurizer pressure causes a reactor trip and not high linear power level.

The licensee states that editorial revisions are necessary to revise the Bases section to clarify the safety valve's design requirements, particularly the requirement to limit secondary system pressure to less than 1210 psia during anticipated operational transients. Also, other editorial modifications to ensure consistent wording and correct numerical errors were evaluated.

The staff agrees with the licensee that the overpressure protection analysis is not affected by the new values for maximum allowable linear power level-high trip in TS Table 3.7-2. While this is a small reduction from the current values found in TS Table 3.7-2, it is appropriate to correct the values to ensure consistency within the Technical Specifications. Independent calculations confirm that the new (and lowered) values presented in the table are correct. Moreover, the design requirement, as specified in the modified Bases, to limit secondary system pressure to less than 1210 psia is appropriate. Administrative editorial modifications for the modified Technical Specifications are appropriate to maintain consistency and eliminate ambiguity.

Therefore, based upon the licensee's letter dated November 8, 1990, and the information presented herein, the proposed modifications to the Technical Specifications are acceptable.

### 3.0 ENVIRONMENTAL CONSIDERATION

The amendments involve changes with respect to the installation or use of a facility component located within the restricted area as defined in 10 CFR Part 20, or changes a surveillance requirement. The staff has determined that the amendments involve no significant increase in the amounts, and no significant change in the types, of any effluents that may be released offsite and that there is no significant increase in individual or cumulative occupational radiation exposure. The Commission has previously issued a proposed finding that the amendments involve no significant hazards consideration and there has been no public comment on such finding. Accordingly, the amendments meet the eligibility criteria for categorical exclusion set forth in 10 CFR 51.22(c)(9). Pursuant to 10 CFR 51.22(b) no environmental impact statement or environmental assessment need be prepared in connection with the issuance of these amendments.

### 4.0 CONCLUSION

We have concluded, based on the considerations discussed above, that (1) there is reasonable assurance that the health and safety of the public will not be endangered by operation in the proposed manner, and (2) such activities will be conducted in compliance with the Commission's regulations, and the issuance of

these amendments will not be inimical to the common defense and security or to the health and safety of the public.

Principal Contributor: Lawrence E. Kokajko

Dated: December 28, 1990