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December 26, 1990

PG&E Letter No. HBL-90-063



U.S. Nuclear Regulatory Commission
ATTN: Document Control Desk
Washington, D.C. 20555

Re: Docket No. 50-133, OL-DPR-7
Humboldt Bay Power Plant, Unit 3
10 CFR 50.59 Annual Report of Changes, Tests, and Experiments
January 1, 1989 - December 31, 1989

Gentlemen:

Pursuant to 10 CFR 50.59, enclosed is the Annual Report of Changes, Tests, and Experiments for the reporting interval of January 1, 1989 to December 31, 1989.

Licensees are allowed by 10 CFR 50.59 to make changes in the facility as described in the SAFSTOR Decommissioning Plan (SSDP), make changes in procedures as described in the SSDP, and conduct tests and experiments not described in the SSDP without prior NRC approval, provided that the changes, tests, or experiments do not involve changes in the Technical Specifications or unreviewed safety questions. Such changes are required to be reported on an annual basis by 10 CFR 50.59(b)(2). This report is intended to satisfy that requirement.

Proposed facility changes, procedure changes, tests, or experiments that are determined to involve unreviewed safety questions or changes to the Technical Specifications are submitted separately to the NRC for prior approval under the provisions of 10 CFR 50.90.

Changes in the Facility As Described in the SSDP. The enclosed annual report provides a brief description of the 10 CFR 50.59 facility design changes, including a summary of each safety evaluation. Each change was reviewed and accepted by the Plant Staff Review Committee (PSRC). Furthermore, each change has been completed and accepted by the plant staff.

None of the design changes involved an unreviewed safety question or a change to the HBPP Technical Specifications, as determined by the PSRC. The changes do not (a) increase the probability or consequences of an accident or malfunction of equipment important to safety as previously analyzed in the SSDP; (b) create the possibility of an accident or malfunction of equipment not previously analyzed in the SSDP; or (c) reduce the margin of safety as defined in the bases of the Technical Specifications.

Changes in Procedures As Described in the SSDP. The PSRC has reviewed all plant-specific administrative and technical procedure revisions and

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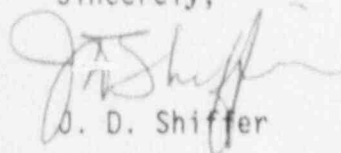
December 26, 1990

concluded that they do not represent unreviewed safety questions or changes to the HBPP Technical Specifications.

No General Office procedure changes resulted in deviations from the SAFSTOR Decommissioning PLAN (SSPD).

Tests and Experiments Not Described in the SSPD. There were no tests or experiments conducted that are not described in the SSPD and that impact the safety of operations.

Sincerely,



D. D. Shiffer

cc: A. P. Hodgdon
D. Martin
J. B. Martin
CPUC
Humboldt Distribution

Enclosure

5129S/0085K/RLK/1392

ENCLOSURE

10 CFR 50.59 ANNUAL REPORT OF CHANGES, TESTS, AND EXPERIMENTS

JANUARY 1, 1989 - DECEMBER 31, 1989

HUMBOLDT BAY UNIT 3
DOCKET NO. 50-133

Pacific Gas & Electric Company

ENCLOSURE

HUMBOLDT BAY POWER PLANT

ANNUAL REPORT OF DESIGN CHANGES FOR 1989

Listed below by number are all design changes made to Unit 3 in 1989, along with brief descriptions of the changes and summaries of the safety evaluations. More complete records of these design changes have been reviewed by the Plant Staff Review Committee and determined not to involve either unreviewed safety questions or changes to the plant Technical Specifications.

1. DCN HB3-O-SE-249 Sheriff's Office Remote Arming Switch

This modification is Security Safeguards Information, not suitable for discussion herein.

Safety Evaluation Summary

Not applicable

2. DCN HB3-O-SM-268 Install Radwaste Building Step-Off Pad

This modification consisted of installing a permanent, enclosed step-off pad. This enclosed step-off pad was constructed from a steel cargo shipping container that was converted to a semi-portable building, complete with doors at each end for ingress and egress. The step-off pad was placed into the roll-up door at the northwest corner of the Radwaste Enclosure and flashing was installed in order to seal the penetration. This seal was necessary to ensure that the flow of ventilation air would be into the Radwaste Enclosure and that no unmonitored releases could occur. This step-off pad replaced a temporary, wooden structure that served as a step-off pad.

Safety Evaluation Summary

The installation of a non-combustible structure for the Radwaste Enclosure Step-off Pad reduced the fire loading within the Radiological Control Area, thereby increasing both personnel and radiological safety. The step-off pad was also relocated to an area of lower radiation background, which improved contamination control by providing the capability for more effective personnel exit contamination surveys.