



Northern States Power Company

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December 26, 1990

10 CFR Part 50  
Section 50.90

U S Nuclear Regulatory Commission  
Attn: Document Control Desk  
Washington, DC 20555

PRAIRIE ISLAND NUCLEAR GENERATING PLANT  
Docket Nos. 50-282 License Nos. DPR-42  
50-306 DPR-60

License Amendment Request Dated December 26, 1990  
Feedwater Isolation Limitations

Attached is a request for a change to the Technical Specifications, Appendix A of the Operating Licenses, for the Prairie Island Nuclear Generating Plant. This request is submitted in accordance with the provisions of 10 CFR Part 50, Section 50.90.

Generic Letter 89-19, "Request for Action Related to Resolution of Unresolved Safety Issue A-47", recommended that Technical Specifications for all Westinghouse plants include provisions to periodically verify the operability of the main feedwater overflow protection and ensure that the automatic overflow protection is operable during reactor power operation. Our response to Generic Letter 89-19, dated March 15, 1990, committed to submit a License Amendment Request to revise Technical Specification Table TS.3.5-4 to include limiting conditions for operations for feedwater isolation.

The NRC closeout of our response to Generic Letter 89-19, transmitted by letter dated July 17, 1990, requested that Technical Specification Surveillance Section 4.0 also be changed to require surveillance of both low and high steam generator water level instrumentation.

Prairie Island Technical Specification Tables TS.3.5-4 and TS.4.1-1 are being revised to incorporate the feedwater isolation specifications requested by Generic Letter 89-19 and the subsequent NRC closeout.

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Prairie Island Technical Specification Table TS.4.1-1 is also being corrected to remove surveillances related to the low steam generator water level coincident with steam/feedwater mismatch reactor trip which was removed from the Technical Specifications by License Amendments Nos. 87 and 80.

Exhibit A contains a description of the proposed change, the reasons for requesting the changes and the supporting safety evaluation/significant hazards determination. Exhibit B contains current Prairie Island Technical Specification pages marked up to show the proposed changes. Exhibit C contains the revised Technical Specification pages.

Please contact us if you have any questions related to this License Amendment Request.



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Nuclear Support Services

c: Regional Administrator-III, NRC  
NRR Project Manager, NRC  
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Attn: J W Ferman  
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Attachments:

Affidavit

Exhibit A - Evaluation of Proposed Changes to the Technical Specifications

Exhibit B - Proposed Changes Marked Up on Existing Technical Specification Pages

Exhibit C - Revised Technical Specification Pages