

# UNITED STATES NUCLEAR REGULATORY COMMISSION WASHINGTON, D. C. 20555

#### GENERAL ELECTRIC COMPANY

DOCKET NO. 50-70

#### GENERAL ELECTRIC TEST REACTOR

#### AMENDMENT TO FACILITY LICENSE

Amendment No. 9 License No. TR-1

- 1. The Nuclear Regulatory Commission (the Commission) has found that:
  - A. The application for amendment by General Electric Company (the licensee), dated February 4, 1982, as supplemented by letters dated March 22, 1982 and April 15, 1982, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations as set forth in 10 CFR Chapter I;
  - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
  - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
  - D. The issuance of this amendment will not be inimical to the common defense and security, or to the health and safety of the public;
  - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations, and all applicable requirements have been satisfied; and
  - F. Publication of notice of this amendment is not required since it does not involve a significant hazards consideration nor amendment of a license of the type described in 10 CFR Section 2.106(a)(2).

- Accordingly, the license is amended by changes to the Technical Specifications, as indicated in the attachment to this license amendment, and paragraph 3.B of Facility License No. TR-1 is hereby amended to read as follows:
  - B. Technical Specifications

The Technical Specifications contained in Appendix A, as revised through Amendment No. 9 are, hereby, incorporated in the license. The licensee shall operate the facility in accordance with the Technical Specifications.

3. This license amendment is effective as of the date of issuance.

FOR THE NUCLEAR REGULATORY COMMISSION

Cecil O. Olromas

Cecil O. Thomas, Acting Chief Standardization & Special Projects Branch Division of Licensing

Attachment: Changes to the Technical Specifications

Date of Issuance: OCT 1 5 1982

# ATTACHMENT TO LICENSE AMENDMENT NO. 9 FACILITY LICENSE TR-1 DOCKET NO. 50-70

Revise Appendix A as follows:

Remove Page	Add Pages
15b	15b
	19
	20

Changes on the revised pages are identified by marginal lines.

- 9.4 Action to be Taken in the Event of an Abnormal Occurrence

  Upon the occurrence of abnormal reactor behavior, including its

  controls and safety systems, action shall be taken promptly to

  assure the safety of the plant, to determine the cause of the abnormal behavior, and to take appropriate corrective measures.
  - 9.5 Any additions, modifications or maintenance to the containment building or its isolation system, the reactor, the primary coolant system, auxiliary fluid systems connected thereto, or the reactor safety system shall be designed, fabricated, and tested in accordance with the original specifications or specifications subsequently reviewed in accordance with Technical Specification 9.2.3 and approved by the facility manager.
- The GETR Water Processing Tank Farm, Equipment Building, Cask Storage Pad east of the equipment airlock, and the Radioactive Waste Temporary Storage Area located in the northwest quadrant of the GETR perimeter fenced area are exempt from the alarm device requirements of 10 CFR 20.203(c)(2), provided that each of these areas is bounded by a fence or by a distinctively colored rope or chain constituting a barrier, and posted in accordance with 10 CFR 20.203(c)(1). This barrier shall be placed where dose rates do not exceed 100 mr/hr, and the radiation level of any readily accessible surface within this inner area does not exceed 1000 mr/hr at any time.

## 11.0 System Tests and Calibrations

- 11.1 The following tests and calibrations need not be performed during the cold shutdown period begun in October, 1977.
  - a. Semiannual operability check of the primary system siphon break valves per Technical Specification 4.4.
  - Annual calibration of the pool instrumentation per Technical Specification 4.5.
  - c. Semiannual control rod drop time measurements per Technical Specification 6.2.
  - d. Annual calibration of all instruments listed in Table II per Technical Specification 6.8.
  - e. Annual operability test of the poison injection system per Technical Specification 6.9.
  - f. Annual operability test of the pool emergency recirculation system per Technical Specification 7.1.
  - g. Semiannual load test of the emergency generator per Technical Specification 7.2.
  - h. Annual leak test of the experiment exhaust system holdup tanks per Technical Specification 7.4.
  - Bimonthly operability check of the experiment exhaust system radiation monitors per Technical Specification 7.5.

### 11.0 System Tests and Calibrations - continued

- 11.2 Following the cold shutdown period in which tests were postponed in accordance with Technical Specification 11.1, all postponed tests and calibrations shall be performed prior to returning fuel or experiments to the reactor or pool (except for the control rod drop time tests which shall be performed prior to startup), as well as any additional tests and calibrations deemed necessary by General Electric or the Commission.
- 11.3 After startup following the cold shutdown period, the original test and calibration frequencies and schedules shall be resumed.
- 11.4 Following the cold shutdown period in which tests were postponed in accordance with Technical Specification 11.1,

  General Electric Company will submit a reactor restart program to the Nuclear Regulatory Commission 90 days in advance of the planned restart. No fuel (new or irradiated) will be loaded into the reactor until authorized by the Commission.