



Northern States Power Company

414 Nicollet Mall
Minneapolis, Minnesota 55401-1927
Telephone (612) 330-5500

December 28, 1990

10 CFR Part 50
Section 50.73

U S Nuclear Regulatory Commission
Attn: Document Control Desk
Washington, DC 20555

PRAIRIE ISLAND NUCLEAR GENERATING PLANT
Docket Nos. 50-282 License Nos. DPR-42
50-306 DPR-60

Discovery That Certain Valves Should Be Subject to ASME
Section XI Testing Found through Design Basis Reconstitution

The Licensee Event Report for this occurrence is attached.

Please contact us if you require additional information related to this event.

for *Monica M. Vike*

Thomas M Parker
Manager
Nuclear Support Services

c: Regional Administrator - Region III, NRC
NRR Project Manager, NRC
Senior Resident Inspector, NRC
MPCA
Attn: Dr J W Ferman

Attachment

1022

LICENSEE EVENT REPORT (LER)

ESTIMATED BURDEN PER RESPONSE TO COMPLY WITH THIS INFORMATION COLLECTION REQUEST: 500 HRS. FORWARD COMMENTS REGARDING BURDEN ESTIMATES TO THE RECORDS AND REPORTS MANAGEMENT BRANCH (F-530), U.S. NUCLEAR REGULATORY COMMISSION WASHINGTON, DC 20555, AND TO THE PAPERWORK REDUCTION PROJECT (3150-0104), OFFICE OF MANAGEMENT AND BUDGET WASHINGTON, DC 20503.

FACILITY NAME (1) **Prairie Island Nuclear Generating Plant Unit 1** DOCKET NUMBER (2) **0 5 0 0 0 2 8 2** PAGE (3) **1 OF 0 2**

TITLE (4) **Discovery That Certain Valves Should Be Subject to ASME Section XI Testing Found through Design Basis Reconstitution**

EVENT DATE (5)			LER NUMBER (6)			REPORT DATE (7)			OTHER FACILITIES INVOLVED (8)	
MONTH	JAY	YEAR	YEAR	SEQUENTIAL NUMBER	REVISION NUMBER	MONTH	DAY	YEAR	FACILITY NAMES	DOCKET NUMBER(S)
11	30	90	90	018	00	12	28	90	Prairie Island Unit 2	0 5 0 0 0 3 0 6
										0 5 0 0 0 1 1

OPERATING MODE (9) **N**

POWER LEVEL (10) **1 0 0**

THIS REPORT IS SUBMITTED PURSUANT TO THE REQUIREMENTS OF 10 CFR § (Check one or more of the following) (11)

<input type="checkbox"/> 20.402(b)	<input type="checkbox"/> 20.406(a)	<input type="checkbox"/> 50.73(a)(2)(iv)	<input type="checkbox"/> 73.71(a)
<input type="checkbox"/> 20.406(a)(1)(iii)	<input type="checkbox"/> 50.76(a)(1)	<input type="checkbox"/> 50.73(a)(2)(v)	<input type="checkbox"/> 73.71(a)
<input type="checkbox"/> 20.406(a)(1)(ii)	<input type="checkbox"/> 50.76(a)(2)	<input type="checkbox"/> 50.73(a)(2)(iv)	<input type="checkbox"/> OTHER (Specify in Abstract below and in Text, NRC Form 366A)
<input type="checkbox"/> 20.406(a)(1)(iii)	<input checked="" type="checkbox"/> 50.75(a)(2)(i)	<input type="checkbox"/> 50.73(a)(2)(v)(i)(A)	
<input type="checkbox"/> 20.406(a)(1)(iv)	<input type="checkbox"/> 50.73(a)(2)(iv)	<input type="checkbox"/> 50.73(a)(2)(v)(ii)(B)	
<input type="checkbox"/> 20.406(a)(1)(v)	<input type="checkbox"/> 50.73(a)(2)(ii)	<input type="checkbox"/> 50.73(a)(2)(i)	

LICENSEE CONTACT FOR THIS LER (12)

NAME **Arne A Hunstad** TELEPHONE NUMBER **6 1 2 3 8 8 - 1 1 2 1**

COMPLETE ONE LINE FOR EACH COMPONENT FAILURE DESCRIBED IN THIS REPORT (13)

CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NRC	CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NRC

SUPPLEMENTAL REPORT EXPECTED (14) YES (If yes, complete EXPECTED SUBMISSION DATE) NO

EXPECTED SUBMISSION DATE (15) **0 2 0 4 9 1**

ABSTRACT (Limit to 1400 words, i.e., approximately fifteen single-space typewritten lines) (16)

On November 30, 1990, both units were at 100% power. A review of the design bases for Cooling Water System revealed that certain cooling water and chilled water valves required to mitigate the consequences of an accident are not included in the Prairie Island ASME Section XI inservice testing program. The valves in question were cycled during motor operated valve periodic maintenance and surveillance testing, but this testing did not satisfy the testing requirements of Section XI; that is, valve timing was not performed or trended, and the test was not done often enough. However, the testing is considered satisfactory to verify that the valves had been operable.

The investigation of this event is not yet complete because conflicting information exists regarding the safety functions of these valves. Resolution of those conflicts is expected shortly, at which time a supplemental report will be submitted.

These valves have all subsequently been cycled to obtain baseline data for their inclusion in the ASME Section XI inservice testing program. No abnormal parameters were observed. Additional corrective actions will be discussed in the supplemental report.

LICENSEE EVENT REPORT (LER)
TEXT CONTINUATIONESTIMATED BURDEN PER RESPONSE TO COMPLY WITH THIS
INFORMATION COLLECTION REQUEST 500 HRS. FORWARD
COMMENTS REGARDING BURDEN ESTIMATE TO THE RECORDS
AND REPORTS MANAGEMENT BRANCH (P-530), U.S. NUCLEAR
REGULATORY COMMISSION, WASHINGTON, DC 20555, AND TO
THE PAPERWORK REDUCTION PROJECT (3150-0104), OFFICE
OF MANAGEMENT AND BUDGET, WASHINGTON, DC 20503.

FACILITY NAME (1)

DOCKET NUMBER (2)

LER NUMBER (6)

PAGE (3)

Prairie Island Unit 1

0 5 0 0 0 2 8 2

YEAR SEQUENTIAL REVISION

NUMBER NUMBER NUMBER

9 0 -- 0 1 8 -- 0 0

0 2 OF 0 2

TEXT (IF MORE SPACE IS REQUIRED, USE ADDITIONAL NRC Form 366A (1) (17))

DESCRIPTION

On November 30, 1990, both units were at 100% power. A review of the design bases for Cooling Water System revealed that certain cooling water and chilled water valves required to mitigate the consequences of an accident are not included in the Prairie Island ASME Section XI inservice testing program.

ASME Section XI requires periodic exercising of all valves required to mitigate the consequences of an accident. The valves in question were cycled during motor operated valve periodic maintenance and surveillance testing, but this testing did not satisfy the testing requirements of Section XI; that is, valve timing was not performed or trended, and the test was not done often enough. However, the testing is considered satisfactory to verify that the valves had been operable.

The investigation of this event is not yet complete because conflicting information exists regarding the safety functions of these valves. Resolution of those conflicts is expected shortly, at which time a supplemental report will be submitted.

CAUSE OF THE EVENT

Cause of the event will be discussed in the supplemental report.

ANALYSIS OF THE EVENT

This event is reportable pursuant to 10CFR50.73(a)(2)(i)(B) since Technical Specification 4.2.A.2 was violated.

CORRECTIVE ACTION

These valves have all subsequently been cycled to obtain baseline data for their inclusion in the ASME Section XI inservice testing program. No abnormal parameters were observed.

Additional corrective actions will be discussed in the supplemental report.

FAILED COMPONENT IDENTIFICATION

None.

PREVIOUS SIMILAR EVENTS

There have been no previous similar events reported at Prairie Island.