UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

COMMONWEALTH EDISON COMPANY

DOCKET NO. 50-373

LASALLE COUNTY STATION, UNIT 1

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 76
License No. NPF-11

1. The Nuclear Regulatory Commission (the Commission) has found that:

   A. The application for amendment filed by the Commonwealth Edison Company (the licensee), dated September 21, 1990, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's regulations set forth in 10 CFR Chapter I;

   B. The facility will operate in conformity with the application, the provisions of the Act, and the regulations of the Commission;

   C. There is reasonable assurance: (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations set forth in 10 CFR Chapter I;

   D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and

   E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.

2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the enclosure to this license amendment and paragraph 2.C.(2) of the Facility Operating License No. NPF-11 is hereby amended to read as follows:

(2) Technical Specifications and Environmental Protection Plan

The Technical Specifications contained in Appendix A, as revised through Amendment No. 76 , and the Environmental Protection Plan contained in Appendix B, are hereby incorporated in the license. The licensee shall operate the facility in accordance with the Technical Specifications and the Environmental Protection Plan.

3. This amendment is effective upon date of issuance.

FOR THE NUCLEAR REGULATORY COMMISSION

Richard J. Barrett, Director
Project Directorate III-2
Division of Reactor Projects III/IV/V
Office of Nuclear Reactor Regulation

Attachment:
Changes to the Technical
Specifications

Date of Issuance: December 18, 1990

ATTACHMENT TO LICENSE AMENDMENT NO. 76

FACILITY OPERATING LICENSE NO. NPF-11

DOCKET NO. 50-373

Replace the following pages of the Appendix "A" Technical Specifications with the enclosed pages. The revised pages are identified by amendment number and contain a vertical line indicating the area of change.

REMOVE	INSERT
3/4 8-25	3/4 8-25
---	3/4 8-25a
B 3/4 8-3	B 3/4 8-3

TABLE 3.8.3.2-1 (Continued)

DEVICE NUMBER AND LOCATION	SYSTEM/COMPONENT POWERED
d) MCC 134X-1 (Compt. B4)	NB/MOV 1B21-F002
e) MCC 136Y-1 (Compt. B-2) (Normal)	RH/MOV 1E12-F009
f) MCC 136Y-2 (Compt. E4)	RI/MOV 1E51-F063
g) MCC 135Y-1 (Compt. A1)	RR/MOV 1B33-F023A
h) MCC 135Y-1 (Compt. A4)	RR/MOV 1B33-F067A
i) MCC 133-1 (Compt. C2)	RT/MOV 1G33-F102
j) MCC-133-1 (Compt. E1)	NB/MOV 1B21-F005
k) MCC-136Y-2 (Compt. B1)	NB/MOV 1B21-F016
l) MCC 136Y-2 (Compt. E1)	RH/MOV 1E12-F099A
m) MCC 136Y-1 (Compt. E4)	RT/MOV 1G33-F001
n) MCC 136Y-2 (Compt. A5)	WR/MOV 1WR180
o) MCC 136Y-2 (Compt. D6)	RH/MOV 1E12-F099B
p) MCC 136Y-1 (Compt. H5)	VP/MOV 1VP113B
q) MCC 136Y-1 (Compt. H4)	VP/MOV 1VP114A
r) MCC 136Y-1 (Compt. H3)	VP/MOV 1VP113A
s) MCC 136Y-1 (Compt. H6)	VP/MOV 1VP114B
t) MCC 136Y-2 (Compt. A4)	WR/MOV 1WR179
u) MCC 135Y-1 (Compt. D3)	RT/MOV 1G33-F101
v) MCC 135Y-1 (Compt. D4)	RT/MOV 1G33-F100
w) MCC 133-1 (Compt. C3)	RT/MOV 1G33-F106
x) MCC 136Y-2 (Compt. D5)	RI/MOV 1E51-F076
y) MCC 135X-1 (Compt. C2/C3) (Emerg)	RH/MOV 1E12-F009
z) MCC 133-2 (Compt. AC1)	VP/Drywell Cooler 1VP15SA
aa) MCC 133-2 (Compt. AB1)	VP/Drywell Cooler 1VP15SE

TABLE 3.8.3.2-1 (continued)

DEVICE NUMBER AND LOCATION	SYSTEM/COMPONENT POWERED
ab) MCC 133-2 (Compt. AB2)	VP/Drywell Cooler 1VP15SD
ac) MCC 134X-2 (Compt. H1)	VP/Drywell Cooler 1VP15SB
ad) MCC 134X-2 (Compt. H2)	VP/Drywell Cooler 1VP15SC
ae) MCC 134X-2 (Compt. J1)	VP/Drywell Cooler 1VP15SF
2. Type K-M Cat # NZ M12V-630/ZM12AV	
a) MCC 135X-2 (Compt. E4)	VP/Pri. Cont. Vent Supply Fan 1A Backup
b) MCC 136X-2 (Compt. G4)	VP/Pri. Cont. Vent Supply Fan 1B Backup

A.C. SOURCES AND ONSITE POWER DISTRIBUTION SYSTEMS (Continued)

specific gravity, ensures that the decrease in rating will be less than the safety margin provided in sizing; (3) the allowable value for an individual cell's specific gravity ensures that an individual cell's specific gravity will not be more than 0.040 below the manufacturer's full charge specific gravity and that the overall capability of the battery will be maintained within an acceptable limit; and (4) the allowable value for an individual cell's float voltage, greater than 2.07 volts, ensures the battery's capability to perform its design function.

The battery load profile and battery charger specifications will be maintained and are located in Chapter 8, "Electrical Power", section of the Updated Final Safety Analysis Report, UFSAR.

3/4.8.3 ELECTRICAL EQUIPMENT PROTECTIVE DEVICES

Primary containment medium and high voltage (6.9 kV, 4.16 kV and 480 volt) electrical penetrations and penetration conductors are protected by either de-energizing circuits not required during reactor operation or demonstrating the OPERABILITY of primary and backup overcurrent protection circuit breakers by periodic surveillance.

The surveillance requirements applicable to lower voltage circuit breakers and fuses provides assurance of breaker and fuse reliability by testing at least one representative sample of each manufacturers brand of circuit breaker and/or fuse. Each manufacturer's molded case and metal case circuit breakers and/or fuses are grouped into representative samples which are then tested on a rotating basis to ensure that all breakers and/or fuses are tested. If a wide variety exists within any manufacturer's brand of molded case circuit breakers and/or fuses, it is necessary to divide that manufacturer's breakers and/or fuses into groups and treat each group as a separate type of breaker or fuses for surveillance purposes.

The bypassing of the motor operated valves thermal overload protection continuously or during accident conditions by integral bypass devices ensures that the thermal overload protection will not prevent safety related valves from performing their function. The Surveillance Requirements for demonstrating the bypassing of the thermal overload protection continuously and during accident conditions are in accordance with Regulatory Guide 1.106 "Thermal Overload Protection for Electric Motors on Motor Operated Valves", Revision 1, March 1977.

UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

COMMONWEALTH EDISON COMPANY

DOCKET NO. 50-374

LASALLE COUNTY STATION, UNIT 2

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 60
License No. NPF-18

1. The Nuclear Regulatory Commission (the Commission) has found that:

   A. The application for amendment filed by the Commonwealth Edison Company (the licensee), dated September 21, 1990, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's regulations set forth in 10 CFR Chapter I;

   B. The facility will operate in conformity with the application, the provisions of the Act, and the regulations of the Commission;

   C. There is reasonable assurance: (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations set forth in 10 CFR Chapter I;

   D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and

   E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.

2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the enclosure to this license amendment and paragraph 2.C.(2) of the Facility Operating License No. NPF-18 is hereby amended to read as follows:

(2) Technical Specifications and Environmental Protection Plan

The Technical Specifications contained in Appendix A, as revised through Amendment No. 60, and the Environmental Protection Plan contained in Appendix B, are hereby incorporated in the license. The licensee shall operate the facility in accordance with the Technical Specifications and the Environmental Protection Plan.

3. This amendment is effective upon date of issuance.

FOR THE NUCLEAR REGULATORY COMMISSION

Richard J. Barrett, Director
Project Directorate III-2
Division of Reactor Projects III/IV/V
Office of Nuclear Reactor Regulation

Attachment:
Changes to the Technical
Specifications

Date of Issuance: December 18, 1990

ATTACHMENT TO LICENSE AMENDMENT NO. 60

FACILITY OPERATING LICENSE NO. NPF-18

DOCKET NO. 50-374

Replace the following pages of the Appendix "A" Technical Specifications with the enclosed pages. The revised pages are identified by amendment number and contain a vertical line indicating the area of change.

REMOVE	INSERT
3/4 8-25	3/4 8-25
---	3/4 8-25a
B 3/4 8-3	B 3/4 8-3

TABLE 3.8.3.2-1 (Continued)

DEVICE NUMBER AND LOCATION	SYSTEM/COMPONENT POWERED
d) MCC 234X-1 (Compt. B4)	NB/MOV 2B21-F002
e) MCC 236Y-1 (Compt. B2 (Normal)	RH/MOV 2E12-F009
f) MCC 236Y-2 (Compt. E4)	RI/MOV 2E51-F063
g) MCC 235Y-1 (Compt. A1)	RR/MOV 2B33-F023A
h) MCC 235Y-1 (Compt. A4)	RR/MOV 2B33-F067A
i) MCC 233-1 (Compt. C2)	RT/MOV 2G33-F102
j) MCC-233-1 (Compt. E1)	NB/MOV 2B21-F005
k) MCC-236Y-2 (Compt. B1)	NB/MOV 2B21-F016
l) MCC 236Y-2 (Compt. E1)	RH/MOV 2E12-F099A
m) MCC 236Y-1 (Compt. E4)	RT/MOV 2G33-F001
n) MCC 236Y-2 (Compt. A5)	WR/MOV 2WR180
o) MCC 236Y-2 (Compt. D6)	RH/MOV 2E12-F099B
p) MCC 236Y-1 (Compt. H5)	VP/MOV 2VP113B
q) MCC 236Y-1 (Compt. H4)	VP/MOV 2VP114A
r) MCC 236Y-1 (Compt. H3)	VP/MOV 2VP113A
s) MCC 236Y-1 (Compt. H6)	VP/MOV 2VP114B
t) MCC 236Y-2 (Compt. A4)	WR/MOV 2WR179
u) MCC 235Y-1 (Compt. D3)	RT/MOV 2G33-F101
v) MCC 235Y-1 (Compt. D4)	RT/MOV 2G33-F100
w) MCC 233-1 (Compt. C3)	RT/MOV 2G33-F106
x) MCC 236Y-2 (Compt. D5)	RI/MOV 2E51-F076
y) MCC 235X-1 (Compt. C2/C3) (Emerg)	RH/MOV 2E12-F009
z) MCC 233-2 (Compt. AC1)	VP/Drywell Cooler 2VP15SA
aa) MCC 233-2 (Compt. AB1)	VP/Drywell Cooler 2VP15SE

TABLE 3.8.3.2-1 (continued)

DEVICE NUMBER AND LOCATION	SYSTEM/COMPONENT POWERED
ab) MCC 233-2 (Compt. AB2)	VP/Drywell Cooler 2VP15SD
ac) MCC 234X-2 (Compt. H1)	VP/Drywell Cooler 2VP15SB
ad) MCC 234X-2 (Compt. H2)	VP/Drywell Cooler 2VP15SC
ae) MCC 234X-2 (Compt. J1)	VP/Drywell Cooler 2VP15SF
2. Backup breakers are located in the front of the respective MCC.	
a) MCC 235X-2 (Compt. AA4)	VP/Pri. Cont. Vent Supply Fan 2A Backup
b) MCC 236X-2 (Compt. AA4)	VP/Pri. Cont. Vent Supply Fan 2B Backup

A.C. SOURCES AND ONSITE POWER DISTRIBUTION SYSTEMS (Continued)

specific gravity, ensures that the decrease in rating will be less than the safety margin provided in sizing; (3) the allowable value for an individual cell's specific gravity ensures that an individual cell's specific gravity will not be more than 0.040 below the manufacturer's full charge specific gravity and that the overall capability of the battery will be maintained within an acceptable limit; and (4) the allowable value for an individual cell's float voltage, greater than 2.07 volts, ensures the battery's capability to perform its design function.

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The bypassing of the motor operated valves thermal overload protection continuously or during accident conditions by integral bypass devices ensures that the thermal overload protection will not prevent safety related valves from performing their function. The Surveillance Requirements for demonstrating the bypassing of the thermal overload protection continuously and during accident conditions are in accordance with Regulatory Guide 1.106 "Thermal Overload Protection for Electric Motors on Motor Operated Valves", Revision 1, March 1977.