NAC FOR	M 366	
(7.77)		

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U. S. NUCLEAR REGULATORY COMMISSION

(7.77)	LICENSEE EVENT REPORT
	CONTROL BLOCK
	P A T M I I 1 I
CON'T	REPORT X 6 0 5 0 0 0 2 8 9 0 0 8 3 1 8 2 8 1 0 2 8 8 2 9 SOURCE 50 61 DOCKET NUMBER 58 69 EVENT DATE 74 75 REPORT DATE 80
1.1	EVENT DESCRIPTION AND PROBABLE CONSEQUENCES (10)
0 2	During long term cold shutdown, oron was notified by write 2000 that our
03	which had been shipped to them for refurbishment, was severely corroded. Some
04	question exists if the valve could have performed its intended function if
0 5	called upon in its present condition. A spare valve is installed at TMI-1 now
06	and has functioned. Therefore, public health and safety were unaffected. This
07	is not reportable per specific Technical Specification requirements. Information
	L being provided as a special report.
1019	CODE CODE SUBCODE COMPONENT CODE SUBCODE SUBCO
7 8	9 10 11 12 13 18 19 20 9 10 SEQUENTIAL OCCURRENCE REPORT REVISION CODE TYPE NO.
	Image: Construction of the second s
	ACTION FUTURE EFFECT SHUTDOWN HOURS 22 ATTACHMENT NPRD-4 PRIME COMP. COMPUNENT MANUFACTURER MANUFACTURER MANUFACTURER
	CAUSE DESCRIPTION AND CORRECTIVE ACTIONS (27)
10	Cause has not yet been determined. Exhautive investigative programs have been
111	initiated. Long term corrective actions will be reported after source of corro-
112	sion has been identified.
C.L.C.	
[1]3	
1 4	EACILITY (30) METHOD OF DISCOUSERY DESCRIPTION (32)
1 5	STATUS SPOWER OTHER STATUS Discovent Discovent Under Notification 80
, ,	ACTIVITY CONTENT RELEASE OF RELEASE AMOUNT OF ACTIVITY 35 LOCATION OF RELEASE 36
1 6	Z (33) Z (34) N/A N/A N/A 80 9 9 10 11 44 45 80
17	NUMBER 0 0 17 Z 38 N/A 80
	PERSONNEL INJURIES NUMBER DESCRIPTION (4)
7 8	
19	Z 42 8211060078 821028 **/A
/ 8	PUBLICITY PDR ADDCK 05000289 NRC USE ONLY
2 0	N/A 68 69 80 80 80 80 80 80 80 80 80 80 80 80 80
	NAME OF PREPARER C. J. Stephenson (717) 948-8554

LER 82-011/99X-0

SPECIAL REPORT

I. Current Activities at the Time of the Occurrence

TMI-1 was in long term cold shutdown.

II. Leading Circumstances to the Occurrence

A PORV which was installed at the beginning of the second cycle (1976) was removed from service in 1981. In 1982, this valve was shipped to Wyle Laboratories for modification and recertification.

III. Description of the Occurrence

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On August 11, 1982, GPU Nuclear Corporation was notified by Wyle Laboratories that the PORV was corroded and pitted. Wyle took photographs and wrote a description of what had been observed. Unfortunately, prior to testing and inspection, at Wyle Laboratories, the PORV was thoroughly cleaned. When the corrosion was later identified there was not sufficient residue for Wyle Labs to determine the cause of the corrosion. The following is a summary of information currently available:

- 1) Inconel X-750 parts were severely pitted.
- 2) Inconel 600 parts were severely pitted.
- 3) Martensitic Stainless Steel parts were severely pitted.
- 4) 304 Stainless Steel part was corroded.
- 5) Some sulphur has been found but not enough to state that it was the cause.
- 6) B&W Literature search indicates that the type of pitting found is usually caused by chlorides or sulphur.

Even though the source of corrosion has not yet been identified, our extensive investigative program is progressing.

IV. Significant Events Which Occurred as a Result of the Original Occurrence

None. However, from observation of the corroded parts, it appears that the valve was in a closed state when the corrosion attack occurred and it is likely that it would not have functioned properly as a result of the corrosion. A replacement PORV, which was installed in 1981, was actuated during Hot Functional Testing in September, 1981 and functioned properly.

V. Previous Events of a Similar Nature

None. Previous inspections never indicated the state of corrosion as observed at Wyle Laboratories.

VI. Root Cause of Occurrence

The root causes of the corrosion have not, as yet, been determined. The following have been completed or initiated in the process of determining the source of corrosion:

- 1) B&W is doing metallurgical examination of valve internals.
- Safety valve loop seal piping was both visually and ultrasonically inspected. No evidence of corrosion, pitting or cracking was identified.
- As part of OTSG Repair Program, the Reactor Vessel and Steam Generators were thoroughly inspected. No evidence of corrosion was identified.

VII. Immediate Corrective Action

Wyle Laboratories notified GPUN on August 11, 1982 and the TMI-1 Resident Inspector was notified by GPUN on August 16, 1982 after this event was determined to be reportable. An exhaustive investigative program was started as soon as initial observations and photos became available.

VIII. Long Term Corrective Action

No long term corrective action has been decided upon since the cause of the corrosion has not yet been identified.