



Public Service

**Public Service
Company of Colorado**
P.O. Box 840
Denver CO 80201-0840

December 21, 1990
Fort St. Vrain
Unit No. 1
P-90367

A. Clegg Crawford
Vice President
Nuclear Operations

U. S. Nuclear Regulatory Commission
ATTN: Document Control Desk
Washington, D.C. 20555

ATTN: Mr. Seymour H. Weiss, Director
Non-Power Reactor, Decommissioning and
Environmental Project Directorate

Docket No. 50-267

SUBJECT: DECOMMISSIONING TECHNICAL SPECIFICATIONS

- REFERENCES: 1. PSC Letter, Crawford to Weiss, dated November 5, 1990 (P-90318)
2. NRC Generic Letter 89-01, dated January 31, 1989 (G-89040)

Dear Mr. Weiss:

This submittal is a request to amend the Fort St. Vrain (FSV) Facility Operating License and Technical Specifications to provide controls and limits appropriate for decommissioning. The attached proposed Facility License and Decommissioning Technical Specifications (DTS) provide suitable requirements for the decontamination and dismantlement (DECON) of the FSV reactor.

PSC's proposed changes to the Facility Operating License and Technical Specifications are provided in the following attachments:

- Attachment 1: Summary of Proposed Changes
- Attachment 2: Proposed Facility License
- Attachment 3: Proposed Decommissioning Technical Specifications
- Attachment 4: No Significant Hazards Consideration Analysis

In Reference 1, Public Service Company of Colorado (PSC) submitted the FSV Proposed Decommissioning Plan, which included the decommissioning accident analyses. The attached DTS provide administrative controls and limiting conditions to ensure that FSV decommissioning activities are conducted in a manner consistent with the assumptions in the accident analysis, so that potential radiological exposures of the public are within regulatory guidance.

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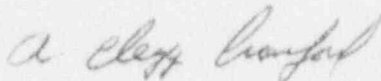
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The DTS were developed in accordance with the guidance in draft Regulatory Guide DG-1005, "Standard Format and Content for Decommissioning Plans for Nuclear Reactors". As suggested in DG-1005, the specification applicabilities were defined such that they would automatically adjust or be discontinued as decommissioning progresses, to minimize requests for future license amendments and technical specification changes. In addition, consistent with the guidance provided in Generic Letter 89-01 (Reference 2), the programmatic controls for the radiological effluent technical specifications are provided in the DTS and the associated procedural details are contained in the FSV Offsite Dose Calculation Manual and the Process Control Program. A detailed description of the implementation of Generic Letter 89-01 guidance is provided in Attachment 5. This separation of procedural details from the technical specifications is also consistent with the guidance in DG-1005, which suggests that procedure-level controls provide flexibility for changing conditions and unforeseen factors.

PSC requests that the proposed Facility License and Decommissioning Technical Specifications be approved for implementation at the beginning of decommissioning activities, after all nuclear fuel has been removed from the Reactor Building and after the NRC has approved the Proposed Decommissioning Plan. The Decommissioning Technical Specifications are intended to supersede and replace in their entirety the current FSV Technical Specifications and Environmental Protection Plan (Non-Radiological), Appendices A and B, respectively, to the FSV Facility Operating License.

If you have any questions regarding these proposed changes, please contact Mr. M. H. Holmes at (303) 480-6960.

Very truly yours,



A. Clegg Crawford
Vice President
Nuclear Operations

ACC/SWC/lmb

Attachments

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cc: Regional Administrator, Region IV
ATTN: Mr. G. L. Constable, Chief
Technical Support Section
Division of Reactor Projects

Robert M. Quillin, Director
Radiation Control Division
Colorado Department of Health

Mr. J. B. Baird
Senior Resident Inspector
Fort St. Vrain

BEFORE THE
UNITED STATES NUCLEAR REGULATORY COMMISSION

In the Matter of the Facility Operating License)
)
)
)
PUBLIC SERVICE COMPANY OF COLORADO)

Application for Amendment to
the Facility Operating License
License No. DPR-34

OF THE
PUBLIC SERVICE COMPANY OF COLORADO
FOR THE
FORT ST. VRAIN NUCLEAR GENERATING STATION

This application for Amendment to the
Facility Operating License, License No. DPR-34,
is submitted for NRC review and approval.

Respectfully submitted,

PUBLIC SERVICE COMPANY OF COLORADO

By A. Clegg Crawford
Vice President,
Nuclear Operations

KELLY, STANSFIELD & O'DONNELL
James K. Tarpey
Mark A. Davidson
Public Service Company Building
Denver, Colorado 80202

Attorneys for Applicant

UNITED STATES OF AMERICA
NUCLEAR REGULATORY COMMISSION

In the Matter of the Facility Operating License)
of)
Public Service Company of Colorado Fort St.) Docket No. 50-267
Vrain Unit No. 1)

AFFIDAVIT

A. Clegg Crawford, being first duly sworn, deposes and says: That he is Vice President, Nuclear Operations, of Public Service Company of Colorado, the Licensee herein, that he has read the foregoing Application of Amendment to the Facility Operating License and knows the contents thereof, and that the statements and matters set forth therein are true and correct to the best of his knowledge, information and belief.

A. Clegg Crawford
A. Clegg Crawford
Vice President,
Nuclear Operations

STATE OF COLORADO)
COUNTY OF DENVER)

Subscribe and sworn to before me, a Notary Public on this
20th day of DECEMBER, 1990.

Delores Romero
Notary Public

My commission expires January 6, 1993.

ATTACHMENT 1

TO P-90367

SUMMARY OF
PROPOSED CHANGES

SUMMARY OF PROPOSED CHANGES

Sections

- Facility Operating License
- Appendix A - Technical Specifications
- Appendix B - Environmental Protection Plan (Non-Radiological)

Description of Change

Replace the License and Appendices A and B in their entirety with the Facility License and Decommissioning Technical Specifications (DTS).

ATTACHMENT 2

TO P-90367

PROPOSED FACILITY LICENSE

2005

PUBLIC SERVICE COMPANY OF COLORADO
FORT ST. VRAIN NUCLEAR GENERATING STATION
DOCKET NO. 50-267
FACILITY LICENSE

License No. DPR-34

1. Facility License No. DPR-34 is hereby issued to the Public Service Company of Colorado to read as follows:
 - A. This license applies to the Fort St. Vrain Nuclear Generating Station, a high temperature gas-cooled nuclear reactor and associated equipment (the facility) owned by the Public Service Company of Colorado. The facility is located near Platteville, in Weld County, Colorado, and is described in the Decommissioning Plan as supplemented and amended and the Environmental Report as supplemented and amended.
 - B. Subject to the conditions and requirements incorporated herein, the Commission hereby licenses the Public Service Company of Colorado:
 - (1) Pursuant to Section 104b of the Act and 10 CFR Part 50, "Domestic Licensing of Production and Utilization Facilities," to possess but not operate the facility at the designated location near Platteville in Weld County, Colorado, and to decommission the facility as described in the licensee's Decommissioning Plan, as supplemented and amended.
 - (2) Pursuant to the Act and 10 CFR Parts 30, 40 and 70, to receive, possess, and use at any time any byproduct, source and special nuclear material as sealed sources for radiation monitoring equipment calibration, in amounts as required;
 - (3) Pursuant to the Act and 10 CFR Parts 30, 40, and 70, to receive, possess and use in amounts as required any byproduct, source or special nuclear material without restriction to chemical or physical form, for sample analysis or instrument calibration or associated with radioactive apparatus or components; and

- (4) Pursuant to the Act and 10 CFR Parts 30, 40 and 70, to possess, but not separate, such byproduct and special nuclear materials as were produced by the operation of the facility authorized herein.
- C. This license shall be deemed to contain and is subject to the conditions specified in the following Commission regulations in 10 CFR Chapter I: Part 20, Section 20.34 of Part 30, Section 40.41 of Part 40, Sections 50.54 and 50.59 of Part 50, and the appropriate sections of Part 70; is subject to all applicable provisions of the Act and to the rules, regulations, and orders of the Commission now or hereafter in effect; and is subject to the additional conditions specified or incorporated below:
- (1) Decommissioning Technical Specifications
- The Decommissioning Technical Specifications contained in Appendix A, as revised through Amendment No. ____, are hereby incorporated in the license. The licensee shall decommission the facility in accordance with the Decommissioning Technical Specifications.
- (2) Applicability
- This Facility License and the Decommissioning Technical Specifications implement the Decommissioning Plan and will be effective after all nuclear fuel has been removed from the Fort St. Vrain Reactor Building and the NRC has approved the Decommissioning Plan.
- D. This license is subject to all Federal, State, and local standards imposed pursuant to the requirements of the Federal Water Pollution Control Act of 1972.
2. This license is effective as of the date of issuance and shall expire at midnight, September 17, 2008.

FOR THE NUCLEAR REGULATORY COMMISSION

Attachments:

Appendix A - Technical Specifications

Date of Issuance: December 21, 1973