



UNITED STATES  
NUCLEAR REGULATORY COMMISSION  
WASHINGTON, D. C. 20555

BOSTON EDISON COMPANY

DOCKET NO. 50-293

PILGRIM NUCLEAR POWER STATION  
AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 63  
License No. DPR-35

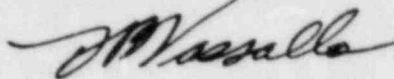
1. The Nuclear Regulatory Commission (the Commission) has found that:
  - A. The application for amendment by the Boston Edison Company (the licensee) dated September 30, 1982 complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations set forth in 10 CFR Chapter I;
  - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
  - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
  - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public;  
- and
  - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.
2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment and paragraph 3.B of Facility Operating License No. DPR-35 is hereby amended to read as follows:

B. Technical Specifications

The Technical Specifications contained in Appendices A and B, as revised through Amendment No. 63, are hereby incorporated in the license. The licensee shall operate the facility in accordance with the Technical Specifications.

3. This license amendment is effective as of the date of issuance.

FOR THE NUCLEAR REGULATORY COMMISSION



Domenic B. Vassallo, Chief  
Operating Reactors Branch #2  
Division of Licensing

Attachment:  
Changes to the Technical  
Specifications

Date of Issuance: October 21, 1982

ATTACHMENT TO LICENSE AMENDMENT NO. 63

FACILITY OPERATING LICENSE NO. DPR-35

DOCKET NO. 50-293

Replace page 160 of the Appendix "A" Technical Specifications with the enclosed page 160.

3.7.D

Primary Containment Isolation Valves

1. Whenever primary containment integrity is required, all isolation valves listed in Table 3.7.1 and all instrument line flow check valves shall be operable except as specified in 3.7.D.2.
  
2. In the event any isolation valve specified in Table 3.7.1 becomes inoperable, at least one containment isolation valve in each line having an inoperable valve shall be placed in the isolated condition.
  
3. If Specification 3.7.D.1 and 3.7.D.2 cannot be met, an orderly shutdown shall be initiated and the reactor shall be in the Cold Shutdown condition within 24 hours.

SURVEILLANCE REQUIREMENTS

4.7.D Primary Containment Isolation Valves

1. The primary containment isolation valves surveillance shall be performed as follows:
  - a. At least once per operating cycle the operable isolation valves that are power operated and automatically initiated shall be tested for simulated automatic initiation and closure times.
  - b. At least once per quarter:
    - (1) All normally open power operated isolation valves (except for the main steam line power-operated isolation valves) shall be fully closed and reopened.
    - (2) Trip main steam isolation valves individually and verify closure time.
  - c. At least twice per week the main steam-line power-operated isolation valves shall be exercised by partial closure and subsequent reopening.
  - d. At least once per operating cycle the operability of the reactor coolant system instrument line flow check valves shall be verified.
  
2. Whenever an isolation valve listed in table 3.7.1 is inoperable, the position of at least one other valve in each line having an inoperable valve shall be recorded daily.