



UNITED STATES  
NUCLEAR REGULATORY COMMISSION  
WASHINGTON, D. C. 20555

GEORGIA POWER COMPANY  
OGLETHORPE POWER CORPORATION  
MUNICIPAL ELECTRIC AUTHORITY OF GEORGIA  
CITY OF DALTON, GEORGIA

DOCKET NO. 50-321

EDWIN I. HATCH NUCLEAR PLANT, UNIT NO. 1

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 91  
License No. DPR-57

1. The Nuclear Regulatory Commission (the Commission) has found that:
  - A. The application for amendment by Georgia Power Company, et al., (the licensee) dated October 6, 1982, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act) and the Commission's rules and regulations set forth in 10 CFR Chapter I;
  - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
  - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
  - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
  - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.
2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment and paragraph 2.C.(2) of Facility Operating License No. DPR-57 is hereby amended to read as follows:

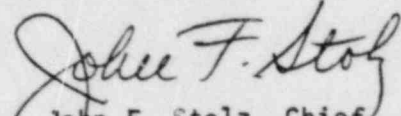
(2) Technical Specifications

The Technical Specifications contained in Appendices A and B, as revised through Amendment No. 91, are hereby incorporated in the license. The licensee shall operate the facility in accordance with the Technical Specifications.

8211050050 821022  
PDR ADOCK 05000321  
PDR

3. This license amendment is effective as of the date of its issuance.

FOR THE NUCLEAR REGULATORY COMMISSION



John F. Stolz, Chief  
Operating Reactors Branch #4  
Division of Licensing

Attachment:  
Changes to the Technical  
Specifications

Date of Issuance: October 22, 1982

ATTACHMENT TO LICENSE AMENDMENT NO. 91

FACILITY OPERATING LICENSE NO. DPR-57

DOCKET NO. 50-321

Replace the following pages of the Appendix "A" Technical Specifications with the enclosed pages. The revised pages are identified by Amendment number and contain a vertical line indicating the area of change.

Remove

3.7-12

--

3.7-13

5.0-1

--

Insert

3.7-12

3.7-12a

3.7-13

5.0-1

5.0-1a

C. Secondary Containment\*1. Secondary Containment Integrity

- a. Integrity of the secondary containment shall be maintained during all modes of Unit 1 plant operation except when all of the following conditions are met:
- (1) The reactor is subcritical and Specification 3.3.A is met.
  - (2) The reactor water temperature is below 212°F and the reactor coolant system is vented.
  - (3) No activity is being performed which can reduce the shutdown margin below that stated in Specification 3.3.A.
  - (4) The fuel cask or irradiated fuel is not being moved in the reactor building.
  - (5) All hatches between Unit 1 secondary containment and Unit 2 secondary containment are closed and sealed.
  - (6) At least one door in each access path between Unit 1 secondary containment and Unit 2 secondary containment is closed.
- b. Integrity of the Unit 1 secondary containment shall be maintained during all modes of Unit 2 plant operations except Operational Condition 4 as defined in the Unit 2 Technical Specifications.

C. Secondary Containment1. Surveillance While Integrity Maintained

Secondary containment surveillance shall be performed as indicated below:

- a. A preoperational secondary containment capability test shall be conducted after isolating the secondary containment and placing the standby gas treatment system filter trains in operation. Such tests shall demonstrate the capability to maintain a minimum 1/4-inch of water vacuum under calm wind (<5 mph) conditions with each filter train flow rate not more than 4000 cfm.
- b. Secondary containment capability to maintain a minimum 1/4-inch of water vacuum under calm wind (<5 mph) conditions with each filter train flow rate not more than 4000 cfm shall be demonstrated at each refueling outage, prior to refueling.

\*For secondary containment during 1982 refueling outage, see page 3.7-12a.

2. Secondary Containment Integrity During 1982 Refueling Outage Only

- a. Unit 1 secondary containment below the refueling floor is not required provided all of the following conditions are met:
- (1) The reactor is subcritical and Specification 3.3.A is met.
  - (2) The reactor water temperature is below 212°F and the reactor coolant system is vented.
  - (3) All hatches between Unit 1 secondary containment and Unit 2 secondary containment are closed and sealed.
  - (4) At least one door in each access path between Unit 1 secondary containment and Unit 2 secondary containment is closed.
  - (5) All hatches separating Unit 1 secondary containment above the refueling floor from Unit 1 secondary containment below the refueling floor are closed and sealed.
  - (6) At least one door in each access path separating Unit 1 secondary containment above the refueling floor from Unit 1 secondary containment below the refueling floor is closed.
- b. Integrity of the Unit 1 secondary containment above the refueling floor shall be maintained during all modes of Unit 2 plant operations except Operational Condition 4 as defined in the Unit 2 Technical Specifications.
- c. Refueling operations may continue in Unit 1 secondary containment (above the refueling floor) provided all conditions in Specification 3.7.C.2.a are met.

2. Surveillance While Integrity Maintained

Secondary containment surveillance shall be performed as indicated below:

- a. A preoperational secondary containment capability test shall be performed after isolating the Unit 1 secondary containment above the refueling floor and placing the standby gas treatment system filter trains in operation. Such tests shall demonstrate the capability to maintain a minimum 1/4-inch of water vacuum under calm wind (<5 mph) conditions with each filter train flow rate not more than 4000 cfm.
- b. If secondary containment integrity should be required as stated in Specification 3.7.C.1, perform surveillance as stated in Specification 4.7.C.1.a. If secondary containment is subsequently required as stated in Specification 3.7.C.2, perform surveillance as stated in Specification 4.7.C.2.a.



### 3.7.C.3 Violation of Secondary Containment Integrity

If Specification 3.7.C.1 cannot be met, procedures shall be initiated to establish conditions listed in Specification 3.7.C.1.a through 3.7.C.1.d.

#### D. Primary Containment Isolation Valves

##### 1. Valves Required to be Operable

During reactor power operation, all primary containment isolation valves listed in Table 3.7-1, and all reactor coolant system instrument line excess flow check valves shall be operable except as stated in Specification 3.7.D.2.

### 4.7.C.3 Surveillance After Integrity Violated

After a secondary containment violation is determined the standby gas treatment system will be operated immediately after the affected zones are isolated from the remainder of the secondary containment. The ability to maintain the remainder of the secondary containment at 1/4-inch of water vacuum pressure under calm ( $\leq 5$  mph) wind conditions shall be confirmed.

#### D. Primary Containment Isolation Valves

##### 1. Surveillance of Operable Valves

Surveillance of the primary containment isolation valves shall be performed as follows:

- a. At least once per operating cycle the operable isolation valves that are power operated and automatically initiated shall be tested for simulated automatic initiation and the closure times specified in Table 3.7-1.

## 5.0 MAJOR DESIGN FEATURES

### A. Site

Edwin I. Hatch Nuclear Plant Unit No. 1 is located on a site of about 2244 acres, which is owned by Georgia Power Company, on the south side of the Altamaha River in Appling County near Baxley, Georgia. The Universal Transverse Mercator Coordinates of the center of the reactor building are: Zone 17R LF 372,935.2m E and 3,533,765.2m N.

### B. Reactor Core

#### 1. Fuel Assemblies

The core shall consist of not more than 560 fuel assemblies of the licensed combination of 7x7 bundles which contain 49 fuel rods and 8x8 fuel bundles which contain 62 or 63 fuel rods each.

#### 2. Control Rods

The reactor shall contain 137 cruciform-shaped control rods. The control material shall be boron carbide powder ( $B_4C$ ) compacted to approximately 70% of its theoretical density.

### C. Reactor Vessel

The reactor vessel is described in Table 4.2-2 of the FSAR. The applicable design specifications shall be as listed in Table 4.2-1 of the FSAR.

### D. Containment

#### 1. Primary Containment

The principal design parameters and characteristics of the primary containment shall be as given in Table 5.2-1 of the FSAR.

#### 2. Secondary Containment\* (See Page 5.0-1a)

The secondary containment shall be as described in Section 5.3.3.1 of the FSAR and the applicable codes shall be as given in Section 12.4.4 of the FSAR.

#### 3. Primary Containment Penetrations

Penetrations to the primary containment and piping passing through such penetrations shall be designed in accordance with standards set forth in Section 5.2.3.4 of the FSAR.

### E. Fuel Storage

#### 1. Spent Fuel

All arrangements of fuel in the spent fuel storage racks shall be maintained in a subcritical configuration having a  $k_{eff}$  not greater than 0.95.

#### 2. New Fuel

The new fuel storage vault shall be such that the  $k_{eff}$  dry shall not be greater than 0.90 and the  $k_{eff}$  flooded shall not be greater than 0.95.

\* 2. Secondary Containment

During the refueling outage beginning October 9, 1982 the Unit 1 secondary containment shall consist of the main stack, the Standby Gas Treatment System, and the portion of the reactor building above the common Unit 1 and 2 refueling floor provided the conditions of Section 3.7.C.2 are met.