

UNITED STATES NUCLEAR REGULATORY COMMISSION WASHINGTON, D. C. 20555

SAFETY EVALUATION BY THE OFFICE OF NUCLEAR REACTOR REGULATION RELATED TO AMENDMENT NO. 76 TO FACILITY OPERATING LICENSE NO. DPR-22 NORTHERN STATES POWER COMPANY

MONTICELLO NUCLEAR GENERATING PLANT

DOCKET NO. 50-263

1.0 INTRODUCTION

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By letter dated October 4, 1990, the Northern States Power Company (the licensee) requested an amendment to the Technical Specifications (TSs) appended to Facility Operating License No. DPR-22 for the Monticello Nuclear Generating Plant. The proposed amendment would correct errors made in previous amendments.

A discussion of each requested change and the NRC staff's evaluation and findings relative to each are addressed in Section 2 of this Safety Evaluation Report.

2.0 DISCUSSION AND EVALUATION

- 2.1 Page 39 Corrections to Bases for Specification 3.1: The proposed change corrects errors made in Amendment 66 issued on May 30, 1989. Text which was inadvertently dropped by Amendment 66 would be restored and minor editorial corrections made. These changes have no safety significance and are therefore acceptable.
- 2.2 Page 127 Safety/Relief Valves Action Statement: The original Monticello overpressure protection system included four safety valves and four safety/relief valves (SRVs). Technical Specification 3.6.E specified the operability requirements for both the salety valves and the SRVs. In April 1974, the four safety valves were replaced with SRVs. Technical Specification 3.6.E was amended by Amendment 3 to the Provisional Operating License to eliminate the safety valve operability requirements and expand the SRV operability requirements reflect the additional SRVs. The amendment deleted the "Action Statement" which specified that an orderly shutdown be initiated if safety valve or SRV operability requirements are not met. It should have provided a new Action Statement applicable to the safety (self-activation) function of the SRVs. The requested amendment will correct this deficiency and is acceptable. (Note: Operability requirements and action statements for the relief and low-low-set functions of SRVs are provided in other Technical Specifications).
- 2.3 Page 127 3.6.E Cross-reference to 3.2.H: Specification 3.6.E requires the low-low-set function of three non-automatic depressurization system SRVs to be operable. The operability requirements for the low-low-set instrumentation which actuates SRVs are contained in Specification 3.2.H. The requested

9101020347 901219 PDR ADOCK 05000263 PDR PDR amendment would add, to Specification 3.6.E, a cross-reference to Specification 3.2.H. A corresponding cruss-reference to 3.6.5 is already provided in 3.2.H. This amendment is editorial in nature and is acceptable.

- 2.4 Page 151 Delete Reference to Thirty Day LCO in 3.6.5 Bases: The Bases for Specification 3.6.E presently states that a thirty-day period is allowed for flexibility in shutting down for a safety-relief valve bellows repair. This statement was included in the original Technical Srecifications but was never incorporated into the associated Limiting Conditions for Operation (LCO). The requested amendment would delete the statement. This will bring the bases into consistency with the associated LCO and is acceptable.
- 2.5 Page 169 Wind Conditions for Secondary Containment Surveillance Test: The proposed change would correct a typographical error made in Amendment No. 3, wherein "(2 < u < 5mph)" was inadvertently replaced by "(2 < u < mph)." The missing "5" would be restored. This change corrects a typographical error made in Amendment 3, issued March 27, 1981, and is acceptable.</p>
- 2.6 Page 189 Reference to USAR for Isolation Valve Closure Time Limits: Amendment No. 71, issued October 19, 1989, relocated a Table specifying isolation valve closure time limits, from the Appendix A Technical specifications, to the Updated Safety Analysis Report (USAR). Due to oversight, Amendment No. 71 neglected to change "4.7.D-Bases Primary Containment Isolation Valves" to reflect the new location of closure time limits. The requested amendment would change "The closure times specified herein" to "The closure times in UFAR Table 5.2-38." This change corrects an administrative error and is acceptable.
- 2.7 Page 189 Delete Paragraph Regarding Steam Line Break Dose Consequences: The proposed amendment would delete a paragraph (second paragraph) from "4.7.D-Bases Primary Containment Isolation Valves" which discusses the basis for a 5-second minimum MSIV closure time. Isolation valve performance requirements were relocated from the Appendix A Technical Specifications to the USAR in Amendment 71 as noted in paragraph 2.6 above. The information to be deleted from the Technical Specifications is contained in paragraph 5.2.3.6.2 of the USAR, and consistent with the intent of Amendment No. 71, may be deleted from the Technical Specifications.
- 2.8 Page 189 References Relating to Primary Containment Isolation Valves: The third paragraph of Page 189 cross-references FSAR Section 5.2.2.4.3 and FSAR Table 5.2.3 for additional information relating to primary containment isolation valves. The requested amendment would change the references to "5.2.2.5.3 and Table 5.2-3b USAR." This is an editorial correction and is acceptable.
- 2.9 <u>Page 198b Reference for Isotopic Analysis:</u> The requested amendment would revise Specification 4.8.8.4.a, changing a reference to "4.8.8.5.c" to "4.8.8.5." This is consistent with Amendment No. 40, which renumbered the 4.8.8.5.c text to 4.8.8.5, and is therefore acceptable.

3.0 ENVIRONMENTAL CONSIDERATION

This amendment involves a change in the installation or use of a facility component located within the restricted area as defined in 10 CFR Part 20

or changes an inspection or surveillance requirement. We have determined that the amendment involves no significant increase in the amounts, and no significant change in the types, of any effluence that may be released offsite, and that there is no significant increase in individual or cumulative occupational radiation exposure. The Commission has previously published a proposed finding that this amendment involves no significant hazards consideration and there has been no public comment on such finding. Accordingly, this amendment meets the eligibility criteria for categorical exclusion set forth in 10 CFR 51.22(c)(9). Pursuant to 10 CFR 51.22(b), no environmental impact statement or environmental assessment need be prepared in connection with the issuance of this amendment.

4.0 CONCLUSION

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We have concluded, based on the considerations discussed above, that (1) there is reasonable assurance that the health and safety of the public will not be endangered by operation in the proposed manner, and (2) such activities will be conducted in compliance with the Commission's regulations, and the issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public.

Principal Contributor. W. Long

Dated: December 19, 1990