



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

SAFETY EVALUATION BY THE OFFICE OF NUCLEAR REACTOR REGULATION

SUPPORTING AMENDMENT NO. 150 TO FACILITY OPERATING LICENSE NO. DPR-71

AND AMENDMENT NO. 180 TO FACILITY OPERATING LICENSE NO. DPR-62

CAROLINA POWER & LIGHT COMPANY

BRUNSWICK STEAM ELECTRIC PLANT, UNITS 1 AND 2

DOCKET NOS. 50-325 AND 50-324

1.0 INTRODUCTION

By letter dated July 20, 1990, Carolina Power & Light Company submitted a request for changes to the Brunswick Steam Electric Plant, Units 1 and 2.

The amendments change the Surveillance Requirements of the Technical Specifications (TS) to comply with Generic Letter 88-01, "NRC Staff Position on IGSCC in BWR Austenitic Stainless Steel Piping." This Generic Letter outlines the NRC's position on intergranular stress corrosion cracking (IGSCC) in boiling water reactor austenitic stainless steel piping.

2.0 EVALUATION

One of the proposed changes is to delete Surveillance Requirement 4.0.5.a.1 and to renumber Surveillance Requirement 4.0.5.a.2 as Surveillance Requirement 4.0.5.a. The Surveillance Requirement 4.0.5.a.1 was only applicable during the time period from issuance of the Facility Operating License to the start of facility commercial operation. Both units have been in commercial operation for several years, therefore, 4.0.5.a.1 is no longer applicable. This change does not impact on the levels of safety that currently exist.

The second proposed change requested that Surveillance Requirement 4.0.6 be deleted and that a new Surveillance Requirement 4.0.5.f be added. Generic Letter 88-01 indicates that the technical specification should include a statement that the in-service inspection program for piping covered by the scope of the Generic Letter will be performed in accordance with the NRC staff positions on schedule, methods, personnel, and sample expansion. The new Surveillance Requirement 4.0.5.f references the new guidance provided in Generic Letter 88-01 and incorporates requirements which supersede those provided in Surveillance Requirement 4.0.6. Therefore, the deletion of Surveillance Requirement 4.0.6 from the TS and addition of Surveillance Requirement 4.0.5.f does not impact on plant safety or operations.

The third proposed change requested that the reactor coolant system (RCS) leakage monitoring frequency, in Surveillance Requirement 4.4.3.2.a, be increased from every 24 hours to every 8 hours. This revision is in compliance with the NRC staff position on leakage monitoring as stated in Generic Letter 88-01. The proposed revision does not involve a physical change or alteration to the facility, therefore, there is no probability of reduction in safety as presently realized. There is no change in the method in which leakage is monitored, and the revision of the monitoring intervals will not impact on the level of safety currently seen. Therefore, this change in monitoring frequency is acceptable.

3.0 ENVIRONMENTAL CONSIDERATIONS

These amendments change a requirement with respect to installation or use of a facility component located within the restricted areas as defined in 10 CFR Part 20 and change the surveillance requirements. The staff has determined that these amendments involve no significant increase in the amounts, and no significant change in the types, of any effluents that may be released off site and that there is no significant increase in individual or cumulative occupational radiation exposure. The Commission has previously issued a proposed finding that these amendments involve no significant hazards consideration, and there has been no public comment on such finding. Accordingly, these amendments meet the eligibility criteria for categorical exclusion set forth in 10 CFR 51.22(c)(9). Pursuant to 10 CFR 51.22(b), no environmental impact statement or environmental assessment need be prepared in connection with the issuance of these amendments.

4.0 CONCLUSION

The Commission made a proposed determination that these amendments involve no significant hazards consideration which was published in the Federal Register (55 FR 34365) on August 22, 1990, and consulted with the State of North Carolina. No public comments or requests for hearing were received, and the State of North Carolina did not have any comments.

The staff has concluded, based on the considerations discussed above, that: (1) there is reasonable assurance that the health and safety of the public will not be endangered by operation in the proposed manner, (2) such activities will be conducted in compliance with the Commission's regulations, and (3) the issuance of the amendments will not be inimical to the common defense and security or to the health and safety of the public.

Dated: December 20, 1990

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AMENDMENT NO. 150 TO FACILITY OPERATING LICENSE NO. DPR-71 - BRUNSWICK, UNIT 1
AMENDMENT NO. 180 TO FACILITY OPERATING LICENSE NO. DPR-62 - BRUNSWICK, UNIT 2

Docket File

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