

**OPERATING DATA REPORT**

DOCKET NO. 50-245  
 DATE 810210  
 COMPLETED BY G. Newburgh  
 TELEPHONE (203) 447-1791  
 X4400

OPERATING STATUS

1. Unit Name: <u>Millstone Unit 1</u>	Notes
2. Reporting Period: <u>September, 1981</u>	
3. Licensed Thermal Power (MWt): <u>2011</u>	
4. Nameplate Rating (Gross MWe): <u>662</u>	
5. Design Electrical Rating (Net MWe): <u>660</u>	
6. Maximum Dependable Capacity (Gross MWe): <u>684</u>	
7. Maximum Dependable Capacity (Net MWe): <u>654</u>	
8. If Changes Occur in Capacity Ratings (Items Number 3 Through 7) Since Last Report, Give Reasons: <u>N/A</u> <u>N/A</u>	
9. Power Level To Which Restricted, If Any (Net MWe): <u>595 MWe</u>	
10. Reasons For Restrictions, If Any: <u>Main Turbine complete 14th stage removal</u>	

	This Month	Yr.-to-Date	Cumulative
i1. Hours In Reporting Period	<u>720</u>	<u>6551</u>	<u>95,015</u>
12. Number Of Hours Reactor Was Critical	<u>698.2</u>	<u>2614.6</u>	<u>69,073.6</u>
13. Reactor Reserve Shutdown Hours	<u>0</u>	<u>1248.5</u>	<u>2,775.8</u>
14. Hours Generator On-Line	<u>622.8</u>	<u>2351.7</u>	<u>66,477.9</u>
15. Unit Reserve Shutdown Hours	<u>0</u>	<u>0</u>	<u>26.5</u>
16. Gross Thermal Energy Generated (MWH)	<u>1,198,531</u>	<u>4,375,177</u>	<u>118,805,304</u>
17. Gross Electrical Energy Generated (MWH)	<u>370,700</u>	<u>1,346,000</u>	<u>40,141,896</u>
18. Net Electrical Energy Generated (MWH)	<u>351,493</u>	<u>1,262,395</u>	<u>38,292,349</u>
19. Unit Service Factor	<u>86.5</u>	<u>35.9</u>	<u>70.0</u>
20. Unit Availability Factor	<u>86.5</u>	<u>35.9</u>	<u>70.0</u>
21. Unit Capacity Factor (Using MDC Net)	<u>74.6</u>	<u>29.5</u>	<u>61.6</u>
22. Unit Capacity Factor (Using DER Net)	<u>74.0</u>	<u>29.2</u>	<u>61.1</u>
23. Unit Forced Outage Rate	<u>13.5</u>	<u>40.2</u>	<u>16.6</u>
24. Shutdowns Scheduled Over Next 6 Months (Type, Date, and Duration of Each): <u>N/A</u>			

25. If Shut Down At End Of Report Period, Estimated Date of Startup: <u>N/A</u>	
26. Units In Test Status (Prior to Commercial Operation):	Forecast      Achieved
INITIAL CRITICALITY	_____
INITIAL ELECTRICITY	_____ N/A _____
COMMERCIAL OPERATION	_____

UNIT SHUTDOWNS AND POWER REDUCTIONS

DOCKET NO. 50-245  
 UNIT NAME Millstone I  
 DATE 811007  
 COMPLETED BY G. Newburgh  
 TELEPHONE (203) 447-1791  
 X4400

REPORT MONTH September

No.	Date	Type <sup>1</sup>	Duration (Hours)	Reason <sup>2</sup>	Method of Shutting Down Reactor <sup>3</sup>	Licensee Event Report #	System Code <sup>4</sup>	Component Code <sup>5</sup>	Cause & Corrective Action to Prevent Recurrence
11	810905	S	55.2	B	1	N/A	HA	TURBIN	Shut down to balance main turbine
12	810914	F	42	A	3	N/A	IA	ZZZZZZ	Power surge to A.T.W.S. system causing scram air header to depressurize

<sup>1</sup>  
 F: Forced  
 S: Scheduled

<sup>2</sup>  
 Reason:  
 A-Equipment Failure (Explain)  
 B-Maintenance of Test  
 C-Refueling  
 D-Regulatory Restriction  
 E-Operator Training & License Examination  
 F-Administrative  
 G-Operational Error (Explain)  
 H-Other (Explain)

<sup>3</sup>  
 Method:  
 1-Manual  
 2-Manual Scram.  
 3-Automatic Scram.  
 4-Other (Explain)

<sup>4</sup>  
 Exhibit G - Instructions for Preparation of Data Entry Sheets for Licensee Event Report (LER) File (NUREG-0161)

<sup>5</sup>  
 Exhibit I - Same Source

REFUELING INFORMATION REQUEST

1. Name of facility: Millstone 1
2. Scheduled date for next refueling shutdown: Fall 1982
3. Scheduled date for restart following refueling: Winter 1982
4. Will refueling or resumption of operation thereafter require a technical specification change or other license amendment?  
Yes. Technical Specification changes regarding:  
(1) Maximum average planar linear heat generating rate  
(2) Maximum critical power ratio
5. Scheduled date(s) for submitting proposed licensing action and supporting information:  
Summer 1982
6. Important licensing considerations associated with refueling, e.g., new or different fuel design or supplier, unreviewed design or performance analysis methods, significant changes in fuel design, new operating procedures:  
172 "Retrofit" 8 X 8 fuel assemblies are scheduled for insertion in Cycle 9  
(Reload 8)
7. The number of fuel assemblies (a) in the core and (b) in the spent fuel storage pool:  
(a) In Core: 580                      (b) In SFP: 954
8. The present licensed spent fuel pool storage capacity and the size of any increase in licensed storage capacity that has been requested or is planned, in number of fuel assemblies:  
2184 Assemblies
9. The projected date of the last refueling that can be discharged to the spent fuel pool assuming the present licensed capacity:  
1985, Spent Fuel Pool, full core off load capability is reached.  
1991, Core Full, spent fuel pool contains 2120 bundles

**AVERAGE DAILY UNIT POWER LEVEL**

DOCKET NO. 50-245  
 UNIT Millstone 1  
 DATE 810210  
 COMPLETED BY G. Newburgh  
 TELEPHONE (203)447-1791  
                   X 4400

MONTH September

DAY	AVERAGE DAILY POWER LEVEL (MWe-Net)	DAY	AVERAGE DAILY POWER LEVEL (MWe-Net)
1	<u>588</u>	17	<u>543</u>
2	<u>588</u>	18	<u>465</u>
3	<u>589</u>	19	<u>589</u>
4	<u>544</u>	20	<u>589</u>
5	<u>0</u>	21	<u>588</u>
6	<u>0</u>	22	<u>589</u>
7	<u>262</u>	23	<u>589</u>
8	<u>551</u>	24	<u>594</u>
9	<u>586</u>	25	<u>593</u>
10	<u>587</u>	26	<u>593</u>
11	<u>587</u>	27	<u>592</u>
12	<u>587</u>	28	<u>592</u>
13	<u>587</u>	29	<u>594</u>
14	<u>281</u>	30	<u>591</u>
15	<u>0</u>	31	<u>N/A -----</u>
16	<u>251</u>		

**INSTRUCTIONS**

On this format, list the average daily unit power level in MWe-Net for each day in the reporting month. Compute to the nearest whole megawatt.

MDC of 654 based on commitment to New  
 England Power Exchange

### September 1981 Operating History

September 1, 1981		Reactor power at 100%
September 4, 1981	2100 Hours	Decreasing reactor power to shutdown for main turbine balancing.
	2355 Hours	Generator off line.
September 7, 1981	2356 Hours	Reactor critical, rod 18-15, notch 6, period 114 sec., moderator temp. 230°F.
September 8, 1981	0711 Hours	Generator on line.
September 8, 1981	1800 Hours	Reactor power at 100%.
September 14, 1981	1131 Hours	Reactor scram due to low scram air header pressure caused by spurious ATWS system isolation.
	2225 Hours	Reactor critical, rod 38-39, notch 6, period 238 sec., moderator temp 470°F.
September 15, 1981	1057 Hours	Reactor scram due to IRM Hi-Hi flux.
	2150 Hours	Reactor critical, rod 18-31, notch 4, period 163 sec., Moderator temp. 453°F.
September 16, 1981	0528 Hours	Generator on line.
September 17, 1981	1600 Hours	Reactor power at 100%.
September 18, 1981	0045 Hours	Reactor power at 40% for main condenser repairs.
	2030 Hours	Reactor power at 100%.
September 23, 1981	0015 Hours	Reactor power at 90% for turbine stop valve testing and main condenser backwashing.
	0117 Hours	Reactor power at 100%.

September 30, 1981

0015 Hours

Reactor power at 90% for  
turbine stop valve testing  
and main condenser  
backwashing.

0150 Hours

Reactor power at 100%.