

OPERATING DATA REPORT

DOCKET NO. 50-409
 DATE 12-10-81
 COMPLETED BY L.S. GOODMAN
 TELEPHONE 608-689-2331

OPERATING STATUS

1. Unit Name: LA CROSSE BOILING WATER REACTOR
2. Reporting Period: 0001, 11-01-81 TO 2400, 11-30-81
3. Licensed Thermal Power (MWt): 165
4. Nameplate Rating (Gross MWe): 65.3
5. Design Electrical Rating (Net MWe): 50
6. Maximum Dependable Capacity (Gross MWe): 50
7. Maximum Dependable Capacity (Net MWe): 48
8. If Changes Occur in Capacity Ratings (Items Number 3 Through 7) Since Last Report. Give Reasons:

Notes

9. Power Level To Which Restricted, If Any (Net MWe): _____
10. Reasons For Restrictions, If Any: _____

	This Month	Yr.-to-Date	Cumulative
11. Hours In Reporting Period	<u>720</u>	<u>8016</u>	<u>105,915</u>
12. Number Of Hours Reactor Was Critical	<u>662.6</u>	<u>6485.0</u>	<u>70,582.0</u>
13. Reactor Reserve Shutdown Hours	<u>0</u>	<u>0</u>	<u>478</u>
14. Hours Generator On-Line	<u>646.4</u>	<u>6111.7</u>	<u>65,143.0</u>
15. Unit Reserve Shutdown Hours	<u>0</u>	<u>0</u>	<u>79</u>
16. Gross Thermal Energy Generated (MWH)	<u>91,085.5</u>	<u>834,305.3</u>	<u>8,918,642.7</u>
17. Gross Electrical Energy Generated (MWH)	<u>26,123</u>	<u>237,126</u>	<u>2,667,761</u>
18. Net Electrical Energy Generated (MWH)	<u>24,388</u>	<u>220,103</u>	<u>2,467,383</u>
19. Unit Service Factor	<u>89.8</u>	<u>76.2</u>	<u>61.5</u>
20. Unit Availability Factor	<u>89.8</u>	<u>76.2</u>	<u>61.6</u>
21. Unit Capacity Factor (Using MDC Net)	<u>70.6</u>	<u>57.2</u>	<u>48.5</u>
22. Unit Capacity Factor (Using DER Net)	<u>67.7</u>	<u>54.9</u>	<u>46.6</u>
23. Unit Forced Outage Rate	<u>10.2</u>	<u>8.8</u>	<u>7.3</u>

24. Shutdowns Scheduled Over Next 6 Months (Type, Date, and Duration of Each):
REFUELING FEBRUARY 1, 1982, 8 WEEKS

25. If Shut Down At End Of Report Period, Estimated Date of Startup: NA

26. Units In Test Status (Prior to Commercial Operation):	Forecast	Achieved
INITIAL CRITICALITY	_____	_____
INITIAL ELECTRICITY	_____	_____
COMMERCIAL OPERATION	_____	_____

AVERAGE DAILY UNIT POWER LEVEL

DOCKET NO. 50-409
 UNIT LACBWR
 DATE 12-10-81
 COMPLETED BY L. S. GOODMAN
 TELEPHONE 608-689-2331

MONTH NOVEMBER 1981

DAY	AVERAGE DAILY POWER LEVEL (MWe-Net)	DAY	AVERAGE DAILY POWER LEVEL (MWe-Net)
1	<u>38</u>	17	<u>38</u>
2	<u>38</u>	18	<u>41</u>
3	<u>32</u>	19	<u>38</u>
4	<u>32</u>	20	<u>39</u>
5	<u>41</u>	21	<u>38</u>
6	<u>41</u>	22	<u>38</u>
7	<u>42</u>	23	<u>38</u>
8	<u>42</u>	24	<u>38</u>
9	<u>42</u>	25	<u>38</u>
10	<u>42</u>	26	<u>38</u>
11	<u>42</u>	27	<u>38</u>
12	<u>15</u>	28	<u>38</u>
13	<u>0</u>	29	<u>38</u>
14	<u>0</u>	30	<u>38</u>
15	<u>9</u>	31	<u> </u>
16	<u>28</u>		

INSTRUCTIONS

On this format, list the average daily unit power level in MWe-Net for each day in the reporting month. Compute to the nearest whole megawatt.

UNIT SHUTDOWNS AND POWER REDUCTIONS

REPORT MONTH NOVEMBER 1981

DOCKET NO. 50-409
 UNIT NAME LACBWR
 DATE 12-10-81
 COMPLETED BY L. S. GOODMAN
 TELEPHONE 608-689-2331

No.	Date	Type ¹	Duration (Hours)	Reason ²	Method of Shutting Down Reactor ³	Licensee Event Report #	System Code ⁴	Component Code ⁵	Cause & Corrective Action to Prevent Recurrence
81-17	11-03-81	S	0	B	4	NA	HC	XXXXX	A POWER REDUCTION FROM 85% TO 64% WAS CONDUCTED FOR THE PURPOSE OF REMOVING EACH OF THE TWO TURBINE CONDENSER WATER BOXES FROM SERVICE TO CLEAN CONDENSER TUBES AND TO REMOVE ACCUMULATED SILT FROM THE WATER BOXES. THE CONDENSER CLEANING WAS ACCOMPLISHED TO IMPROVE TURBINE EXHAUST VACUUM AND TEMPERATURE CONDITIONS AND PLANT EFFICIENCY.
81-18	11-12-81	F	97.6	A	3	NA	IA	INSTRU	THE REACTOR SHUT DOWN AUTOMATICALLY DUE TO A HIGH FLUX SPIKE ON NUCLEAR INSTRUMENTATION CHANNEL 7 OCCURRING DURING THE MONTHLY TECHNICAL SPECIFICATION TEST ON CHANNEL 5, WHICH

¹ F - Forced
 S - Scheduled

² Reason:
 A - Equipment Failure (Explain)
 B - Maintenance or Test
 C - Refueling
 D - Regulatory Restriction
 E - Operator Training & License Examination
 F - Administrative
 G - Operational Error (Explain)
 H - Other (Explain)

³ Method:
 1 - Manual
 2 - Manual Scram
 3 - Automatic Scram
 4 - Other (Explain)

⁴ ... (CONT'D NEXT PAGE)
 Exhibit G - Instructions for Preparation of Data Entry Sheets for Licensee Event Report (LER) File (NUREG-0161)

⁵ Exhibit I - Same Source

UNIT SHUTDOWNS AND POWER REDUCTIONS

REPORT MONTH NOVEMBER 1981

DOCKET NO. 50-409
 UNIT NAME LACBWR
 DATE 12-10-81
 COMPLETED BY L.S. GOODMAN
 TELEPHONE 608-689-2337

No.	Date	Type ¹	Duration (Hours)	Reason ²	Method of Shutting Down Reactor ³	Licensee Event Report #	System Code ⁴	Component Code ⁵	Cause & Corrective Action to Prevent Recurrence
									INVOLVES INSERTING A SCRAM SIGNAL. THUS THE 2 OF 4 LOGIC FOR A HIGH FLUX SCRAM WAS SATISFIED. THE LOW VOLTAGE POWER SUPPLY AND THE FLUX AMPLIFIER FOR NUCLEAR INSTRUMENTATION CHANNEL 7 WERE REPLACED.

¹
 F: Forced
 S: Scheduled

²
 Reason:
 A-Equipment Failure (Explain)
 B-Maintenance or Test
 C-Refueling
 D-Regulatory Restriction
 E-Operator Training & License Examination
 F-Administrative
 G-Operational Error (Explain)
 H-Other (Explain)

³
 Method:
 1-Manual
 2-Manual Scram.
 3-Automatic Scram.
 4-Other (Explain)

⁴
 Exhibit G - Instructions for Preparation of Data Entry Sheets for Licensee Event Report (LER) File (NUREG-0161)

⁵
 Exhibit I - Same Source

(9/77)

INSTRUMENT AND ELECTRICAL MAINTENANCE

EQUIPMENT	NATURE OF MAINTENANCE	LER OR OUTAGE NUMBER	MALFUNCTION		CORRECTIVE ACTION
			CAUSE	RESULT	
RADIATION MONITORING HI-RAD ALARM	CORRECTIVE MR-0977	NA	UNKNOWN	CONTINUOUS ALARM	CLEANED AND ALIGNED PHOTO LAMP
ENVIRONMENTAL MONITOR PUMP	CORRECTIVE MR-1010	NA	USAGE	MOTOR TRIP	REPLACED BEARINGS IN PUMP
STACK MONITOR FILTER DRIVE	CORRECTIVE MR-1014	NA	USAGE	DRIVE SLIP	ADJUSTED DRIVE CLUTCH
SPING 4 (STACK GAS MONITOR)	CORRECTIVE MR-1015	NA	UNKNOWN	HI RANGE CHANNEL INOPERATIVE	REPROGRAMMED UNIT
HI RAD ENTRY ALARM UNIT	CORRECTIVE MR-1024	NA	USAGE	LAMP BURNOUT	REPLACED PHOTO LAMP
AREA MONITOR POWER SUPPLY	CORRECTIVE MR-1030	NA	TUBE SHORT	POWER SUPPLY INOPERATIVE	REPLACED TUBE AND RESISTORS
NUCLEAR CHANNEL 7 POWER SUPPLY	CORRECTIVE MR0991	OUTAGE 81-18	UNKNOWN	SPIKES	REPLACED WITH SPARE
NUCLEAR CHANNEL 3 SYSTEM	CORRECTIVE MR-0997	OUTAGE 81-18	UNKNOWN	PERIOD SPIKES	REPLACED AMPLIFIER, POWER SUPPLY AND CHECKED CABLES
NUCLEAR CHANNEL 5	CORRECTIVE MR-1011	NA	UNKNOWN	IMPROPER OPERATION	REPLACED WITH SPARE
NUCLEAR CHANNEL N-5 THROUGH N-9	PREVENTIVE	NA	TESTS DUE	COMPLETED TESTS	COMPLETED TESTS N-5 THROUGH N-9
SAFETY SYSTEM CHANNELS 1, 2, & 3	PREVENTIVE	NA	TESTS DUE	COMPLETED TESTS	COMPLETED SAFETY SYSTEM TESTS 1, 2 & 3
NUCLEAR CHANNELS 5 & 6 RECORDER	CORRECTIVE MR-0983	NA	WORN OUT MOTOR DRIVE	CHART DRIVE INOPERATIVE	REPLACED MOTOR DRIVE

INSTRUMENT AND ELECTRICAL MAINTENANCE

EQUIPMENT	NATURE OF MAINTENANCE	LER OR OUTAGE NUMBER	MALFUNCTION		CORRECTIVE ACTION
			CAUSE	RESULT	
MAIN AND RESERVE 2400V BUS 1B BREAKER	CORRECTIVE MR-0993	OUTAGE 81-18 LER- 81-13	UNDETERMINED	RESERVE BREAKER DID NOT AUTO- MATICALLY CLOSE WHEN MAIN BREAK- ER OPENED	TESTED ALL SYSTEMS FOR PROPER OPERATION
DRY CLEANER HEATER	CORRECTIVE MR-0996	NA	USAGE	HEATER BURNOUT	REPLACED HEATER
PENETRATION LEAK TEST GAUGES	CORRECTIVE MR-1025	NA	NEW INSTALL- ATION	FITTING LEAKS	REPAIRED LEAKS AT FITTINGS
CRD CHARGING PUMPS	CORRECTIVE MR-1033	NA	RELIEF VALVE LEAKING	PUMP OPERATING EXCESSIVELY	RESET RELIEF VALVE AND PRESSURE SWITCH
WASTE TREATMENT BUILDING SECURITY DOOR	CORRECTIVE MR-0987	NA	LATCH NOT ENGAGING	DOOR NOT SECURE	LUBRICATED LATCH SYSTEM
SECURITY TRUCK RADIO	CORRECTIVE MR-1012	NA	UNKNOWN	FREQUENCY DRIFT	REALIGNED RADIO

MECHANICAL MAINTENANCE

EQUIPMENT	NATURE OF MAINTENANCE	LER OR OUTAGE NUMBER	MALFUNCTION		CORRECTIVE ACTION
			CAUSE	RESULT	
SAMPLING SYSTEM (STACK GAS)	NEW INSTALLATION FACILITY CHANGE FC-71-80-1 MR-1007	NA	NA	NA	NEW INSTALLATION OF POST ACCIDENT SAMPLING SYSTEM
FABRICATING RESTRAINTS	FACILITY CHANGE FC-53-8C-2	NA	NA	NA	FABRICATED SEISMIC RESTRAINTS FOR PIPING
REACTOR FEEDWATER PUMPS	PREVENTIVE MR-0992	NA	NA	NA	LUBRICATED SCOOI UBES ON 1A AND 1B FEEDWATER PUMPS
CONTROL ROD DRIVE (CRD) CHARGING PUMPS	CORRECTIVE MR-0994	OUTAGE 81-17	OVERHEATED	NO PUMP OUTPUT	REPLACED WITH NEW PUMPS
CRD CHARGING SYSTEM DISCHARGE VALVES	CORRECTIVE SUBSTITUTION #32-3 MR-0921	NA	EXCESSIVE WEAR	PYDRAUL LEAKS	SUBSTITUTED WITH NEW WORCESTOR BALL VALVES
1A SEAL INJECTION PUMP	CORRECTIVE MR-0990	NA	EXCESSIVE WEAR	LEAKS SEAL WATER	REPACKED ALL THREE PLUNGERS; CHECKED SEAL WATER TO PACKING
LUBE OIL PURIFIER	CORRECTIVE MR-1006	NA	BEARING OVER-HEATED	SEIZED	REPLACE IDLER ARM, PULLEY, BEARINGS AND SPACERS
AIR CONDITIONER, CONTAINMENT BUILDING (EAST)	CORRECTIVE MR-1009	NA	LOSS OF RE-FRIGERANT	BULLS EYE SITE GLASS LEAKING	REPAIRED BULLS EYE AND RECHARGED

NARRATIVE SUMMARY OF OPERATING EXPERIENCE

NOVEMBER 1981

At the onset of the November reporting period, power generation continued at 85% Reactor Rated Thermal Power (38 MWe-Net). This power level has been scheduled to extend the time period between refueling outages.

On November 3, 1981, a power reduction from 85% Reactor Rated Thermal Power (26 MWe-Net) was conducted for the purpose of removing each of the two turbine condenser water boxes from service to clean condenser tubes and remove accumulated silt from the water boxes. The reactor was returned to 85% Rated Thermal Power on November 4. Power escalation continued until 93% Rated Thermal Power (42 MWe-Net) was achieved on November 8, due to DPC system demand.

At 0831 on November 12, 1981, the reactor automatically shutdown due to a high flux spike occurring on Nuclear Instrumentation (NI) Channel No. 7 while the monthly Technical Specification test was being performed on NI Channel 5, which involves inserting a scram signal. The spike provided the second trip signal to satisfy the 2 of 4 scram logic. The reactor was returned to critical at 1753 on November 14 and the turbine-generator was synchronized to the DPC grid at 1010 on November 15.

Power escalation continued until November 17 when 93% Reactor Rated Thermal Power (42 MWe-Net) was achieved. Power was reduced to 86% (39 MWE-Net) on November 18 due to change in NI Channel 5 output. The drawer for NI Channel 5 was changed and a satisfactory surveillance test performed. Power generation continued at 85% - 86% Rated Thermal Power (38 MWe-Net) for the remainder of the reporting period.

Significant maintenance items performed during the November 1981 reporting period are indicated on the attached Instrument and Electrical Maintenance and Mechanical Maintenance listings.

REFUELING INFORMATION REQUEST

1. Name of Facility

La Crosse Boiling Water Reactor (LACBWR)

2. Scheduled Date for Next Refueling Shutdown

The tentatively scheduled date for the next refueling shutdown (EOC-VII) is February 1, 1982.

3. Scheduled Date for Restart Following Refueling

The tentatively scheduled date for subsequent reactor startup is April 1, 1982.

4. Will Refueling or Resumption of Operation Thereafter Require a Technical Specification Change or Other License Amendment?

If answer is yes, what, in general, will these be?

If answer is no, has the reload fuel design and core configuration been reviewed by your Plant Safety Review Committee to determine whether any unreviewed safety questions are associated with the core reload (Ref. 10 CFR Section 50.59)?

If no such review has taken place, when is it scheduled?

No Technical Specification Change or license amendment will be needed for refueling or subsequent resumption of operation. The Safety Review Committee's 50.59 review of the reload fuel design and core configuration will be conducted prior to the commencement of refueling.

5. Scheduled Date(s) for Submitting Proposed Licensing Action and Supporting Information.

N/A

6. Important Licensing Considerations Associated with Refueling, e.g., New or Different Fuel Design or Supplies, Unreviewed Design or Performance Analysis Methods, Significant Changes in Fuel Design, New Operating Procedure.

None

7. The Number of Fuel Assemblies (a) in the Core and (b) in the Spent Fuel Pool

Core Loading: 72 Fuel Assemblies

Spent Fuel Storage Pool Loading: 165 Irradiated Fuel Assemblies

REFUELING INFORMATION REQUEST - (Cont'd)

8. The Present Licensed Spent Fuel Pool Storage Capacity and the Size of any Increase in Licensed Storage Capacity that has been Requested or is Planned, in Number of Fuel Assemblies.

440 Fuel Assemblies

9. The Projected Date of the Last Refueling that can be Discharged to the Spent Fuel Pool Assuming the Present Licensed Capacity.

1990