

**OPERATING DATA REPORT**

DOCKET NO. 50-409  
 DATE 01-05-82  
 COMPLETED BY L. S. GOODMAN  
 TELEPHONE 608-689-2331

**OPERATING STATUS**

1. Unit Name: La Crosse Boiling Water Reactor
2. Reporting Period: 0001, 12-01-81 TO 2400, 12-31-81
3. Licensed Thermal Power (Mwt): 165
4. Nameplate Rating (Gross MWe): 65.3
5. Design Electrical Rating (Net MWe): 50
6. Maximum Dependable Capacity (Gross MWe): 50
7. Maximum Dependable Capacity (Net MWe): 48
8. If Changes Occur in Capacity Ratings (Items Number 3 Through 7) Since Last Report, Give Reasons:

Notes

9. Power Level To Which Restricted, If Any (Net MWe): \_\_\_\_\_
10. Reasons For Restrictions, If Any: \_\_\_\_\_

	This Month	Yr.-to-Date	Cumulative
11. Hours In Reporting Period	744	8760	106,659
12. Number Of Hours Reactor Was Critical	549.3	7034.3	71,131.3
13. Reactor Reserve Shutdown Hours	0	0	478
14. Hours Generator On-Line	549.3	6660.9	65,692.2
15. Unit Reserve Shutdown Hours	0	0	79
16. Gross Thermal Energy Generated (MWH)	77,146.1	911,451.3	8,995,788.8
17. Gross Electrical Energy Generated (MWH)	22,097	259,223	2,689,858
18. Net Electrical Energy Generated (MWH)	20,609	240,712	2,487,992
19. Unit Service Factor	73.8	76.0	61.6
20. Unit Availability Factor	73.8	76.0	61.7
21. Unit Capacity Factor (Using MDC Net)	57.7	57.3	48.6
22. Unit Capacity Factor (Using DER Net)	55.4	55.0	46.7
23. Unit Forced Outage Rate	26.2	10.5	7.5

24. Shutdowns Scheduled Over Next 6 Months (Type, Date, and Duration of Each):  
REFUELING OUTAGE, MARCH 1, 1982, 8 WEEKS.

25. If Shut Down At End Of Report Period, Estimated Date of Startup: JANUARY 15, 1982

26. Units In Test Status (Prior to Commercial Operation):	Forecast	Achieved
INITIAL CRITICALITY	_____	_____
INITIAL ELECTRICITY	_____	_____
COMMERCIAL OPERATION	_____	_____

**AVERAGE DAILY UNIT POWER LEVEL**

DOCKET NO. 50-409  
 UNIT LACBWR  
 DATE 01-05-82  
 COMPLETED BY L. S. GOODMAN  
 TELEPHONE 608-689-2331

MONTH DECEMBER 1981

DAY	AVERAGE DAILY POWER LEVEL (MWe-Net)	DAY	AVERAGE DAILY POWER LEVEL (MWe-Net)
1	<u>38</u>	17	<u>38</u>
2	<u>38</u>	18	<u>38</u>
3	<u>38</u>	19	<u>38</u>
4	<u>38</u>	20	<u>38</u>
5	<u>38</u>	21	<u>38</u>
6	<u>38</u>	22	<u>38</u>
7	<u>38</u>	23	<u>34</u>
8	<u>38</u>	24	<u>0</u>
9	<u>38</u>	25	<u>0</u>
10	<u>38</u>	26	<u>0</u>
11	<u>38</u>	27	<u>0</u>
12	<u>38</u>	28	<u>0</u>
13	<u>38</u>	29	<u>0</u>
14	<u>38</u>	30	<u>0</u>
15	<u>38</u>	31	<u>0</u>
16	<u>38</u>		

**INSTRUCTIONS**

On this format, list the average daily unit power level in MWe-Net for each day in the reporting month. Compute to the nearest whole megawatt.

UNIT SHUTDOWNS AND POWER REDUCTIONS

DOCKET NO. 50-409  
 UNIT NAME LACBWR  
 DATE 01-05-82  
 COMPLETED BY L. S. GOODMAN  
 TELEPHONE 608-689-2331

REPORT MONTH DECEMBER 1981

No.	Date	Type <sup>1</sup>	Duration (Hours)	Reason <sup>2</sup>	Method of Shutting Down Reactor <sup>3</sup>	Licensee Event Report #	System Code <sup>4</sup>	Component Code <sup>5</sup>	Cause & Corrective Action to Prevent Recurrence
81-19	12-23-81	F	194.7	A	3	LER-81-14	XX	XXXXX	THE CAUSE OF THE SHUTDOWN IS UNDER INVESTIGATION.

<sup>1</sup>  
 F Forced  
 S Scheduled

<sup>2</sup>  
 Reason:  
 A-Equipment Failure (Explain)  
 B-Maintenance of Test  
 C-Refueling  
 D-Regulatory Restriction  
 E-Operator Training & License Examination  
 F-Administrative  
 G-Operational Error (Explain)  
 H-Other (Explain)

<sup>3</sup>  
 Method:  
 1-Manual  
 2-Manual Scram.  
 3-Automatic Scram.  
 4-Other (Explain)

<sup>4</sup>  
 Exhibit G - Instructions for Preparation of Data Entry Sheets for Licensee Event Report (LER) File (NUREG-0161)

<sup>5</sup>  
 Exhibit I - Same Source

## INSTRUMENT AND ELECTRICAL MAINTENANCE

EQUIPMENT	NATURE OF MAINTENANCE	LER OR OUTAGE NUMBER	MALFUNCTION		CORRECTIVE ACTION
			CAUSE	RESULT	
RELOCATED SPING ISOLATOR, RADIATION MONITOR	CORRECTIVE MR-0974 MR-0984 MR-1038	NA	FACTORY INFO	RELOCATED UNIT	RELOCATED ISOLATOR
STACK MONITOR	CORRECTIVE MR-1014	NA	CLUTCH LOOSE	PAPER SLIP	TIGHTENED CLUTCH
ENVIRONMENTAL AIR MONITOR	CORRECTIVE MR-1055	NA	FUSE BLOWN	INOPERATIVE PUMP	REPLACED MOTOR AND PUMP
*AREA MONITOR POWER SUPPLY 11-15	CORRECTIVE MR-1079	OUTAGE 81-19	SHORTED TUBE	INOPERATIVE POWER SUPPLY	REPLACED TUBES AND RESISTOR
*AREA MONITOR #7 DETECTOR	CORRECTIVE MR-1095	OUTAGE 81-19	SPIKES	DETECTOR INOPERATIVE	REPLACED DETECTOR WITH SPARE
SECURITY E-FIELD 19-3	CORRECTIVE MR-1045	NA	UNKNOWN	ALARMS	BALANCED SENSITIVITY
SECURITY CAMERA 5	CORRECTIVE MR-1056	NA	USAGE	POOR PICTURE	ADJUSTED AUTO LENSE SENSITIVITY
SECURITY CARD READER 26	CORRECTIVE MR-1068	NA	UNKNOWN	INOPERATIVE READER	REPLACED WITH SPARE UNIT
NUCLEAR CHANNEL N-5 THROUGH N-9	PREVENTIVE	NA	TESTS DUE	COMPLETED TESTS	COMPLETED TESTS N-5 THROUGH N-9
SAFETY CHANNELS 1, 2 AND 3	PREVENTIVE	NA	TESTS DUE	COMPLETED TESTS	COMPLETED SAFETY SYSTEM TESTS 1, 2 AND 3
NUCLEAR CHANNELS N-5 THROUGH N-8	PREVENTIVE MR-1066	NA	REQUEST	RESET SCRAM POINTS	RESET SCRAM POINTS
FIRE DETECTOR TESTS	PREVENTIVE	NA	TEST DUE	COMPLETED TESTS	COMPLETED SEMI-ANNUAL TESTS

## INSTRUMENT AND ELECTRICAL MAINTENANCE

EQUIPMENT	NATURE OF MAINTENANCE	LER OR OUTAGE NUMBER	MALFUNCTION		CORRECTIVE ACTION
			CAUSE	RESULT	
PURIFICATION RECORDER	CORRECTIVE MR-1048	NA	USAGE	PRINT ASSEMBLY INOPERATIVE	ADJUSTED PRINT MECHAN- ISM
SECONDARY CONTROL ROD DRIVE (CRD) INDICATOR ROD 21	CORRECTIVE MR-0959	NA	USAGE	LAMP INOPERA- TIVE	REPLACED LAMP
*POST ACCIDENT SAMPLE SYSTEM	FACILITY CHANGE FC-71-80-1 MR-1029 MR-1063	OUTAGE 81-19	NES INSTALL- ATION	COMPLETED INSTALLATION	COMPLETED H <sub>2</sub> SAMPLE INSTALLATION
*NUCLEAR CHANNEL N-4	CORRECTIVE MR-1082	OUTAGE 81-19	POWER SUPPLY INOPERATIVE	VOLTAGE LOW	REPLACED TUBES IN POWER SUPPLY
*NUCLEAR CHANNEL N-5 CHASSIS	CORRECTIVE MR-1069 MR-1011	OUTAGE 81-19	UNKNOWN	POOR OPERATION	REPLACED WITH SPARE UNIT
*NUCLEAR CHANNEL N-3 CHASSIS	CORRECTIVE MR-1082	OUTAGE 81-19	UNKNOWN	SPIKES	REPLACED WITH SPARE UNIT
*RESERVE BREAKER 252R1B	CORRECTIVE MR-1080	LER 81-19	CHARGING SWITCH OUT OF POSITION	BREAKER DID NOT CLOSE	REALIGNED SWITCH AND CLOSED BREAKER
*POST ACCIDENT SAMPLE SYSTEM	CORRECTIVE MR-1089	OUTAGE 81-19	INCORRECT VALVES	INCORRECT VALVES	REPLACED SOLENOID VALVES
*GENERATOR VIBRATION MONITOR	CORRECTIVE MR-0896	OUTAGE 81-19	NORMAL USAGE	MOTOR BEARING BAD	REPLACED BEARING IN M-G POWER SUPPLY
*COND. VACUUM CIRCUIT	CORRECTIVE MR-1099	OUTAGE 81-19	SHORTED LAMP	BLOWN FUSE	REPLACED LAMP

## INSTRUMENT AND ELECTRICAL MAINTENANCE

EQUIPMENT	NATURE OF MAINTENANCE	LER OR OUTAGE NUMBER	MALFUNCTION		CORRECTIVE ACTION
			CAUSE	RESULT	
*LPSW MAKE-UP COOLING VALVE	CORRECTIVE MR-1033	OUTAGE 81-19	VALVE LEAKING	VALVE STICKING	REPAIRED VALVE AND STROKED VALVE
*CONTROL ROD DRIVE #3	CORRECTIVE MR-0989	OUTAGE 81-19	STICKING FULL OUT SWITCH	LAMP INOPERATIVE	REPLACED WITH SPARE - REPLACED SWITCH
*FORCED CIRCULATION PUMP 1A INSTRUMENTS	CORRECTIVE MR-1092	OUTAGE 81-19	SEAL LEAKING	LEAKAGE	REMOVED AND REPLACED INSTRUMENTS FOR SEAL OVERHAUL

MECHANICAL MAINTENANCE

EQUIPMENT	NATURE OF MAINTENANCE	LER OR OUTAGE NUMBER	MALFUNCTION		CORRECTIVE ACTION
			CAUSE	RESULT	
1B CRD CHARGING PUMP	CORRECTIVE MR-1041	NA	OVERHEATED	WOULD NOT CHARGE UP TO FULL PRESSURE	INSTALLED A NEW PUMP
SAMPLING SYSTEM (STACK GAS)	NEW INSTALLATION FACILITY CHANGE FC-45-81-4 MR-1007	NA	NA	NA	MEASURE STACK GAS
EMERGENCY SERVICE WATER SUPPLY SYSTEM (ESWSS)	CORRECTIVE MR-1064	NA	FROZE UP	BROKE 5" VALVE	REPLACED 5" VALVE WITH USE OF CRANE
CONTROL ROD DRIVE (CRD) SYSTEM CHARGING BLADDER	CORRECTIVE MR-1075	OUTAGE 81-19	VALVE CORE LOOSE	LOST ITS CHARGE	LIGHTENED VALVE CORE
LOWER CONTROL ROD DRIVES (LCRD's) POSITION #4 & #13	CORRECTIVE #4 RCD/MR-0843 #13 ROD/MR-1002	OUTAGE 81-19	WORN PACKING	OIL LEAK (PYDRAUL)	DROPPED RODS AND REPACKED
1A FORCED CIRCULATION PUMP	CORRECTIVE MR-1042	OUTAGE 81-19	WORN SEALS	EXCESSIVE SEAL WATER LEAKOFF FLOW	REPLACED UPPER SEALS, FLOATING SLEEVE, O-RING MECHANICAL SEAL, O-RING IS SUBSTITUTION #50-01
1A FORCED CIRCULATION PUMP	FACILITY CHANGE FC-50-81-3	OUTAGE 81-19	NA	NA	REMOVED THERMOCOUPLE WIRES FROM DUAL SEAL, PLUGGED HOLES WITH 1/8" PIPE PLUGS
REACTOR SITE GLASS #50-42-803	CORRECTIVE MR-0905	OUTAGE 81-19	STEAM CUT MICA'S	STEAM LEAKAGE	REPLACED GASKETS, MICA'S, GLASSES AND CUSHIONS IN UPPER SECTION 4-5-6 PARTS FROM TOP BOTH SIDES.
1A FORCED CIRCULATION PUMP ROTOVALVE DISCHARGE	CORRECTIVE MR-1088	OUTAGE 81-19	WORN PACKING	EXCESSIVE LEAKAGE	REPACKED VALVE WITH CHESTERTON 1725 & 1500 PACKING

NARRATIVE SUMMARY OF OPERATING EXPERIENCE

DECEMBER 1981

At the onset of the December 1981 reporting period, power generation continued at 85% Reactor Rated Thermal Power (38 MWe-Net). This power level has been scheduled to extend the time period between refueling outages.

At 2115 on December 23, 1981, the reactor automatically shut down. The cause of the shutdown is under investigation. The plant remained in the shutdown condition for the duration of the December 1981 reporting period.

Significant maintenance items performed during December 1981 are indicated on the attached. Instrument and Electrical Maintenance and Mechanical Maintenance listings.



REFUELING INFORMATION REQUEST

1. Name of Facility

La Crosse Boiling Water Reactor (LACBWR)

2. Scheduled Date for Next Refueling Shutdown

The tentatively scheduled date for the next refueling shutdown (EOC-VII) is March 1, 1982.

3. Scheduled Date for Restart Following Refueling

The tentatively scheduled date for subsequent reactor startup is May 1, 1982.

4. Will Refueling or Resumption of Operation Thereafter Require a Technical Specification Change or Other License Amendment?

If answer is yes, what, in general, will these be?

If answer is no, has the reload fuel design and core configuration been reviewed by your Plant Safety Review Committee to determine whether any unreviewed safety questions are associated with the core reload (Ref. 10 CFR Section 50.59)?

If no such review has taken place, when is it scheduled?

No Technical Specification Change or license amendment will be needed for refueling or subsequent resumption of operation. The Safety Review Committee's 50.59 review of the reload fuel design and core configuration will be conducted prior to the commencement of refueling.

5. Scheduled Date(s) for Submitting Proposed Licensing Action and Supporting Information.

N/A

6. Important Licensing Considerations Associated with Refueling, e.g., New or Different Fuel Design or Supplies, Unreviewed Design or Performance Analysis Methods, Significant Changes in Fuel Design, New Operating Procedure.

None

7. The Number of Fuel Assemblies (a) in the Core and (b) in the Spent Fuel Pool.

Core Loading: 72 Fuel Assemblies

Spent Fuel Storage Pool Loading: 165 Irradiated Fuel Assemblies

REFUELING INFORMATION REQUEST - (Cont'd)

8. The Present Licensed Spent Fuel Pool Storage Capacity and the Size of any Increase in Licensed Storage Capacity that has been Requested or is Planned, in Number of Fuel Assemblies.

440 Fuel Assemblies

9. The Projected Date of the Last Refueling that can be Discharged to the Spent Fuel Pool Assuming the Present Licensed Capacity.

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