RESPONSE TO U.S. NUCLEAR REGULATORY COMMISSION GENERIC LETTER 88-11 FOR THE BRAIDWOOD UNIT 1 REACTOR VESSEL

by

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WORK PERFORMED FOR COMMONWEALTH EDISON COMPANY

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RESPONSE TO U.S. NUCLEAR REGULATORY COMMISSION GENERIC LETTER 88-11 FOR THE BRAIDWOOD UNIT 1 REACTOR VESSEL

1. INTRODUCTION

The U.S. Nuclear Regulatory Commission issued Generic Letter 88-11, "NRC Position on Radiation Embrittlement of Reactor Vessel Materials and its Impact on Plant Operations,"[1] on July 12, 1988. The purpose of the generic letter was to call attention to Revision 2 to Regulatory Guide 1.99, "Radiation Embrittlement of Reactor Vessel Materials,"[2] which became effective May 31, 1988.

Within 180 days of the effective date of the Regulatory Guide (November 27, 1988), each licensee must submit the results of a technical analysis relative to the implementation of this new regulation. This analysis includes the following:

1) Recalculation of adjusted reference temperature values at the 1/4 and 3/4 reactor vessel wall thickness locations for all potentially limiting materials using Regulatory Guide 1.99 Revision 2 and making a comparison of these values with the reference temperatures that were the basis for the pressure-temperature (P/T) limits in the current plant Technical Specifications.

2) Determining the date of applicability of the P/T limits using Regulatory Guide Revision 2.

3) If the use of the Revision 2 methodology may result in a modification of the P/T limits contained in the Technical Specification, a proposed schedule must be defined for whatever actions need to be taken, including those required to address an expected restriction in operating flexibility. If such actions are necessary (e.g., new P/T limits, procedure and hardware modifications, etc.), a schedule for completion (fully implemented and operational) is negotiable as

long as all actions are completed within two plant outages (approximately 3 years) after the effective date of Revision 2 to Regulatory Guide 1.99.

4) Recalculation of reference temperatures for pressurized thermal shock (RT_{PTS}), if so desired by the licensee, since the NRC is presently considering an amendment to the PTS Rule, 10 CFR 50.61 [3]. This amendment will replace the equations for RT_{PTS} in the PTS Rule with the calculational procedure of Revision 2 to Regulatory Guide 1.99, but will not change the screening criterion.

Following a discussion of the Regulatory Guide 1.99 Revision 2 methodology in section 2, the results of the above technical analysis are then provided. Items 1 and 2 are presented in section 3, and revised RT_{PTS} values (item 4) are shown in section 4. Conclusions are given in section 5 and are followed by the references for the report in section 6.

2. REGULATORY GUIDE 1.99 - REVISION 2 METHODOLOGY [2]

2.1 Surveillance Data Not Available

When credible surveillance data from the reactor in question are not available, calculation of neutron radiation embrittlement of the beltline reactor vessels of light-water reactors should be based on the procedures below.

The adjusted reference temperature (ART) for each material in the beltline is given by the following expression:

$$ART = \text{Initial } RT_{NDT} + \Delta RT_{NDT} + \text{Margin} \qquad (1)$$

Initial RT_{NDT} is the reference temperature for the unirradiated material as defined in Paragraph NB-2331 of Section III of the ASME Boiler and Pressure Vessel Code [4]. If measured values of initial RT_{NDT} for the material in question are not available, generic mean values for that class* of material may be used if there are sufficient test results to establish a mean and standard deviation for the class.

ΔRT_{NDT} is the mean value of the adjustment in reference temperature caused by irradiation and should be calculated as follows:

$$\Delta RT_{NDT} = (CF)\, f^{(0.28 - 0.10 \log f)} \qquad (2)$$

CF (°F) Is the chemistry factor, a function of copper and nickel content. CF is given in tables for welds and base metal (plates and forgings) in the Guide and linear interpolation is permitted to obtain the value. In the table, the "weight-percent copper" and "weight-percent nickel" are the best-estimate values for the material, which will normally be the mean of the measured values for a plate or forging or for weld samples made with the weld wire heat number that matches the critical vessel weld. If such values are not

*The class for estimating initial RT_{NDT} is generally determined, for the welds with which this guide is concerned, by the type of welding flux (Linde 80 or other); for base metal, by the ASTM Standard Specification.

available, the upper limiting values given in the material specifications to which the vessel was built may be used. If not available, conservative estimates (mean plus one standard deviation) based on generic data may be used if justification is provided. If there is no information available, 0.35% copper and 1.0% nickel should be assumed.

The neutron fluence at any depth in the vessel wall, $f(10^{19}$ n/cm^2, E > 1 MeV), is determined as follows:

$$f = f_{surf} \left(e^{-0.24x}\right) \qquad (3)$$

where f_{surf} (10^{19} n/cm^2, E > 1 MeV) is the calculated value of the neutron fluence at the inner wetted surface of the vessel at the location of the postulated defect, and x (in inches) is the depth into the vessel wall measured from the vessel inner (wetted) surface. Alternatively, if dpa calculations are made as part of the fluence analysis, the ratio of dpa at the depth in question to dpa at the inner surface may be substituted for the exponential attenuation factor in equation 3.

The fluence factor, $f^{(0.28 - 0.10 \log f)}$, is determined by calculation or from a figure in the guide.

"Margin" is the quantity, °F, that is to be added to obtain conservative, upper-bound values of adjusted reference temperature for the calculations required by Appendix G to 10 CFR Part 50.

$$\text{Margin} = 2\sqrt{\sigma_I^2 + \sigma_\Delta^2} \qquad (4)$$

Here, σ_I is the standard deviation for the initial RT_{NDT}. If a measured value of initial RT_{NDT} for the material in question is available, σ_I is to be estimated from the precision of the test method. If not, a generic mean values for that class of material are used, σ_I is the standard deviation obtained from the set of data used to establish the mean.

The standard deviation for ΔRT_{NDT}, σ_Δ, is 28°F for welds and 17°F for base metal, except that σ_Δ need not exceed 0.50 times the mean value of ΔRT_{NDT}.

2.2 Surveillance Data Available

When two or more credible surveillance data sets become available from the reactor in question, they may be used to determine the adjusted reference temperature of the beltline materials.

The adjusted reference temperature should be obtained as follows. First, if there is clear evidence that the copper or nickel content of the surveillance weld differs from that of the vessel weld, i.e., differs from the average for the weld wire heat number associated with the vessel weld and the surveillance weld, the measured values of ΔRT_{NDT} should be adjusted by multiplying them by the ratio of the chemistry factor for the vessel weld to that for the surveillance weld. Second, the surveillance data should be fitted using equation 2 to obtain the relationship of ΔRT_{NDT} to fluence. To do so, the chemistry factor, CF, is calculated for the best fit by multiplying each adjusted ΔRT_{NDT} by its corresponding fluence factor, summing the products, and dividing by the sum of the squares of the fluence factors. The resulting value of CF when entered in equation 2 will give the relationship of ΔRT_{NDT} to fluence that fits the plant surveillance data in such a way as to minimize the sum of the squares of the errors.

To calculate the margin in this case, equation 4 is used; the values given there for a σ_Δ may be cut in half.

If this procedure gives a higher value of adjusted reference temperature than that given by using the procedures in section 2.1, the surveillance data should be used. If this procedure gives a lower value, either may be used.

For plants having surveillance data that are credible in all respects except that the material does not represent the critical material in the vessel, the calculative procedures in this guide should be used to obtain mean values of shift, ΔRT_{NDT}. In calculating the margin, the value of σ_Δ may be reduced from the values given in the last paragraph of section 2.1 by an amount to be decided on a case-by-case basis, depending on where the measured values fall relative to the mean calculated for the surveillance materials.

3. TECHNICAL ANALYSIS

3.1 Identification and Location of Beltline Region Materials

Figure 1 identifies and indicates the location of all beltline region materials for the Braidwood Unit 1 reactor vessel. The beltline region is defined to be "the region of the reactor vessel (shell material including welds, heat affected zones, and plates or forgings) that directly surrounds the effective height of the active core and adjacent regions of the reactor vessel that are predicted to experience sufficient neutron irradiation damage to be considered in the selection of the most limiting material with regard to radiation damage" [2].

3.2 Definition of Plant-Specific Material Properties

The pertinent chemical and mechanical properties of the beltline region shell forgings and girth weld materials of the Braidwood Unit 1 reactor vessel are given in table 1.

The "weight-percent copper"(Cu) and "weight-percent nickel"(Ni) material chemistry values and the initial RT_{NDT} (I) values shown in the table are the same as those used in the Braidwood Unit 1 pressurized thermal shock submittal [5] except as noted.

The chemistry factors and "margin" (M) terms that are also shown in table 1 were determined in accordance with the Regulatory Guide 1.99 Revision 2 methods described in section 2. The chemistry factors and margins are based upon material chemistry measurement information since credible surveillance data are not yet available for the Braidwood Unit 1 reactor vessel materials.

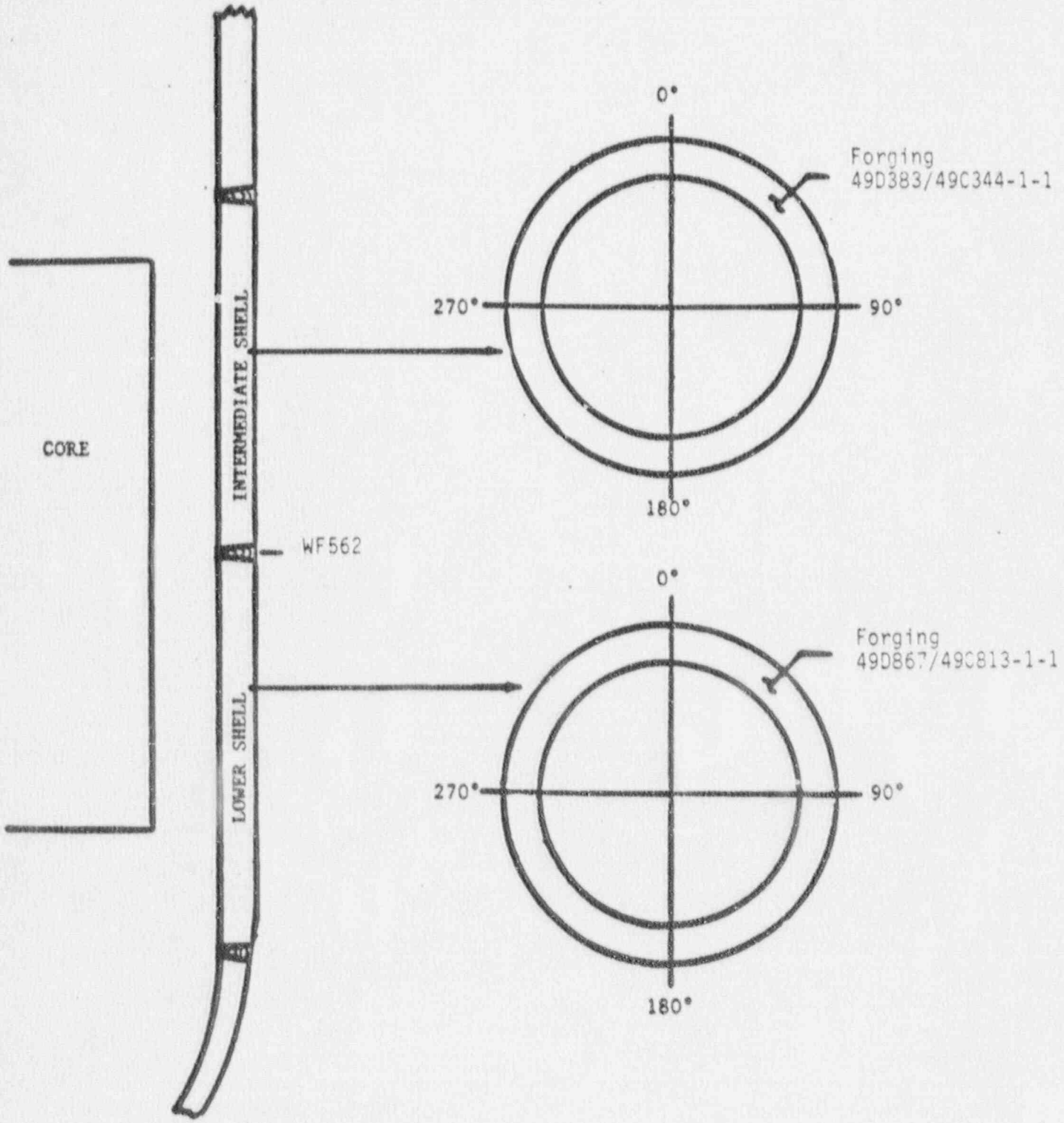


FIGURE 1
IDENTIFICATION AND LOCATION OF BELTLINE REGION MATERIAL
FOR THE BRAIDWOOD UNIT 1 REACTOR VESSEL

TABLE 1

BRAIDWOOD UNIT 1 REACTOR VESSEL BELTLINE REGION MATERIAL PROPERTIES

Material	Cu (Wt.%)	Ni (Wt.%)	CF	I[(a)] (°F)	M[(b)] (°F)
Intermediate Shell Forging 49D383/49C344-1-1	0.05	0.73	31	-30	34
Lower Shell Forging 49D867/49C813-1-1	0.03	0.73	20	-20	34
Circumferential Weld WF 562	0.04[(c)]	0.67[(c)]	54	40	56

(a) The initial RT_{NDT} (I) values for the forgings and weld are measured values.

(b) Maximum margin (M) as per Reg. Guide 1.99, rev. 2; the standard deviation for the initial RT_{NDT} margin term is assumed to be zero since the initial RT_{NDT} values were obtained from conservative (i.e., "upper bound") test results. Smaller values are obtained when $\sigma_{\Delta} = 1/2\ \Delta RT_{NDT}$.

(c) These values from Weld Certification Test Report WF562 (Appendix A) are higher than the values of 0.03% Cu and 0.67% Ni used in the PTS submittal [5].

Drywell Temperature

Time	Temperature	Justification
0700	129°F	Normal
0800	129°F	Temperature increase until ESW system is made available for drywell cooling
0802	135°F	Drywell cooling in-service
1200	115°F	

Drywell Pressure

Time	Pressure	Justification
0700	1.9 psig	Accident events have no effect on drywell pressure
1200	1.9 psig	

Torus Pressure

Time	Pressure	Justification
0700	-0.1 psig	Normal pressure
1200	0.1 psig	

Torus Water Temperature

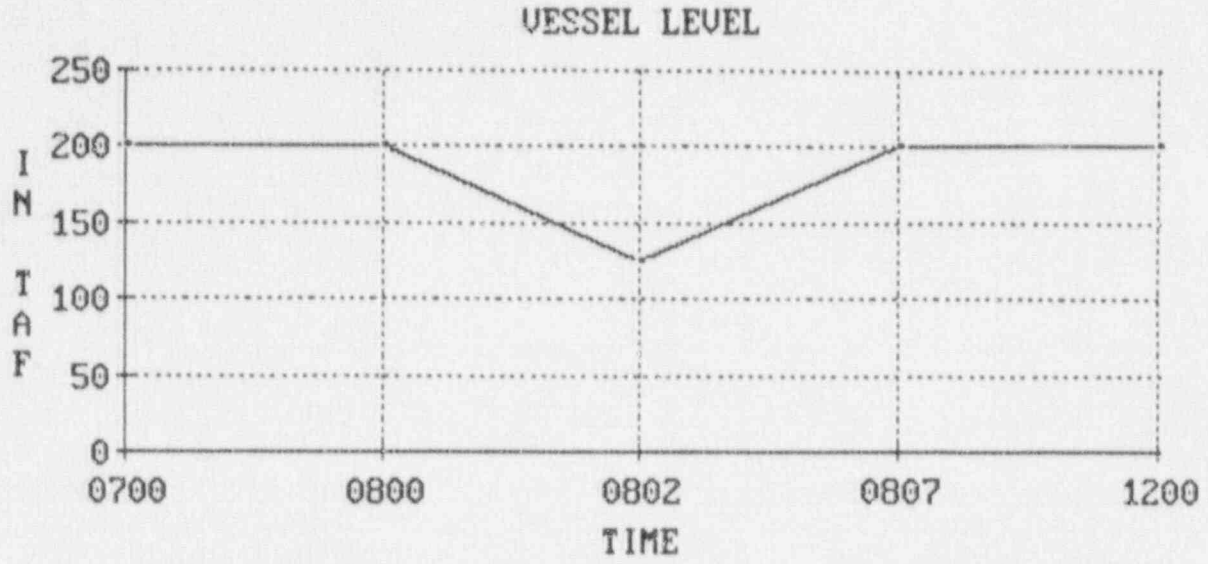
Time	Temperature	Justification
0700	76°F	Normal temperature
0802	76°F	HPCI/RCIC controlling vessel level and pressure
1200	82°F	

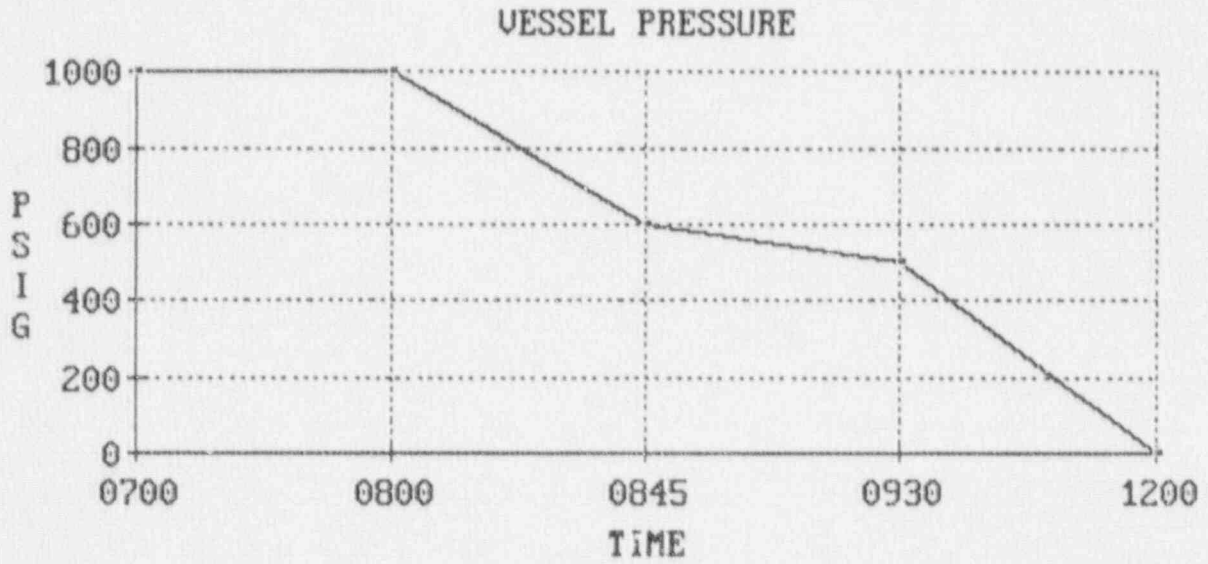
HREM
Turbine Building Exhaust

Time	High Range	Justification
0700	0 mR/hr	Normal HREM reading
0845	0 mR/hr	Break in main steam line drain pipe releasing activity in steam tunnel and Turbine Building
0900	0.1 mR/hr	Release through Turbine Building ventilation initiated
0915	0.25 mR/hr	Release through ventilation exhaust continues
1130	0.25 mR/hr	Activity in Turbine Building has diminished
1145	0 mR/hr	

Drywell Radiation Monitor

Time	Dose Rate	Justification
0700	8 R/hr	Normal reading at 100% power
0800	8 R/hr	Reactor scram
0802	2 R/hr	Normal reading
1200	1 R/hr	


VESSEL LEVEL
250
200
150
100
50
0
IN TAF
0700
0800
0802
0807
1200
TIME


VESSEL PRESSURE
1000
800
600
400
200
0
PSIG
0700
0800
0845
0930
1200
TIME

SECTION 3

James A. FitzPatrick Nuclear [illegible]er Plant
1990 Partial Scale Emergency [illegible]xercise

December 19, 1990

DOSE ASSESSMENT DISPLAYS

The following displays are examples of the type of MMRAS Computer Model outputs used to obtain dose projections. The outputs were obtained for a projected single phase steam line break at 15-minute intervals over the projected release duration.

On the day of the exercise, actual meteorological conditions will be used to determine the direction and concentration of the atmospheric release.

Continuous release, due to Steam Line break

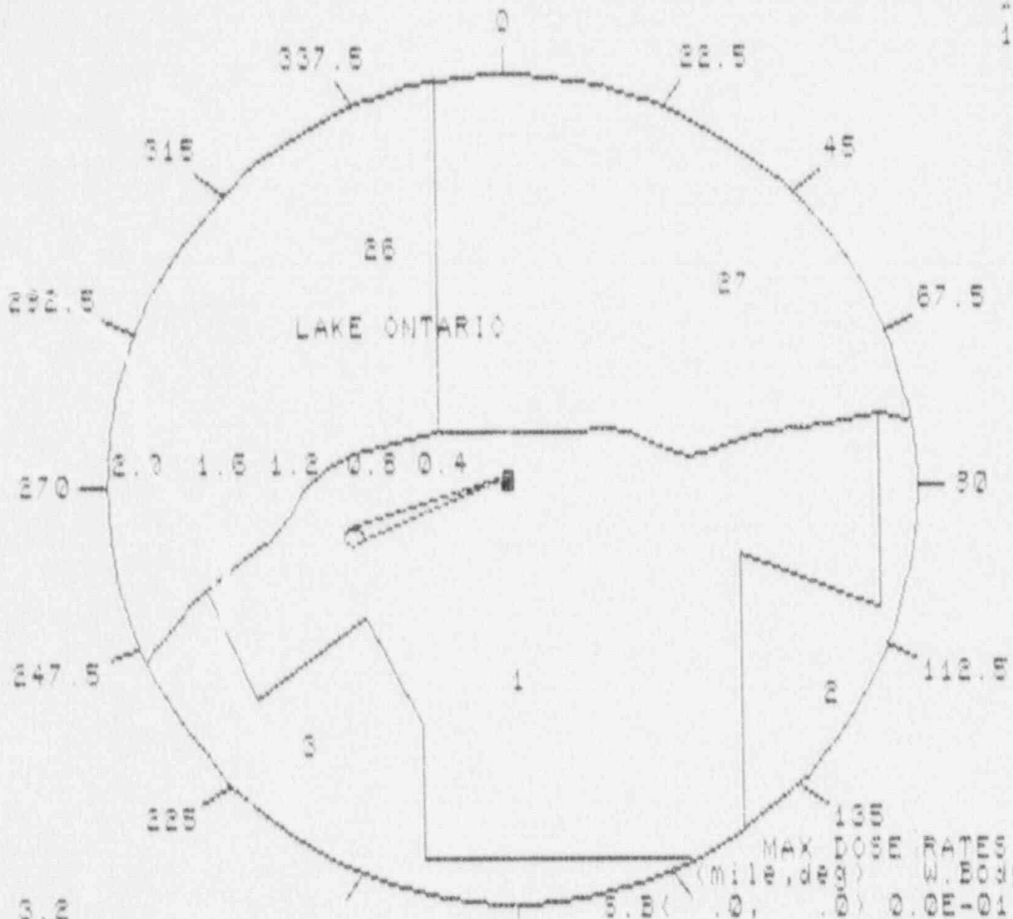
Isotope	Source Strength	Isotope	Source Strength (Ci/sec)
KR-83M	0.000E-01	MN-54	0.000E-01
KR-85M	2.652E-01	CO-58	0.000E-01
KR-87	5.188E-01	CO-60	0.000E-01
KR-88	7.288E-01	FE-59	0.000E-01
XE-133	2.173E+00	ZN-65	0.000E-01
XE-135M	0.000E-01	SR-89	0.000E-01
XE-135	2.812E-01	SR-90	0.000E-01
XE-138	0.000E-01	SB-125	0.000E-01
I-131	3.200E-03	TE-132	0.000E-01
I-132	5.200E-03	CS-134	0.000E-01
I-133	8.400E-03	CS-136	0.000E-01
I-134	9.200E-03	CS-137	0.000E-01
I-135	7.200E-03	BA-140	0.000E-01
H-3	0.000E-01	LA-140	0.000E-01

Total release is 4.000E+00 Ci/sec

Do you wish to change total source strength value (Y/N)? ■

PLUME DISPLAY (REAL TIME)

Q- 0:15
11-OCT-90
12: 4



WS (mph) 3.2
WD (deg) 63.2
Sv A Sh G

MAX DOSE RATES (mREM/HR)

(mile,deg)	W.Body	C.Thy.
5.8(.0, .0)	0.0E-01	0.0E-01
2MI(.0, .0)	0.0E-01	0.0E-01
5MI(.0, .0)	0.0E-01	0.0E-01
10MI(.0, .0)	0.0E-01	0.0E-01

OFF-SITE RECEPTORS (REAL TIME) 0- 0:15
11-OCT-90

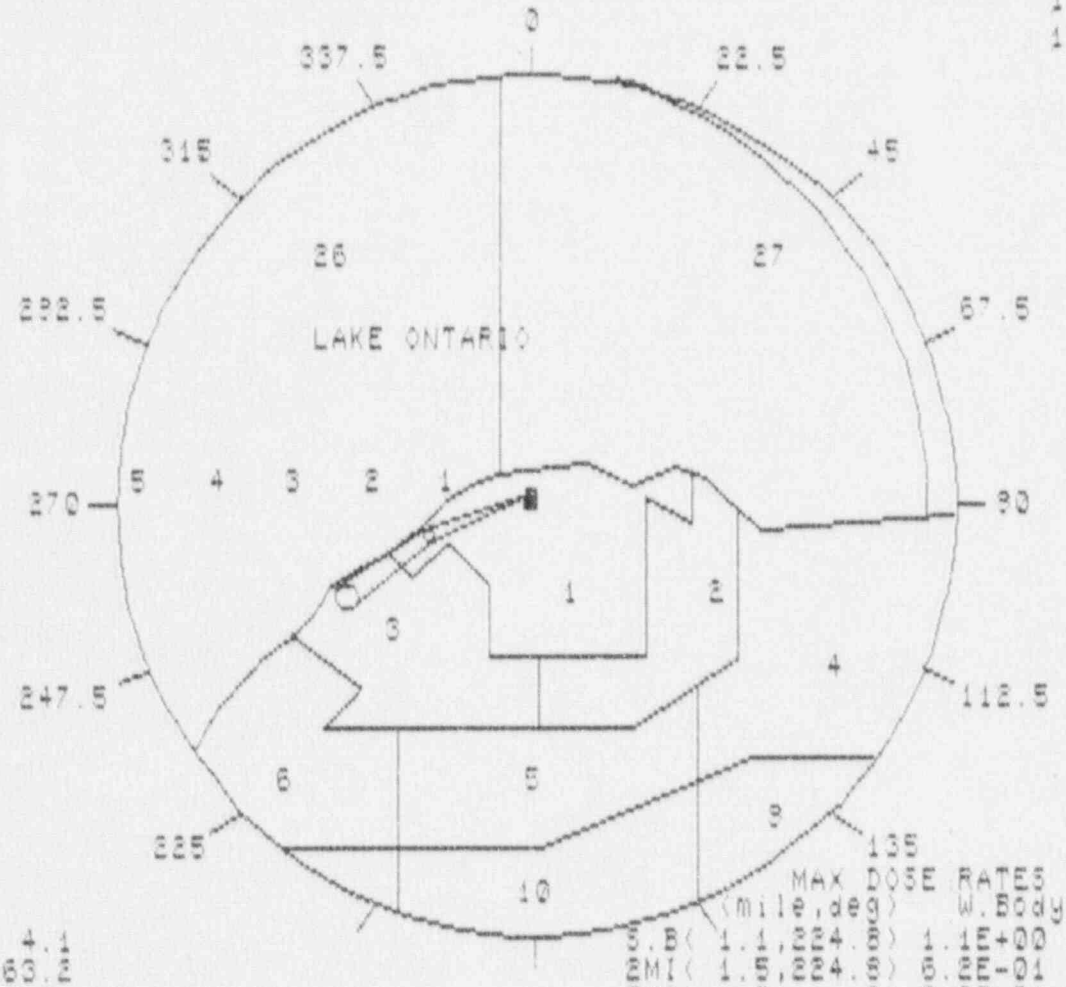
ERPA	Receptor		Dose Rate (mR/hr)		Cumulative Dose (mREM)		
	Distance (Miles)	Direction (Degree)	Gamma	Skin	Whole Body	Skin	Child Thyroid
	---------	---------	-----	----	-----	----	---------

THE GAMMA DOSE RATE IS ACCURATE WITHIN PLUS OR MINUS 100.0 PERCENT

ENTER (1=page one, LINE FEED=page forward, B=page back, M=menu) :

PLUME DISPLAY (REAL TIME)

0- 0:30
11-OCT-90
12:22



WS (mph) 4.1
WD (deg) 83.2
Sv A Sk G

Max DOSE RATES (mREM/Hr)

	(mile,deg)	W.Body	C.Thy.
5.8(	1.1,224.8)	1.1E+00	1.0E+01
2MI(	1.5,224.8)	5.2E-01	5.2E+00
5MI(	.0, .0)	0.0E-01	0.0E-01
10MI(	.0, .0)	0.0E-01	0.0E-01

OFF-SITE RECEPTORS (REAL TIME) 0- 0:30
11-OCT-90

ERPA	Receptor		Dose Rate (mR/hr)		Cumulative Dose (mREM)		
	Distance (Miles)	Direction (Degree)	Gamma	Skin	Whole Body	Skin	Child Thyroid
2	1	1.5	222.6	.66	1.1	.16	.28	.70
4	1	.8	221.3	1.5	2.5	.38	.62	1.2

THE GAMMA DOSE RATE IS ACCURATE WITHIN PLUS OR MINUS 100.0 PERCENT

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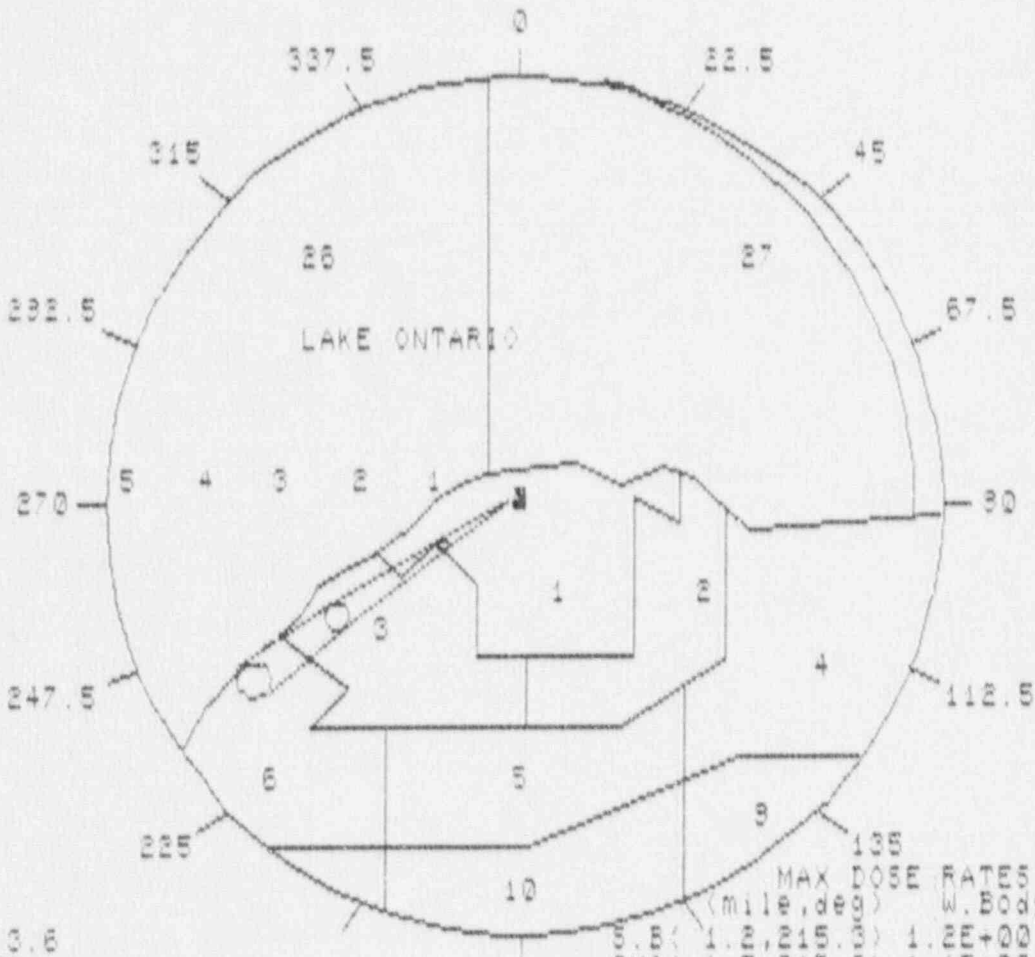

WS (mph) 3.6
WD (deg) 50.1
Sv A Sh 3

MAX DOSE RATES (mREM/Hr)

	(mile,deg)	W.Body	C.Thy.
5.8	(1.2,215.3)	1.2E+00	7.9E+00
2MI	(1.5,217.8)	1.1E+00	6.8E+00
5MI	(.0, .0)	0.0E-01	0.0E-01
10MI	(.0, .0)	0.0E-01	0.0E-01

OFF-SITE RECEPTORS (REAL TIME) 0- 0:45
11-OCT-90

ERPA	Receptor		Dose Rate (mR/hr)		Cumulative Dose (mREM)		
	Distance (Miles)	Direction (Degree)	Gamma	Skin	Whole Body	Skin	Child Thyroid
2	1	1.5	222.6	.92	1.3	.40	.60	.72
4	1	.8	221.3	0.00E-01	0.00E-01	.38	.62	1.2
5	1	1.0	206.6	1.2	1.7	.31	.43	4.45E-02

THE GAMMA DOSE RATE IS ACCURATE WITHIN PLUS OR MINUS 100.0 PERCENT

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PLUME DISPLAY (REAL TIME)

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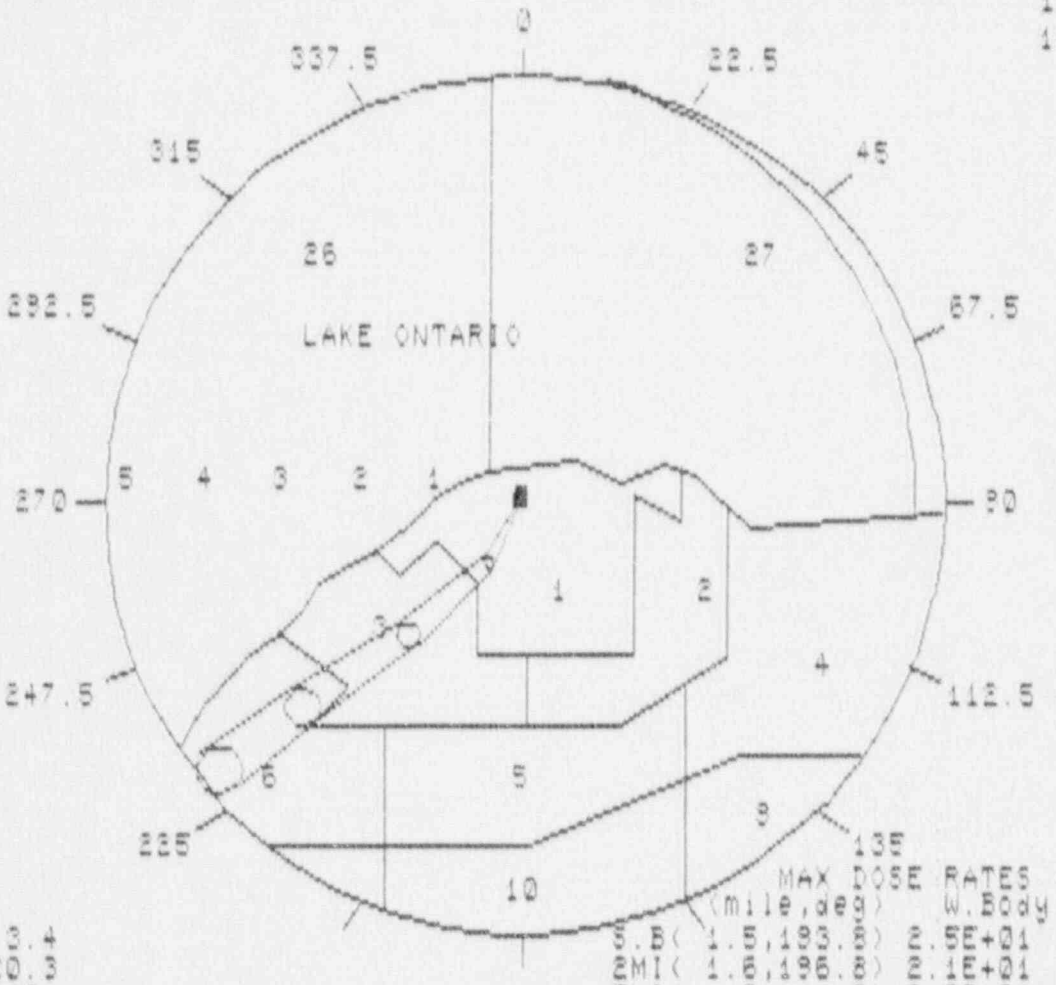
12:52



WS (mph) 3.4
WD (deg) 80.3
Sv E Sh G

	MAX DOSE RATES (mREM/Hr) (mile,deg)	W.Body	C.Thy.
S.B	(1.5,193.8)	2.5E+01	5.3E+02
2MI	(1.8,196.8)	2.1E+01	4.0E+02
5MI	(.0, .0)	0.0E-01	0.0E-01
10MI	(.0, .0)	0.0E-01	0.0E-01

OFF-SITE RECEPTORS (REAL TIME) 0- 1: 0
11-OCT-90

ERPA	Receptor		Dose Rate (mR/hr)		Cumulative Dose (mREM)		
	Distance (Miles)	Direction (Degree)	Gamma	Skin	Whole Body	Skin	Child Thyroid
2	1	1.5	222.6	0.00E-01	0.00E-01	.40	.60	.72
3	1	2.0	201.2	14.	36.	3.4	9.1	59.
4	1	.8	221.3	0.00E-01	0.00E-01	.38	.62	1.2
5	1	1.0	206.6	0.00E-01	0.00E-01	.31	.43	4.45E-02
7	1	1.0	173.6	5.5	8.0	1.4	2.0	2.0
24	3	3.0	209.5	5.7	17.	1.4	4.2	33.
25	3	2.9	208.2	3.1	8.4	.77	2.1	16.
27	3	1.9	205.2	3.4	6.7	.85	1.7	7.4

THE GAMMA DOSE RATE IS ACCURATE WITHIN PLUS OR MINUS 100.0 PERCENT

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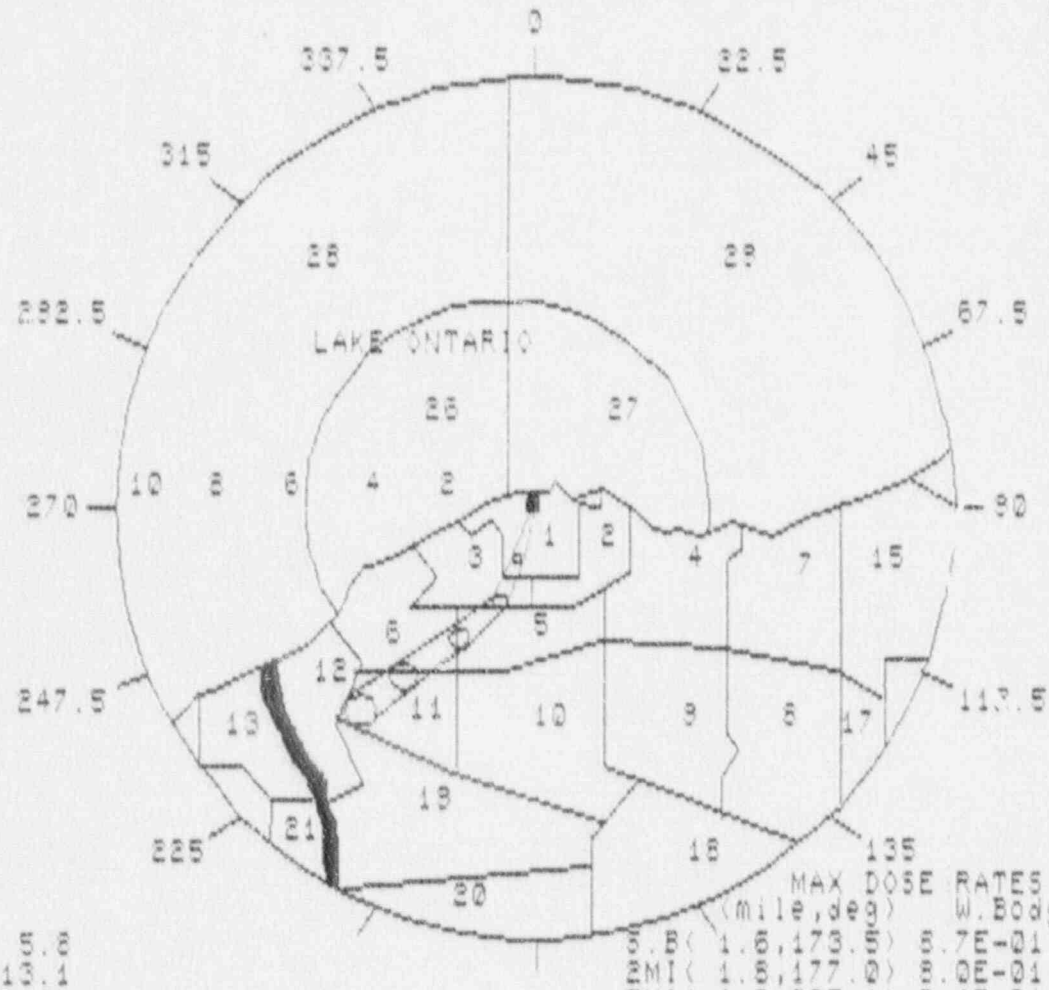
PLUME DISPLAY (REAL TIME)

0- 1:15
11-OCT-90
13: 7



	MAX DOSE RATES (mREM/Hr)		
	(mile,deg)	W.Body	C.Thy.
5.8	(1.6,173.5)	8.7E-01	9.3E+00
2MI	(1.6,177.0)	8.0E-01	8.2E+00
5MI	(4.6,205.1)	3.4E-01	3.1E+00
10MI	(.0, .0)	0.0E-01	0.0E-01

WS (mph) 5.8
WD (deg) 13.1
Sv A Sh G

OFF-SITE RECEPTORS (REAL TIME) 0- 1:15
11-OCT-90

ERPA	Receptor		Dose Rate (mR/hr)		Cumulative Dose (mREM)		
	Distance (Miles)	Direction (Degree)	Gamma	Skin	Whole Body	Skin	Child Thyroid
2	1	1.5	222.6	0.00E-01	0.00E-01	.40	.60	.72
3	1	2.0	201.2	0.00E-01	0.00E-01	3.4	9.1	59.
4	1	.8	221.3	0.00E-01	0.00E-01	.38	.62	1.2
5	1	1.0	206.6	0.00E-01	0.00E-01	.31	.43	4.45E-02
7	1	1.0	173.6	0.00E-01	0.00E-01	1.4	2.0	2.0
24	3	3.0	209.5	0.00E-01	0.00E-01	1.4	4.2	33.
25	3	2.9	208.2	0.00E-01	0.00E-01	.77	2.1	16.
27	3	1.9	205.2	0.00E-01	0.00E-01	.85	1.7	7.4
30	3	2.6	184.9	.52	.81	.13	.20	.33
31	3	2.0	182.3	.67	.97	.17	.24	.17

ENTER (1=page one, LINE FEED=page forward, B=page back, M=menu) :

OFF-SITE RECEPTORS (REAL TIME) 0- 1:15
11-OCT-90

ERPA	Receptor		Dose Rate (mR/hr)		Cumulative Dose (mREM)		
	Distance (Miles)	Direction (Degree)	Gamma	Skin	Whole Body	Skin	Child Thyroid
88	11	4.9	210.5	.15	.21	3.72E-02	5.37E-02	6.23E-03

THE GAMMA DOSE RATE IS ACCURATE WITHIN PLUS OR MINUS 100.0 PERCENT

ENTER (1=page one, LINE FEED=page forward, B=page back, M=menu) :

PLUME DISPLAY (REAL TIME)

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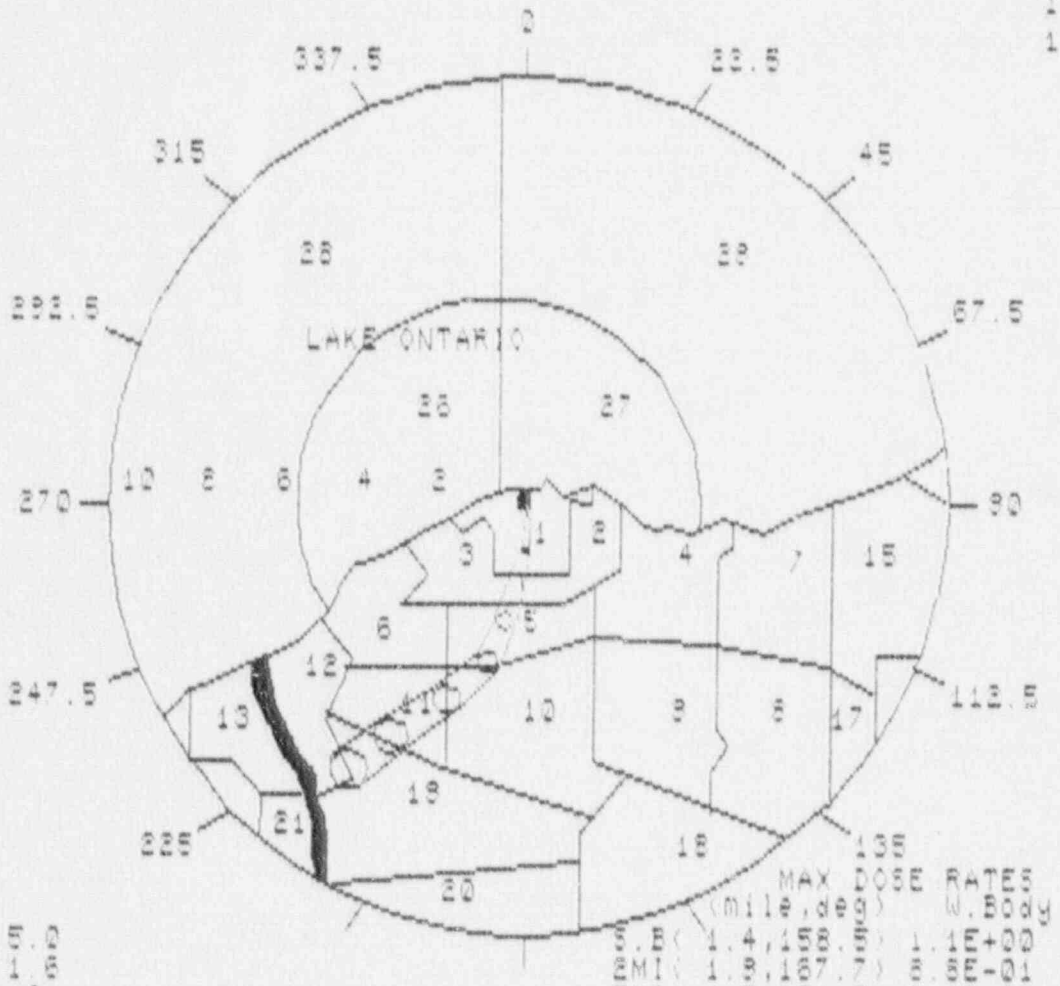
11-OCT-90
13:22



WS (mph) 5.0
WD (deg) 1.6
St A Sh S

MAX DOSE RATES (mREM/Hr)

	(mile,deg)	W.Body	C.Thy.
S.B(	1.4,158.5)	1.1E+00	7.0E+00
2MI(	1.9,167.7)	3.8E-01	4.7E+00
5MI(	4.7,121.7)	5.4E-01	3.2E+00
10MI(	.0, .0)	0.0E-01	0.0E-01

OFF-SITE RECEPTORS (REAL TIME) 0- 1:30
11-OCT-90

ERPA	Receptor		Dose Rate (mR/hr)		Cumulative Dose (mREM)		
	Distance (Miles)	Direction (Degree)	Gamma	Skin	Whole Body	Skin	Child Thyroid
2	1	1.5	222.6	0.00E-01	0.00E-01	.40	.60	.72
3	1	2.0	201.2	0.00E-01	0.00E-01	3.4	9.1	59.
4	1	.8	221.3	0.00E-01	0.00E-01	.38	.62	1.2
5	1	1.0	206.6	0.00E-01	0.00E-01	.31	.43	4.45E-02
7	1	1.0	173.6	0.00E-01	0.00E-01	1.4	2.0	2.0
8	1	1.7	169.0	.79	1.1	.20	.28	4.70E-02
9	1	1.1	153.7	2.3	3.4	.58	.85	.53
24	3	3.0	209.5	0.00E-01	0.00E-01	1.4	4.2	33.
25	3	2.9	208.2	0.00E-01	0.00E-01	.77	2.1	16.

ENTER (1=page one, LINE FEED=page forward, B=page back, M=menu) : ■

OFF-SITE RECEPTORS (REAL TIME) 0- 1:30
11-OCT-90

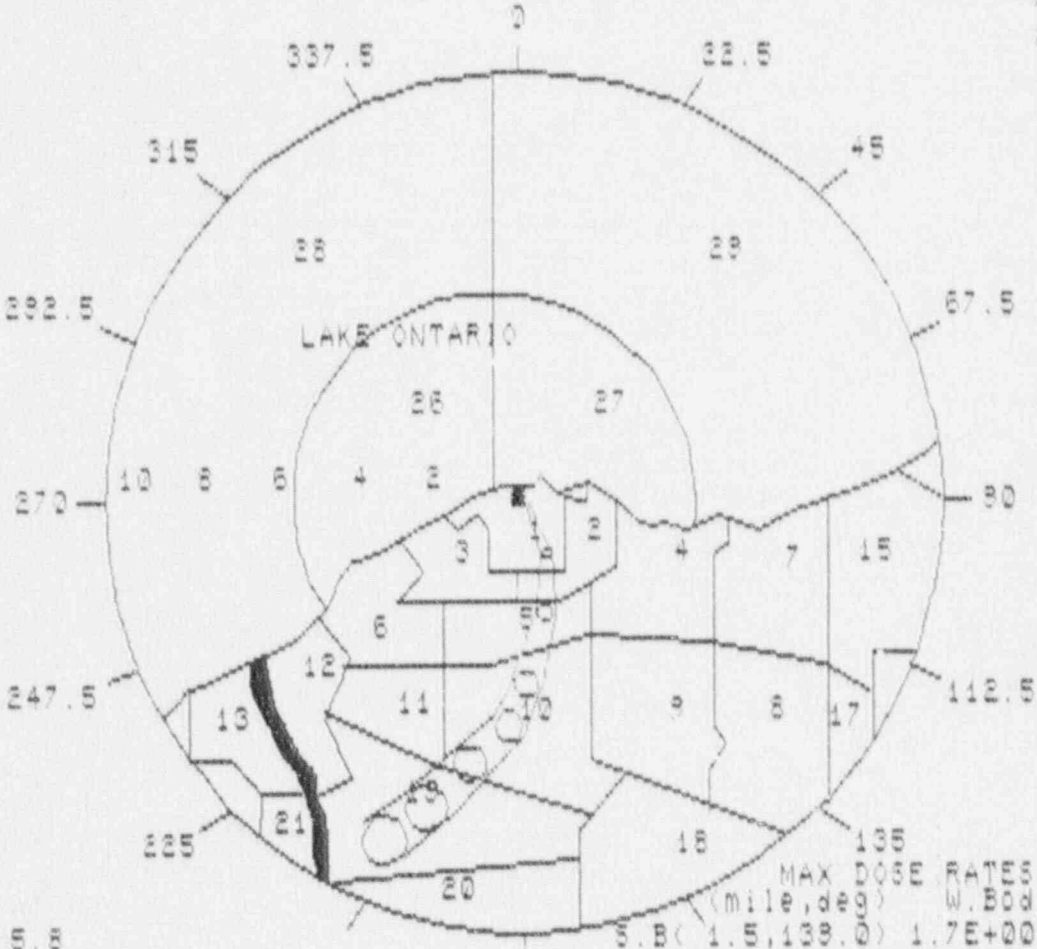
ERPA	Receptor		Dose Rate (mR/hr)		Cumulative Dose (mREM)		
	Distance (Miles)	Direction (Degree)	Gamma	Skin	Whole Body	Skin	Child Thyroid
27	3	1.9	205.2	0.00E-01	0.00E-01	.85	1.7	7.4
28	3	3.9	188.9	.29	.41	7.28E-02	.10	1.07E-02
30	3	2.6	184.9	0.00E-01	0.00E-01	.13	.20	.33
31	3	2.0	182.3	0.00E-01	0.00E-01	.17	.24	.17
45	5	4.0	186.8	.35	.63	8.74E-02	.16	.54
47	5	4.0	183.2	.34	.52	8.43E-02	.13	.19
87	11	6.2	204.0	9.88E-02	.15	2.47E-02	3.68E-02	2.56E-02
88	11	4.9	210.5	0.00E-01	0.00E-01	3.72E-02	5.37E-02	6.23E-03
89	11	6.0	204.1	.10	.15	2.59E-02	3.75E-02	6.56E-03

THE GAMMA DOSE RATE IS ACCURATE WITHIN PLUS OR MINUS 100.0 PERCENT

ENTER (1=page one, LINE FEED=page forward, B=page back, M=menu) : ■

PLUME DISPLAY (REAL TIME)

2- 1:45
11-OCT-90
13:37



MAX DOSE RATES (mREM/HR)

	(mile,deg)	W.Body	C.Thy.
S.B	(1.5,138.0)	1.7E+00	1.0E+01
2MI	(2.1,151.0)	8.1E-01	5.4E+00
5MI	(4.8,174.8)	2.8E-01	3.1E+00
10MI	(.0, .0)	0.0E-01	0.0E-01

WS (mph) 5.8
WD (deg) 342.8
St A Sh G

OFF-SITE RECEPTORS (REAL TIME) 0- 1:45
11-OCT-90

ERPA	Receptor		Dose Rate (mR/hr)		Cumulative Dose (mREM)		
	Distance (Miles)	Direction (Degree)	Gamma	Skin	Whole Body	Skin	Child Thyroid
2	1	1.5	222.6	0.00E-01	0.00E-01	.40	.60	.72
3	1	2.0	201.2	0.00E-01	0.00E-01	3.4	9.1	59.
4	1	.8	221.3	0.00E-01	0.00E-01	.38	.62	1.2
5	1	1.0	206.6	0.00E-01	0.00E-01	.31	.43	4.45E-02
7	1	1.0	173.6	0.00E-01	0.00E-01	1.4	2.0	2.0
8	1	1.7	169.0	0.00E-01	0.00E-01	.20	.28	4.70E-02
9	1	1.1	153.7	0.00E-01	0.00E-01	.58	.85	.53
10	1	1.0	128.8	5.3	9.4	1.3	2.4	6.0
11	1	1.2	135.2	2.8	4.5	.70	1.1	1.7

ENTER (1=page one, LINE FEED=page forward, B=page back, M=menu) :

OFF-SITE RECEPTORS (REAL TIME) 0- 1:45
11-OCT-90

ERPA	Receptor		Dose Rate (mR/hr)		Cumulative Dose (mREM)		
	Distance (Miles)	Direction (Degree)	Gamma	Skin	Whole Body	Skin	Child Thyroid
12	1	2.0	151.6	.77	1.2	.19	.31	.59
17	2	1.8	151.9	.68	.95	.17	.24	5.66E-04
24	3	3.0	209.5	0.00E-01	0.00E-01	1.4	4.2	33.
25	3	2.9	208.2	0.00E-01	0.00E-01	.77	2.1	16.
27	3	1.9	205.2	0.00E-01	0.00E-01	.85	1.7	7.4
28	3	3.9	188.9	0.00E-01	0.00E-01	7.28E-02	.10	1.07E-02
30	3	2.6	184.9	0.00E-01	0.00E-01	.13	.20	.33
31	3	2.0	182.3	0.00E-01	0.00E-01	.17	.24	.17
45	5	4.0	186.8	0.00E-01	0.00E-01	8.74E-02	.16	.54
47	5	4.0	183.2	0.00E-01	0.00E-01	8.43E-02	.13	.19

ENTER (1=page one, LINE FEED=page forward, B=page back, M=menu) : ■

OFF-SITE RECEPTORS (REAL TIME) 0- 1:45
11-OCT-90

ERPA	Receptor		Dose Rate (mR/hr)		Cumulative Dose (mREM)		
	Distance (Miles)	Direction (Degree)	Gamma	Skin	Whole Body	Skin	Child Thyroid
48	5	6.2	181.4	.12	.18	3.01E-02	4.44E-02	1.96E-02
49	5	2.7	162.6	.59	.82	.15	.20	1.86E-02
50	5	3.8	166.6	.39	.59	9.63E-02	.15	.18
51	5	3.1	163.3	.52	.93	.13	.23	.71
52	5	3.6	164.7	.40	.58	9.31E-02	.14	8.97E-02
59	6	3.7	168.4	.43	.75	.11	.19	.54
79	10	5.8	180.2	.15	.25	3.64E-02	6.14E-02	.15
87	11	6.2	204.0	0.00E-01	0.00E-01	2.47E-02	3.68E-02	2.56E-02
88	11	4.9	210.5	0.00E-01	0.00E-01	3.72E-02	5.37E-02	6.23E-03
89	11	6.0	204.1	0.00E-01	0.00E-01	2.59E-02	3.75E-02	6.56E-03

ENTER (1=page one, LINE FEED=page forward, B=page back, M=menu) : _

OFF-SITE RECEPTORS (REAL TIME) 0- 1:45
11-OCT-90

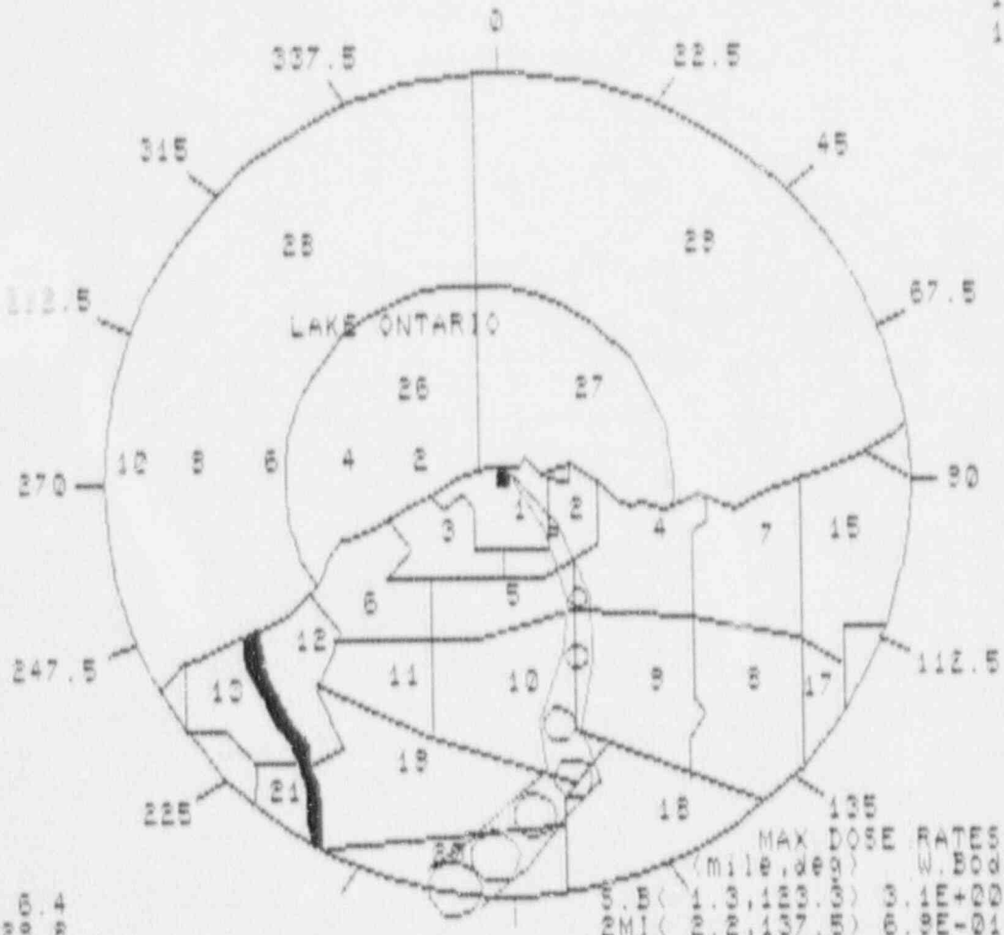
ERPA	Receptor		Dose Rate (mR/hr)		Cumulative Dose (mREM)		
	Distance (Miles)	Direction (Degree)	Gamma	Skin	Whole Body	Skin	Child Thyroid
91	11	6.0	188.3	.10	.15	2.59E-02	3.74E-02	3.58E-03
157	19	7.0	188.7	.13	.28	3.23E-02	6.92E-02	.43

THE GAMMA DOSE RATE IS ACCURATE WITHIN PLUS OR MINUS 100.0 PERCENT

ENTER (1=page one, LINE FEED=page forward, B=page back, M=menu) : ■

PLUME DISPLAY (REAL TIME)

0- 2: 0
11-OCT-90
13:52



WS (mph) 6.4
WD (deg) 329.8
St A Sh G

MAX DOSE RATES (mREM/Hr)

	(mile,deg)	W.Body	C.Thy.
5.8	(1.3,122.3)	3.1E+00	2.6E+01
2MI	(2.2,137.5)	6.8E-01	4.6E+00
5MI	(5.0,160.7)	8.4E-01	1.8E+00
10MI	(.0, .0)	0.0E-01	0.0E-01

OFF-SITE RECEPTORS (REAL TIME) 0- 2: 0
11-OCT-90

ERPA	Receptor		Dose Rate (mR/hr)		Cumulative Dose (mREM)		
	Distance (Miles)	Direction (Degree)	Gamma	Skin	Whole Body	Skin	Child Thyroid
2	1	1.5	222.6	0.00E-01	0.00E-01	.40	.60	.72
3	1	2.0	201.2	0.00E-01	0.00E-01	3.4	9.1	59.
4	1	.8	221.3	0.00E-01	0.00E-01	.38	.62	1.2
5	1	1.0	206.6	0.00E-01	0.00E-01	.31	.43	4.45E-02
7	1	1.0	173.6	0.00E-01	0.00E-01	1.4	2.0	2.0
8	1	1.7	169.0	0.00E-01	0.00E-01	.20	.28	4.70E-02
9	1	1.1	153.7	0.00E-01	0.00E-01	.58	.85	.53
10	1	1.0	128.8	0.00E-01	0.00E-01	1.3	2.4	6.0
11	1	1.2	135.2	0.00E-01	0.00E-01	.70	1.1	1.7

ENTER (1=page one, LINE FEED=page forward, B=page back, M=menu) : ■

OFF-SITE RECEPTORS (REAL TIME) 0- 2: 0
11-OCT-90

ERPA	Receptor		Dose Rate (mR/hr)		Cumulative Dose (mREM)		
	Distance (Miles)	Direction (Degree)	Gamma	Skin	Whole Body	Skin	Child Thyroid
12	1	2.0	151.6	0.00E-01	0.00E-01	.19	.31	39
13	1	1.3	117.5	2.7	3.8	.67	.94	8.18E-06
15	1	2.2	140.2	.60	.87	.15	.22	.11
17	2	1.8	151.9	0.00E-01	0.00E-01	.17	.24	5.66E-04
24	3	3.0	209.5	0.00E-01	0.00E-01	1.4	4.2	33.
25	3	2.9	208.2	0.00E-01	0.00E-01	.77	2.1	16.
27	3	1.9	205.2	0.00E-01	0.00E-01	.85	1.7	7.4
28	3	3.9	188.9	0.00E-01	0.00E-01	7.28E-02	.10	1.07E-02
30	3	2.6	184.9	0.00E-01	0.00E-01	.13	.20	.33
31	3	2.0	182.3	0.00E-01	0.00E-01	.17	.24	.17

ENTER (1=page one, LINE FEED=page forward, B=page back, M=menu) : ■

OFF-SITE RECEPTORS (REAL TIME) 0- 2: 0
11-OCT-90

ERPA	Receptor		Dose Rate (mR/hr)		Cumulative Dose (mREM)		
	Distance (Miles)	Direction (Degree)	Gamma	Skin	Whole Body	Skin	Child Thyroid
45	5	4.0	186.8	0.00E-01	0.00E-01	8.74E-02	.16	.54
47	5	4.0	183.2	0.00E-01	0.00E-01	8.43E-02	.13	.19
48	5	6.2	181.4	0.00E-01	0.00E-01	3.01E-02	4.44E-02	1.96E-02
49	5	2.7	162.6	0.00E-01	0.00E-01	.15	.20	1.86E-02
50	5	3.8	166.6	0.00E-01	0.00E-01	9.63E-02	.15	.18
51	5	3.1	163.3	0.00E-01	0.00E-01	.13	.23	.71
52	5	3.6	164.7	0.00E-01	0.00E-01	9.91E-02	.14	8.97E-02
53	5	2.8	143.8	.57	1.0	.14	.26	.78
59	6	3.7	168.4	0.00E-01	0.00E-01	.11	.19	.54
79	10	5.8	180.2	0.00E-01	0.00E-01	3.64E-02	6.14E-02	.15

ENTER (1=page one, LINE FEED=page forward, B=page back, M=menu) :

OFF-SITE RECEPTORS (REAL TIME) 0- 2: 0
11-OCT-90

ERPA	Receptor		Dose Rate (mR/hr)		Cumulative Dose (mREM)		
	Distance (Miles)	Direction (Degree)	Gamma	Skin	Whole Body	Skin	Child Thyroid
82	10	5.1	165.4	.16	.23	3.98E-02	5.71E-02	4.38E-03
83	10	4.9	164.0	.17	.25	4.37E-02	6.26E-02	6.70E-03
84	10	6.1	166.0	.13	.31	3.13E-02	7.76E-02	.54
85	10	7.2	165.8	7.95E-02	.18	1.99E-02	4.55E-02	.30
87	11	6.2	204.0	0.00E-01	0.00E-01	2.47E-02	3.68E-02	2.56E-02
88	11	4.9	210.5	0.00E-01	0.00E-01	3.72E-02	5.37E-02	6.23E-03
89	11	6.0	204.1	0.00E-01	0.00E-01	2.59E-02	3.75E-02	6.56E-03
91	11	6.0	188.3	0.00E-01	0.00E-01	2.59E-02	3.74E-02	3.58E-03
157	19	7.0	188.7	0.00E-01	0.00E-01	3.23E-02	6.92E-02	.43
158	19	8.2	179.6	5.85E-02	8.52E-02	1.46E-02	2.13E-02	1.19E-02

ENTER (1=page one, LINE FEED=page forward, B=page back, M=menu) : _

OFF-SITE RECEPTORS (REAL TIME) 0- 2: 0
11-OCT-90

ERPA	Receptor		Dose Rate (mR/hr)		Cumulative Dose (mREM)		
	Distance (Miles)	Direction (Degree)	Gamma	Skin	Whole Body	Skin	Child Thyroid
159	19	7.1	166.4	.12	.28	2.92E-02	7.07E-02 .50

THE GAMMA DOSE RATE IS ACCURATE WITHIN PLUS OR MINUS 100.0 PERCENT

ENTER (1=page one, LINE FEED=page forward, B=page back, M=menu) : ▮

ON-SITE RECEPTORS (REAL TIME) 0- 2: 0
11-OCT-90

ERPA	Receptor		Dose Rate (mR/hr)		Cumulative Dose (mREM)		
	Distance (Miles)	Direction (Degree)	Gamma	Skin	Whole Body	Skin	Child Thyroid
1	1	.5	197.6	0.00E-01	0.00E-01	.88	1.8	7.1
2	1	.6	209.2	0.00E-01	0.00E-01	.66	.93	9.70E-02
3	1	.6	224.4	0.00E-01	0.00E-01	.65	1.4	6.3
4	1	.7	232.8	0.00E-01	0.00E-01	.45	.62	1.63E-02
18	1	1.2	136.2	0.00E-01	0.00E-01	.65	.95	.47

THE GAMMA DOSE RATE IS ACCURATE WITHIN PLUS OR MINUS 100.0 PERCENT

ENTER (1=page one, LINE FEED=page forward, B=page back, M=menu) :
NO MORE DATA IN TABLES

SECTION 3

James A. FitzPatrick Nuclear Power Plant
1990 Partial Scale Emergency Exercise

December 19, 1990

METEOROLOGICAL FORECAST

The attached meteorological forecast information should be completed for the exercise. Actual meteorological forecasting should be performed.

SECTION 3

James A. FitzPatrick Nuclear Power Plant
1990 Partial Scale Emergency Exercise

December 19, 1990

WEATHER DESCRIPTION AND FORECAST

CURRENT DATE:
CURRENT TIME:

FORECAST LEVEL (FEET):
FORECASTER'S INITIALS:

GENERAL WEATHER DESCRIPTION:

FORECAST TIME (LOCAL)	WIND SPEED (MPH)	WIND DESCRIPTION (FROM)	STABILITY CLASS	TEMPERATURE °F	PRECIPITATION (TYPE, QUANTITY)

SECTION 4

James A. FitzPatrick Nuclear Power Plant
1990 Partial Scale Emergency Exercise

December 19, 1990

MEETINGS

SECTION 4

James A. FitzPatrick Nuclear Power Plant
1990 Partial Scale Emergency Exercise

December 19, 1990

MEETINGS

December 18, 1990

1300 to 1500	NYPA, State, NRC Controller/Observer Briefing (Training Building)
1500 to 1700	NRC Training and Badging (Training Building)

December 19, 1990

0700 to 1230	NRC Observed Partial Scale Emergency Exercise

December 20, 1990

0900 to 1030	Observer Debrief (Training Building - Classroom 3)
1030 to 1130	NYPA Exercise Critique (Training Building - Classroom 3)
1145 to 1215	NRC Exercise Critique (Training Building - Classroom 3)

SECTION 5

James A. FitzPatrick Nuclear Power Plant
1990 Partial Scale Emergency Exercise

December 19, 1990

TIMELINE

SECTION 6

James A. FitzPatrick Nuclear Power Plant
1990 Partial Scale Emergency Exercise

December 19, 1990

EXERCISE/CONTINGENCY MESSAGES

I. Controller Sheets

II. Message Sheets

III. Plant SPDS and Parameter Sheets (optional distribution) } Sequentially

FORM SAP 1.3

CONTROLLER SHEET

TIME	MESSAGE NUMBER	ISSUED TO	SUMMARY OF MESSAGE CONTENTS	ANTICIPATED RESULTS AND COMMENTS	EMERGENCY CLASSIFICATION
0630	N/A	Initial Plant Briefing Partici-pants	See attached Daily Schedule		

DAILY SCHEDULE

DATE December 19, 1990

ACTIVITY NO.	DEPT.	DESCRIPTION	START	IN PROG.
ISP-30-1	JIC	Exo-Sensor H2/O2 Ana[illegible]zer Cal		
ISP-3-4		Reactor Level Instrumentation		
ISP-3-5		Reactor Level Monitoring Instrumentation		
ISP-23-1		ESW Inst Cals		
PM		Outage PM Planning		
PM		42TS-112L Caustic Disch Lo Temp Sw		
PM		42LT-102 Caustic Storage Tank Level Xmitter		
PM		Check Gas		
PM		Walk-Thru of New Warehouse w/Security Eng		
99/81645		SAS Door Latch Inop		
/65227		Refuel Bridge Headset		
00/71682		Refuel Floor Remote Camera System		
/65409		70868, CI-113, 112C Waste Demon Out Cond		
76/74930		FI-103 New Fire Pump Test Flow Ind		
76/73541		SD-430 Zone 79 Smoke Detector		
07/80837		81194, LPRM 28-21B, 28-05A		
13/80077		MTU-284 RCIC Steam Flow Input Cap		
/65111		RM-463A RW Bldg High Range Rad Monitor		
/73602		Ann HV-5A-12 Cont Rm Fan 4A No Flow		
39/500859	JMD-ME	Run in Packing on B A-C		
42/80755		Repair Pinhole Leak on 4" Line Disch of 42P-11A&B		
76/73658		FPS-314 Packing Leak		
76/73042		Check Valve Downstream of FPS-313 Stuck Open		
46/74101		Replace 46SWS-707		
72/74891		72CR-11 Cond Unit Clean & Paint		
72/74892		72CR-18 Cond Unit Clean & Paint		
71/71437	JMD-EL	71UPS-5 - Replace Battery #8		
71/73311		71UPS-5 - Clean All Cells		
71/10676		17P-36B Greases Bearings - Rx Build Vent Monitor "B" Sample Pump Motor		
71/9210		MCC-263-0C4 PM Controller for 72AHU-30B		
75/81252		75P-5A(M) Check Out Level Switch on Pump in So. Tank Pit - Condensate Tank "A"		
71/501140		Switchyard - Weekly		
88/80529		EOD-32 Turb Bldg Door Won't Open		
71/75295		Heat Trace		
	JOPS			
	JCS	Warehouse Security		
		Locker Room Rehab		
		Remove Scaffolding in Crescent		

Plant Status:

- 100% power
- 115KV Line #4 out of service due to maintenance being done by Niagara Mohawk. Estimated return to service at 0800 tomorrow.
- ST-1D "MSIVS and MSLDV and RWSV LFT" and ST-20C "Control Rod Operability Test" are in progress.

FORM SAP 1.3

CONTROLLER SHEET

TIME	MESSAGE NUMBER	ISSUED TO	SUMMARY OF MESSAGE CONTENTS	ANTICIPATED RESULTS AND COMMENTS	EMERGENCY CLASSIFICATION
0700	1	Control Room	- Plant is operating at 100% power. - Annunciator 09-8-6-15 NMP-FITZ 115KV Line 4 BKR 10012 TRIP is sealed in due to maintenance on the line. NiMo estimates return to service for the #4 line to be 0800 tomorrow. - Completion of ST-20C and AOP-24 have identified control rod 30-35 to be stuck in the fully withdrawn position. - Inboard main steam line drain isolation valve 29MOV-74 has failed to close during performance of surveillance testing. - Plant parameters as indicated on SPDS display.	None	None

CONTROLLER'S NOTE: Control rod 30-35 is a member of RWM Group 1 and will not move. In accordance with Technical Specification 3.3.A.2.a, the reactor must be in Cold Shutdown within 24 hours.

FORM SAP 1.3

CONTROLLER SHEET

TIME	MESSAGE NUMBER	ISSUED TO	SUMMARY OF MESSAGE CONTENTS	ANTICIPATED RESULTS AND COMMENTS	EMERGENCY CLASSIFICATION
0705	2	Control Room	Plant parameters as indicated on SPDS display.	Continue attempts to free control rod.	None

CONTROLLER'S NOTE: Recirc run back has reduced power to 65% and flow to 50%.

FORM SAP 1.3

CONTROLLER SHEET

TIME	MESSAGE NUMBER	ISSUED TO	SUMMARY OF MESSAGE CONTENTS	ANTICIPATED RESULTS AND COMMENTS	EMERGENCY CLASSIFICATION
0715	3	Control Room	Plant parameters as indicated on SPDS display.	- Commence reactor shutdown due to stuck control rod. - Initiate notifications of Unusual Event classification due to 24-hour LCO.	Unusual Event

FORM SAP 1.3

CONTROLLER SHEET

TIME	MESSAGE NUMBER	ISSUED TO	SUMMARY OF MESSAGE CONTENTS	ANTICIPATED RESULTS AND COMMENTS	EMERGENCY CLASSIFICATION
0730	4	Control Room	The following annunciators alarm: 09-8-6-10 LHH-Fitz 115KV Line 3 Bkr 10022 Trip 09-8-6-14 115KV SYS VOLT LOSS - Plant parameters as indicated on SPDS display.	- Contact Niagara Mohawk to determine cause for trip on 115KV line. - Discuss requirements for continuing plant shutdown in lieu of loss of 115KV reserve power.	Unusual Event

CONTROLLER'S NOTE: Niagara Mohawk Central regional controller informs the Control Room of a fault on the line. Maintenance crews have been dispatched in an attempt to locate the problem.

If operability of EDG's are to be demonstrated due to loss of reserve power, the "A" diesel starts but its speed oscillates excessively.

The plant is at 50% power and 50% flow.

FORM SAP 1.3

CONTROLLER SHEET

TIME	MESSAGE NUMBER	ISSUED TO	SUMMARY OF MESSAGE CONTENTS	ANTICIPATED RESULTS AND COMMENTS	EMERGENCY CLASSIFICATION
0745	5	Control Room	Plant parameters as indicated on SPDS display.	Continue investigating problems with offsite power.	Unusual Event

CONTROLLER'S NOTE: NiMo has suspended maintenance on the #4 line. A temporary fix for emergency restoration of power will be completed within two hours.

If necessary, the Emergency Director will be prompted to maintain power at 45% due to loss of offsite reserve power.

FORM SAP 1.3

CONTROLLER SHEET

TIME	MESSAGE NUMBER	ISSUED TO	SUMMARY OF MESSAGE CONTENTS	ANTICIPATED RESULTS AND COMMENTS	EMERGENCY CLASSIFICATION
0800	6	Control Room	The following annunciators alarm: 09-3-1-9 Fuel Pool Cool & Cln Up Trouble 09-3-1-10 Core Spray A or B Disch Line Not Full 09-3-1-18 RHR A or B Disch Line Not Full 09-3-1-19 ARM Downscale 09-3-1-20 Refuel Area ARM Rad Hi 09-3-1-29 New Fuel Storage ARM Rad Hi 09-3-1-30 Radw Bldg ARM Rad Hi 09-3-1-39 Turb Bldg ARM Rad Hi 09-3-1-40 Rx Bldg ARM Rad Hi 09-3-2-7 Off Gas Smpl Flow Hi or Lo 09-3-2-10 Off Gas Timer Initiated 09-3-2-17 Off Gas Rad Mon Downscale or Inop	- Determine cause of scram. - Take actions in accordance with AOP's for loss of power. - Review classifications and consider activating emergency facilities.	Unusual Event

FORM SAP 1.3

CONTROLLER SHEET

TIME	MESSAGE NUMBER	ISSUED TO	SUMMARY OF MESSAGE CONTENTS	ANTICIPATED RESULTS AND COMMENTS	EMERGENCY CLASSIFICATION
0800	6 (cont'd)	Control Room	Annunciators continued: 09-3-2-19 Rx Bldg Vent Rad Mon Downscale 09-3-3-11 Main Stm Rad Mon Downscale 09-3-3-22 HPCI Stm Leak Det Logic Pwr Fail 09-4-0-1 Cont Hi Range Rad Mon A Fail or Pwr Loss 09-4-0-4 Cont Hi Range Rad Mon B Fail or Pwr Loss 09-4-0-21 PASS Gas Smpl Ht Trace A Trouble 09-4-0-22 PASS Gas Smpl Ht Trace B Trouble 09-4-0-23 PASS Exh Fan Trouble 09-4-1-16 SRV Leaking 09-4-1-19 Rx Bldg Equip Sump A Temp Hi 09-4-1-20 Rx Bldg Equip Sump B Temp Hi		Unusual Event

TIME	MESSAGE NUMBER	ISSUED TO	SUMMARY OF MESSAGE CONTENTS	ANTICIPATED RESULTS AND COMMENTS	EMERGENCY CLASSIFICATION
0800	6 (cont'd)	Control Room	Annunciators continued: 09-4-2-1 DW Equip Drn Isol Vlv Closed 09-4-2-2 DW Floor Drn Isol Vlv Closed 09-4-2-3 PCIS Sys A Vlv Overload or Pwr Loss 09-4-2-6 SRV Sonic Mon Alarm Hi 09-4-2-11 DW Equip Sump Temp Hi or Cool Wtr Flow Lo 09-4-2-13 PCIS Sys B Vlv Overload or Pwr Loss 09-4-2-20 RWR MG A Drv Mtr Bkr Trip 09-4-2-21 DW Equip Sump HI-HI or LO-LO 09-4-2-36 TIP ARM Rad Hi 09-4-2-37 RWR MG A Oil Mist Elim Diff Press Hi 09-4-3-1 RWR MG A Gen Lockout		Unusual Event

FORM SAP 1.3

CONTROLLER SHEET

TIME	MESSAGE NUMBER	ISSUED TO	SUMMARY OF MESSAGE CONTENTS	ANTICIPATED RESULTS AND COMMENTS	EMERGENCY CLASSIFICATION
0800	6 (cont'd)	Control Room	Annunciators continued: 09-4-3-2 RWR Loop A Out of Serv 09-4-3-5 RWR Loop B Out of Serv 09-4-3-10 RWR MG B Gen Lockout 09-4-3-11 RWR MG A Scoop Tube Lock 09-4-3-19 RWR MG B Drv Mtr Bkr Trip 09-4-3-20 RWR MG B Scoop Tube Lock 09-4-3-21 RWR MG A Gen Aux Lockout 09-4-3-30 RWR MG B Gen Aux Lockout 09-4-3-31 RWR MG A Speed Cntrl Sig Failed 09-4-3-36 RWR MG B Oil Mist Elim Diff Press Hi 09-4-3-40 RWR MG B Speed Cntrl Sig Failed 09-5-1-1 SDIV A or B Hi Lvl Trip		Unusual Event

FORM SAP 1.3

CONTROLLER SHEET

TIME	MESSAGE NUMBER	ISSUED TO	SUMMARY OF MESSAGE CONTENTS	ANTICIPATED RESULTS AND COMMENTS	EMERGENCY CLASSIFICATION
0800	6 (cont'd)	Control Room	Annunciators continued: 09-5-1-2 MSIVS Not Full Open Trip 09-5-1-3 RPS A Auto Scram 09-5-1-4 RPS B Auto Scram 09-5-1-5 RPS MG A Trouble 09-5-1-6 PCIS Sys A Rx Lvl LO-LO-LO 09-5-1-7 PCIS Sys B Rx Lvl LO-LO-LO 09-5-1-9 CRD Charging Wtr Press Lo 09-5-1-10 CRD Drv Wtr Filter Diff Press Hi 09-5-1-13 RPS A Man Scram 09-5-1-14 RPS B Man Scram 09-5-1-15 RPS Mg B Trouble 09-5-1-16 PCIS Sys A Stm Tun Temp Hi 09-5-1-17 PCIS Sys B Stm Tun Temp Hi 09-5-1-21 RPS Hi DW Press Trip 09-5-1-22 RPS Hi Rx Press Trip		Unusual Event

FORM SAP 1.3

CONTROLLER SHEET

TIME	MESSAGE NUMBER	ISSUED TO	SUMMARY OF MESSAGE CONTENTS	ANTICIPATED RESULTS AND COMMENTS	EMERGENCY CLASSIFICATION
0800	6 (cont'd)	Control Room	Annunciators continued: 09-5-1-24 Fdtwr Cntrl Lo Flow Vlv Elect Sig Fail 09-5-1-26 PCIS Sys A Main Stm Line Flow Hi 09-5-1-27 PCIS Sys B Main Stm Line Flow Hi 09-5-1-28 Rx Wtr Lvl Alarm Hi or Lo 09-5-1-31 RPS Rx Vessel Lo Lvl Trip 09-5-1-32 RPS Main Stm Line Hi Rad Trip 09-5-1-36 Sys A Stm Line Press Lo 09-5-1-37 Sys B Stm Line Press Lo 09-5-1-41 Neutron Mon Sys Trip 09-5-1-43 CRD Accum Press Lo or Lvl Hi 09-5-1-44 SDIV A or B Not Drained 09-5-1-51 TCV Fast Closure Trip		Unusual Event

FORM SAP 1.3

CONTROLLER SHEET

TIME	MESSAGE NUMBER	ISSUED TO	SUMMARY OF MESSAGE CONTENTS	ANTICIPATED RESULTS AND COMMENTS	EMERGENCY CLASSIFICATION
0800	6 (cont'd)	Control Room	Annunciators continued: 09-5-1-53 TSV Closure Trip 09-5-1-54 Scram Air Hdr Press Hi or Lo 09-5-1-55 PCIS Sys A Group I Isol 09-5-1-56 PCIS Sys B Group I Isol 09-5-1-58 Interposing Relay Sys on DC Pwr 09-5-2-2 Rod Withdrawal Block 09-5-2-3 Rod Drift 09-5-2-6 EHC Lo Press Trip 09-5-2-9 Main Turb Trip 09-5-2-16 Main Turb Exh Hood Hi Temp Trip 09-5-2-20 Main Turb Exh Hood Temp Hi 09-5-2-22 RSCS In Serv 09-5-2-25 Flow Ref Off Norm		Unusual Event

FORM SAP 1.3

CONTROLLER SHEET

TIME	MESSAGE NUMBER	ISSUED TO	SUMMARY OF MESSAGE CONTENTS	ANTICIPATED RESULTS AND COMMENTS	EMERGENCY CLASSIFICATION
0800	6 (cont'd)	Control Room	Annunciators continued: 09-5-2-26 Main Turb Lo Vac Trip 09-5-2-27 Main Turb Loss of Stator Coolant Trip 09-5-2-30 Main Turb Bellows Fail or Lo Vac Trip 09-5-2-32 IRM Downscale 09-5-2-34 APRM Downscale 09-5-2-35 RBM Downscale 09-5-2-36 Main Turb Hi Vib Trip 09-5-2-44 APRM Upscale 09-5-2-45 RBM Upscale or Inop 09-5-2-54 APRM Trip Sys A Inop or Upscale Hi 09-5-2-55 APRM Trip Sys B Inop or Upscale Trip 09-5-2-57 ATTS RPS Div A1 or A2 Pwr Supp Fail or TU Out of File		Unusual Event

FORM SAP 1.3

CONTROLLER SHEET

TIME	MESSAGE NUMBER	ISSUED TO	SUMMARY OF MESSAGE CONTENTS	ANTICIPATED RESULTS AND COMMENTS	EMERGENCY CLASSIFICATION
0800	6 (cont'd)	Control Room	Annunciators continued: 09-5-2-58 ATTS RPS Div A1 or A2 Gross Fail or TU Inop 09-5-2-59 ATTS RPS Div B1 or B2 Pwr Supp Fail or TU Out of File 09-5-2-60 ATTS RPS Div B1 or B2 Gross Fail or TU Inop 09-6-1-3 Circ Wtr Pmp 36P-1A Overload or Trip 09-6-1-10 Screen Wash Bstr Pmp 46P-6A or 6B Trip or Cntrl Pwr Loss 09-6-1-11 Circ Wtr Pmp 36P-1B Overload or Trip 09-6-1-14 Perimeter Drn Pmp Trouble 09-6-1-18 Trvlg Wtr Screen Trip or Cntrl Pwr Loss		Unusual Event

TIME	MESSAGE NUMBER	ISSUED TO	SUMMARY OF MESSAGE CONTENTS	ANTICIPATED RESULTS AND COMMENTS	EMERGENCY CLASSIFICATION
0800	6 (cont'd)	Control Room	Annunciators continued: 09-6-1-19 Circ Wtr Pmp 36P-1C Overload or Trip 09-6-1-24 Stack Dilution Fan 26FN-2A or 2B Fail 09-6-1-29 CNDSR Vac Lo 09-6-1-30 SJAE Stm Press Lo 09-6-1-31 Serv Air Hdr Press Lo 09-6-2-2 RBC Hx Disch Temp Hi or Lo 09-6-2-9 Cond Xfer Pmp 33P-13A Lockout or Trip 09-6-2-12 RBC Conduct Hi 09-6-2-19 Cond Xfer Pmp 33P-13B Lockout or Trip 09-6-2-22 RBC Makeup Tk Lvl Hi or Lo 09-6-2-29 Cond Xfer Hdr Press Lo		Unusual Event

TIME	MESSAGE NUMBER	ISSUED TO	SUMMARY OF MESSAGE CONTENTS	ANTICIPATED RESULTS AND COMMENTS	EMERGENCY CLASSIFICATION
0800	6 (cont'd)	Control Room	Annunciators continued: 09-6-2-30 CNDSR Hotwell Lvl Hi or Lo 09-6-2-31 RBC Hdr Press Lo 09-6-2-32 RBC Hdr Press LO-LO 09-6-2-34 Serv Wtr Hdr Press Lo 09-6-2-36 TBC Hdr Press Lo 09-6-2-38 Breathing Air Hdr Press Lo 09-6-2-39 Serv Air Hdr Isol Vlv Closed 09-6-2-40 Breathing Air Hdr Isol Vlv Cls 09-6-3-20 RFP A & B Seal Wtr Diff Press Lo 09-6-3-30 SJAE or SPE Drn Tk Lvl Hi or Lo 09-6-3-31 Cond Pmp Disch Hdr Press Lo 09-6-3-32 Cond Demin Sys Trouble 09-6-4-1 RFPT A Vac Lo 09-6-4-2 RFPT A Trip		Unusual Event

TIME	MESSAGE NUMBER	ISSUED TO	SUMMARY OF MESSAGE CONTENTS	ANTICIPATED RESULTS AND COMMENTS	EMERGENCY CLASSIFICATION
0800	6 (cont'd)	Control Room	Annunciators continued: 09-6-4-3 RFPT A Oil Clr Disch Oil Temp Hi 09-6-4-4 RFPT A or B Aux Oil Pmp Trouble 09-6-4-6 RFPT A or B Turning Gear Trouble 09-6-4-8 RFPT B Vac Lo 09-6-4-9 RFPT B Trip 09-6-4-10 RFPT B Oil Clr Disch Oil Temp Hi 09-6-4-15 RFPT A or B Emerg Brg Oil Pmp Running 09-7-1-3 Main Gen Auto Reg Trip to Man 09-7-1-5 Unit Prim Lockout 09-7-1-15 Main XSFMR T-18 Temp Hi 09-7-1-26 Main Gen Exciter Filed Bkr Trip		Unusual Event

FORM SAP 1.3

CONTROLLER SHEET

TIME	MESSAGE NUMBER	ISSUED TO	SUMMARY OF MESSAGE CONTENTS	ANTICIPATED RESULTS AND COMMENTS	EMERGENCY CLASSIFICATION
0800	6 (cont'd)	Control Room	Annunciators continued: 09-7-1-28 Main Gen Field Grd Will Alarm Only 09-7-1-31 Main XSFMR T-18 Trouble 09-7-2-1 Main Gen Disc 10031 Open 09-7-2-3 Fitz-Edic 345KV Line 1 Bkr 10052 Trip 09-7-2-7 Fitz-Scriba 345KV Line 10 Bkr 10042 Trip 09-7-2-8 Main Turb Brg Oil Vapor Extractor Trouble 09-7-2-10 Reboiler Lvl Cntrl Trouble 09-7-2-16 EHC Fluid Pwr Unit Tk Trouble 09-7-2-18 Reboiler Trouble 09-7-2-21 345KV Bkr 10042 Ext Dry Time or Loss of AC for Htrs		Unusual Event

FORM SAP 1.3

CONTROLLER SHEET

TIME	MESSAGE NUMBER	ISSUED TO	SUMMARY OF MESSAGE CONTENTS	ANTICIPATED RESULTS AND COMMENTS	EMERGENCY CLASSIFICATION
0800	6 (cont'd)	Control Room	Annunciators continued: 09-7-2-32 Main Gen Core Mon Trouble 09-7-3-1 Main Turb Brg 3 Lift Pmp Trouble 09-7-3-2 Main Turb Brg 4 Lift Pmp Trouble 09-7-3-3 Main Turb Mtr Drvn Suct Oil Pmp Trouble 09-7-3-8 Main Gen Stator Wtr Sys Trouble 09-7-3-13 Main Turb Brg 5 Lift Pmp Trouble 09-7-3-14 Main Turb Brg 6 Lift Pmp Trouble 09-7-3-19 Main Gen Emerg Seal Oil Pmp Running 09-7-3-22 Moist Sep Drn Tk 2A Lvl Lo 09-7-3-24 2nd Stage RHTR Drn Tk 4A Lvl Lo 09-7-3-25 Main Turb Brg 7 Lift Pmp Trouble		Unusual Event

TIME	MESSAGE NUMBER	ISSUED TO	SUMMARY OF MESSAGE CONTENTS	ANTICIPATED RESULTS AND COMMENTS	EMERGENCY CLASSIFICATION
0800	6 (cont'd)	Control Room	Annunciators continued: 09-7-3-26 Main Turb Brg 8 Lift Pmp Trouble 09-7-3-29 EHC Pmp A or B Trip or Cntrl Pwr Loss 09-7-3-32 SPE Fan Overload Cntrl Volts Lo 09-7-3-38 Main Turb Turning Gear Oil Pmp Running 09-7-3-40 Main Stm Leads Drn Pot Lvl Hi 09-7-3-43 SPE Vac Lo 09-7-3-44 Seal Stm Hdr Press Lo 09-7-3-46 Moist Sep Drn Tk 28 Lvl Lo 09-7-3-48 2nd Stage RHTR Drn Tk 48 Lvl Lo 09-8-1-4 Computer Cntrl Volt Trouble 09-8-1-8 UPS on DC Drive 09-8-2-4 EDG C Load Bkr 10512 Closed		Unusual Event

FORM SAP 1.3

CONTROLLER SHEET

TIME	MESSAGE NUMBER	ISSUED TO	SUMMARY OF MESSAGE CONTENTS	ANTICIPATED RESULTS AND COMMENTS	EMERGENCY CLASSIFICATION
0800	6 (cont'd)	Control Room	Annunciators continued: 09-8-2-8 Bus 10300-10500 Tie Bkr 10514 Trip or Cntrl Pwr Loss 09-8-2-9 EDG A Eng Trouble or Shutdown 09-8-3-9 10100 or 10200 or 10300 or 10400 Prolong UV Trip 09-8-3-17 Bus 10300-10500 4KV Tie Bkr 10304 Trip 09-8-3-18 L13 L23 L33 L43 4KV Supp Fdr Bkr 10340 Trip 09-8-3-23 L14 L24 L34 L44 4KV Supp Fdr Bkr 10440 Trip 09-8-3-24 4160V Bus 10500 Degraded Voltage Timer Initiated 09-8-3-26 Bus 10300 Norm Supp Bkr 10302 Trip		Unusual Event

FORM SAP 1.3

CONTROLLER SHEET

TIME	MESSAGE NUMBER	ISSUED TO	SUMMARY OF MESSAGE CONTENTS	ANTICIPATED RESULTS AND COMMENTS	EMERGENCY CLASSIFICATION
0800	6 (cont'd)	Control Room	Annunciators continued: 09-8-3-32 Bus 10400 Norm Supp Bkr 10402 Trip 09-8-4-1 Bus 10400-10600 Tie Bkr 10614 Trip or Cntrl Pwr Loss 09-8-4-3 EDG B Load Bkr 10602 Closed 09-8-4-6 EDG D Load Bkr 10612 Closed 09-8-4-11 EDG B Eng Trouble or Shutdown 09-8-4-14 EDG D Eng Trouble or Shutdown 09-8-5-3 Bus 10100 Norm Supp Bkr 10102 Trip 09-8-5-13 Bus 10700 Supp Bkr 10702 Trip 09-8-5-15 Bus 10200 Norm Supp Bkr 10202 Trip		Unusual Event

TIME	MESSAGE NUMBER	ISSUED TO	SUMMARY OF MESSAGE CONTENTS	ANTICIPATED RESULTS AND COMMENTS	EMERGENCY CLASSIFICATION
0800	6 (cont'd)	Control Room	Annunciators continued: 09-8-6-1 LHH-Fitz 115KV Line 3 Prim Relay Trip 09-8-6-10 LHH-Fitz 115KV Line 3 Bkr 10022 Air Lo or AC Pwr Loss 09-8-6-14 115KV SYS Volt Loss 09-8-6-15 NMP-Fitz 115KV Line 4 Bkr 10012 Trip - HPCI/RCIC auto initiate and control level. - "B", "C" and "D" EDG's start and load the 10500 and 10600 buses. - Plant parameters as indicated on SPDS display.		Unusual Event

FORM SAP .3

CONTROLLER SHEET

TIME	MESSAGE NUMBER	ISSUED TO	SUMMARY OF MESSAGE CONTENTS	ANTICIPATED RESULTS AND COMMENTS	EMERGENCY CLASSIFICATION
0800	6 (cont'd)	Control Room			Unusual Event

CONTROLLER'S NOTE: Main turbine #8 bearing vibration jumped to 10.0 mils, causing turbine trip and subsequent scram.

Steam tunnel temperatures are not available until RPS is restored. After RPS restoration, temperature indications of 200°F near the break and 160°F further down the tunnel are available.

Only 10500 and 10600 emergency bus loads remain powered (see attached list).

Drywell temperatures increase until ESW is lined-up to coolers.

HPCI and RCIC are in manual control.

16. Upon restoring 10500 bus to service:

    a. Restore isolated systems to service in accordance with F-AOP-15, Recovery from Isolations.

    b. Place LPCI A, 24 VDC A, and Station A Batteries on float or equalize charge as necessary.

    c. Restore to service equipment isolated to minimize battery loads.

D. SUMMARY OF IMPORTANT 10500 BUS LOADS:

1. RHR A and RHRSW A Systems
2. Core Spray System A
3. CRD Pump A
4. SGT System A
5. Drywell Sump Isolation Valves, 20MOV-82 and 94 fail "as is"
6. Drywell Cooler Fans 68FN-2A, 2C, 4A, and 4D.
7. SLC Pump A and SLC Heaters
8. LPCI MOV Independent Power Supply A
9. Drywell Radiation and Atmosphere Monitors A
10. RCIC Test Power
11. CAD System A
12. RPS System A
13. RFP Turning Gear A
14. Station Battery A Charger and Battery Ventilation

15. UPS Alternate Supply
16. ESW System A
17. Main Turbine Lift Pumps 4A, 4C and 4E
18. Inboard Drywell Vent and Purge AOVs
19. EPIC normal supply

c. Restore to service equipment isolated to minimize battery loads.

D. SUMMARY OF IMPORTANT 10600 BUS LOADS:

1. RHR B and RHRSW B Systems
2. Core Spray System B
3. CRD Pump B
4. SGT System B
5. Drywell Sump Isolation Valves 20AOV-83 and 95
6. Drywell Cooler Fans 68FN-2B, 2D, 4B, and 4D
7. SLC Pump B and SLC Heaters
8. LPCI Independent Power Supply B
9. Drywell Radiation and Atmosphere Monitors B
10. Main Turbine Turning Gear Oil Pump and Turning Gear
11. CAD System B
12. RPS System B
13. RFP B Turning Gear
14. Station Battery B Charger and Battery Ventilation
15. UPS Normal AC Supply
16. ESW System B
17. Main Turbine Lift Pumps 94P-4B, 4D, and 4F

TITLE: LOSS OF 10600 BUS* NO. F-AOP-19

18. Outboard Drywell Vent and Purge AOVs

19. Crescent and Reactor Building Smoke Detectors

20. HPCI Test Power

FORM SAP 1.3

CONTROLLER SHEET

TIME	MESSAGE NUMBER	ISSUED TO	SUMMARY OF MESSAGE CONTENTS	ANTICIPATED RESULTS AND COMMENTS	EMERGENCY CLASSIFICATION
0801	7	Control Room	The following annunciators are sealed in: 09-3-3-2 Div I Ambient Temp Hi 09-3-3-12 Div II Ambient Temp Hi 09-5-1-16 PCIS Sys A Stm Tun Temp Hi 09-5-1-17 PCIS Sys B Stm Tun Temp Hi - 09-4 panel dual position indications for 29MO[illegible]-74, -77 and -79.	- Recognize steam leak without isolation. - Classify Site Area Emergency. - Activate TSC, OSC and EOF.	Site Area Emergency

CONTROLLER'S NOTE: Once RPS is restored, all associated annunciators clear except those listed above.

FORM SAP 1.3

CONTROLLER SHEET

TIME	MESSAGE NUMBER	ISSUED TO	SUMMARY OF MESSAGE CONTENTS	ANTICIPATED RESULTS AND COMMENTS	EMERGENCY CLASSIFICATION
0815	8	Control Room	Plant parameters as indicated on SPDS display.	- Continue activation of emergency facilities. - Assemble non-essential staff in Training Building.	Site Area Emergency

FORM SAP 1.3

CONTROLLER SHEET

TIME	MESSAGE NUMBER	ISSUED TO	SUMMARY OF MESSAGE CONTENTS	ANTICIPATED RESULTS AND COMMENTS	EMERGENCY CLASSIFICATION
0830	9	Control Room	- The following annunciator alarms: 09-7-3-42 Hydrogen Seal Oil Trouble - Plant parameters as indicated on SPDS display.	- Monitor hydrogen pressure. - Take actions in accordance with AOP-48 for loss of generator hydrogen. - Maintain contact with Niagara Mohawk for restoration of offsite power.	Site Area Emergency

CONTROLLER'S NOTE: Main generator hydrogen pressure should be decreasing at a rate which will be zero pressure by 0900.

If the Emergency Director is not in the process of having declared a Site Area Emergency, this must be prompted.

FORM SAP 1.3

CONTROLLER SHEET

TIME	MESSAGE NUMBER	ISSUED TO	SUMMARY OF MESSAGE CONTENTS	ANTICIPATED RESULTS AND COMMENTS	EMERGENCY CLASSIFICATION
0845	10	Control Room	Plant parameters as indicated on SPDS display.	Continue actions for restoring plant stability.	Site Area Emergency

FORM SAP 1.3

CONTROLLER SHEET

TIME	MESSAGE NUMBER	ISSUED TO	SUMMARY OF MESSAGE CONTENTS	ANTICIPATED RESULTS AND COMMENTS	EMERGENCY CLASSIFICATION
0900	11	Control Room	Plant parameters as indicated on SPDS display.	Continue plant assessment activities.	Site Area Emergency

CONTROLLER'S NOTE: Main generator hydrogen pressure is zero.

FORM SAP 1.3

CONTROLLER SHEET

TIME	MESSAGE NUMBER	ISSUED TO	SUMMARY OF MESSAGE CONTENTS	ANTICIPATED RESULTS AND COMMENTS	EMERGENCY CLASSIFICATION
0915	12	Control Room	- NMP Line #4 is restored and T-3 transformer can be placed back in service. - Plant parameters as indicated on SPDS display.	- Close 10012 and 10022 breakers.	Site Area Emergency

FORM SAP 1.3

CONTROLLER SHEET

TIME	MESSAGE NUMBER	ISSUED TO	SUMMARY OF MESSAGE CONTENTS	ANTICIPATED RESULTS AND COMMENTS	EMERGENCY CLASSIFICATION
0930	13	Control Room	- An explosion is heard coming from the Turbine Building. - Plant parameters as indicated on SPDS display.		Site Area Emergency

CONTROLLER'S NOTE: At 0930, an explosion is heard coming from the Turbine Building. The explosion occurred from the ignition of a pocket of hydrogen during unisolation of the ventilation system.

FORM SAP 1.3

CONTROLLER SHEET

TIME	MESSAGE NUMBER	ISSUED TO	SUMMARY OF MESSAGE CONTENTS	ANTICIPATED RESULTS AND COMMENTS	EMERGENCY CLASSIFICATION
0945	14	Control Room	Plant parameters as indicated on SPDS display.		Site Area Emergency

FORM SAP 1.3

CONTROLLER SHEET

TIME	MESSAGE NUMBER	ISSUED TO	SUMMARY OF MESSAGE CONTENTS	ANTICIPATED RESULTS AND COMMENTS	EMERGENCY CLASSIFICATION
1000	15	Control Room	Plant parameters as indicated on SPDS display.		Site Area Emergency

FORM SAP 1.3

CONTROLLER SHEET

TIME	MESSAGE NUMBER	ISSUED TO	SUMMARY OF MESSAGE CONTENTS	ANTICIPATED RESULTS AND COMMENTS	EMERGENCY CLASSIFICATION
1015	16	Control Room	The following annunciators alarm: 09-8-1-20 125VDC Batt Chgr A Current or Output Lo - Plant parameters as indicated on SPDS display.		Site Area Emergency

CONTROLLER'S NOTE: If Operations attempts to shift UPS back to AC-Drive, the MG-Set trips and failover to the alternate feeder occurs.

FORM SAP 1.3

CONTROLLER SHEET

TIME	MESSAGE NUMBER	ISSUED TO	SUMMARY OF MESSAGE CONTENTS	ANTICIPATED RESULTS AND COMMENTS	EMERGENCY CLASSIFICATION
1030	17	Control Room	Plant parameters as indicated on SPDS display.		Site Area Emergency

FORM SAP 1.3

CONTROLLER SHEET

TIME	MESSAGE NUMBER	ISSUED TO	SUMMARY OF MESSAGE CONTENTS	ANTICIPATED RESULTS AND COMMENTS	EMERGENCY CLASSIFICATION
1045	18	Control Room	Plant parameters as indicated on SPDS display.		Site Area Emergency

FORM SAP 1.3

CONTROLLER SHEET

TIME	MESSAGE NUMBER	ISSUED TO	SUMMARY OF MESSAGE CONTENTS	ANTICIPATED RESULTS AND COMMENTS	EMERGENCY CLASSIFICATION
1100	19	Control Room	Plant parameters as indicated on SPDS display.		Site Area Emergency

FORM SAP 1.3

CONTROLLER SHEET

TIME	MESSAGE NUMBER	ISSUED TO	SUMMARY OF MESSAGE CONTENTS	ANTICIPATED RESULTS AND COMMENTS	EMERGENCY CLASSIFICATION
1115	20	Control Room	Plant parameters as indicated on SPDS display.		Site Area Emergency

FORM SAP 1.3

CONTROLLER SHEET

TIME	MESSAGE NUMBER	ISSUED TO	SUMMARY OF MESSAGE CONTENTS	ANTICIPATED RESULTS AND COMMENTS	EMERGENCY CLASSIFICATION
1130	21	Control Room	Plant parameters as indicated on SPDS display.		Site Area Emergency

FORM SAP 1.3

CONTROLLER SHEET

TIME	MESSAGE NUMBER	ISSUED TO	SUMMARY OF MESSAGE CONTENTS	ANTICIPATED RESULTS AND COMMENTS	EMERGENCY CLASSIFICATION
1145	22	Control Room	Plant parameters as indicated on SPDS display.		Site Area Emergency

FORM SAP 1.3

CONTROLLER SHEET

TIME	MESSAGE NUMBER	ISSUED TO	SUMMARY OF MESSAGE CONTENTS	ANTICIPATED RESULTS AND COMMENTS	EMERGENCY CLASSIFICATION
1200	23	Emergency Facility Managers	Plant is stable. Commence discussions on terminating the emergency and entering recovery.		

FORM SAP 1.4 Page 1 of 28

MESSAGE SHEET

MESSAGE NO. 1

Exercise Time: 0700

Issued To: Control Room

Issued By: Lead Controller

MESSAGE

Plant is operating at 100% power.

Annunciator 09-8-6-15 NMP-FITZ 115KV Line 4 BKR 10012 TRIP is sealed in due to maintenance on the line. NiMo estimates return to service for the #4 line to be 0800 tomorrow.

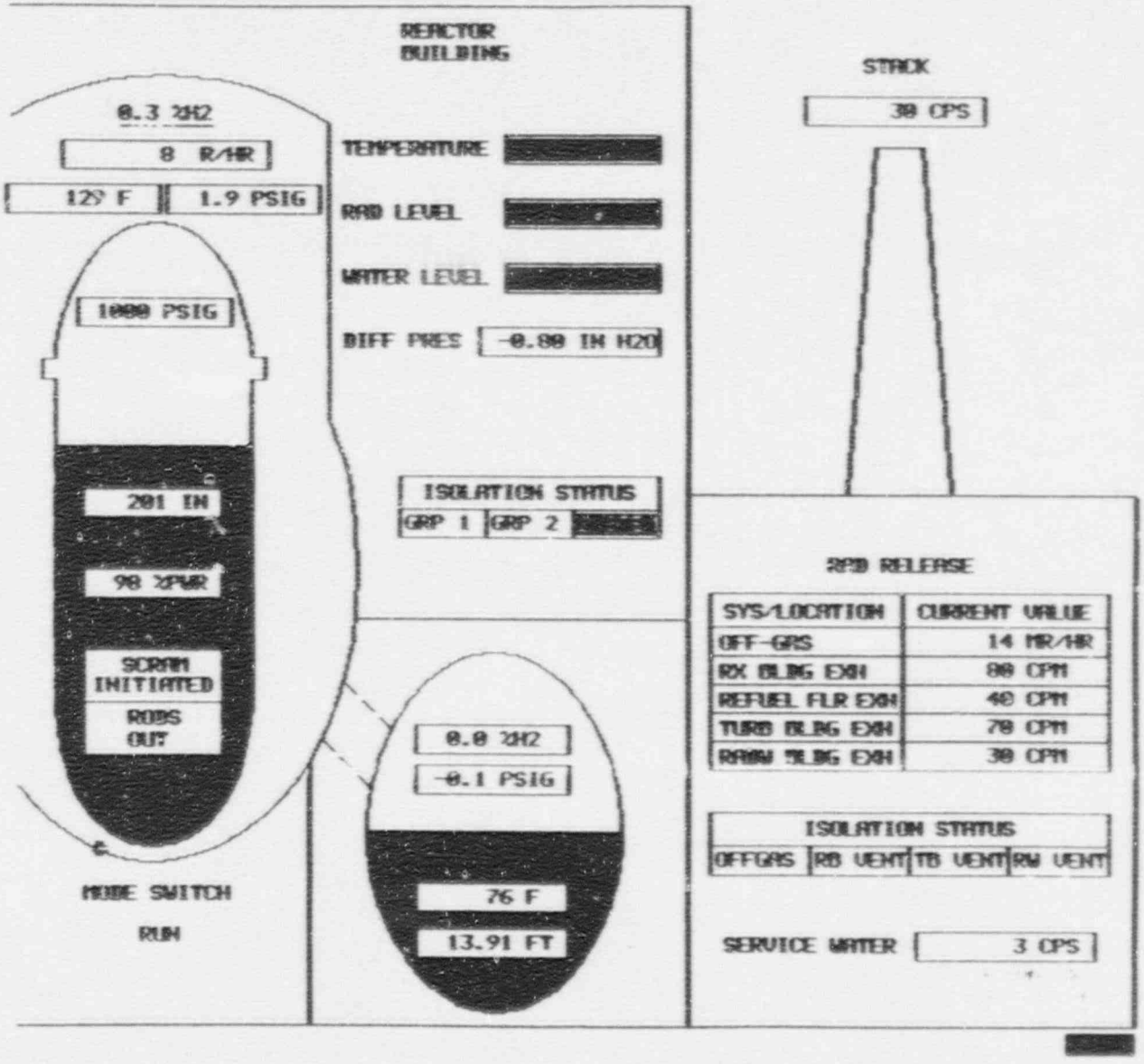
Completion of ST-20C and AOP-24 have identified control rod 30-35 to be stuck in the fully withdrawn position.

Inboard main steam line drain isolation valve 29MOV-74 has failed to close during performance of surveillance testing.

Plant parameters as indicated on SPDS display.

06:22:18 17RE-463 0.000 MR/HR // 06:22:18 17RE-434 0.000 MR/HR

PLANT PLA



06:22:18 17RE-463 0.000 MR/HR // 06:22:18 17RE-434 0.000 MR/HR

Form EAP-1.1.3
James A. Fitzpatrick Nuclear Power Plant
PART III - Plant Parameters

Sequence No. _____

APRM REACTOR POWER	98.00	%
IRM REACTOR POWER	5.000	%
SRM REACTOR POWER	0.9000E+06	CPS
RX WATER LEVEL	201.0	Inch TAF
FEEDWATER FLOW	18.34	MLB/HR
RCIC FLOW	0.0000E+00	GPM
REACTOR PRESSURE	1000.	PSIG
HPCI FLOW	0.0000E+00	GPM
LPCI A FLOW	0.0000E+00	GPM
LPCI B FLOW	0.0000E+00	GPM
"A" CORE SPRAY FLOW	0.0000E+00	GPM
"B" CORE SPRAY FLOW	0.0000E+00	GPM
DRYWELL SUMP LEVEL	15.00	Inch
STACK GAS RAD.	75.00	UCI/sec
RX BLDG VENT RAD.	19.00	UCI/sec
REFUEL FLR EXH RAD.	2.000	UCI/sec
TB BLDG VENT RAD.	9.000	UCI/sec
RW BLDG VENT RAD.	2.000	UCI/sec
SERVICE WATER RAD.	0.1500E-05	UCI/ml
STACK HI RANGE RAD.	0.2819	CI/sec
TB BLD HI RANGE RAD.	0.0000E+00	CI/sec
RW BLD HI RANGE RAD.	0.0000E+00	CI/sec
OFFGAS RAD.	14.00	mR/Hr
DRYWELL RAD.	8.000	R/Hr
HIGHEST MSL RAD MON	1500.	mR/Hr
DRYWELL PRESSURE	1.900	PSIG
DRYWELL TEMPERATURE	129.0	Deg F
TORUS WATER AVG TMP	76.00	Deg F
TORUS WATER LEVEL	13.91	FT
DRYWELL H2 CONC	0.3999	%
DRYWELL O2 CONC	1.000	%
CST LEVEL	250.0	Inch

Emergency Director Approval: _____ TIME: _____

MESSAGE SHEET

MESSAGE NO. 2

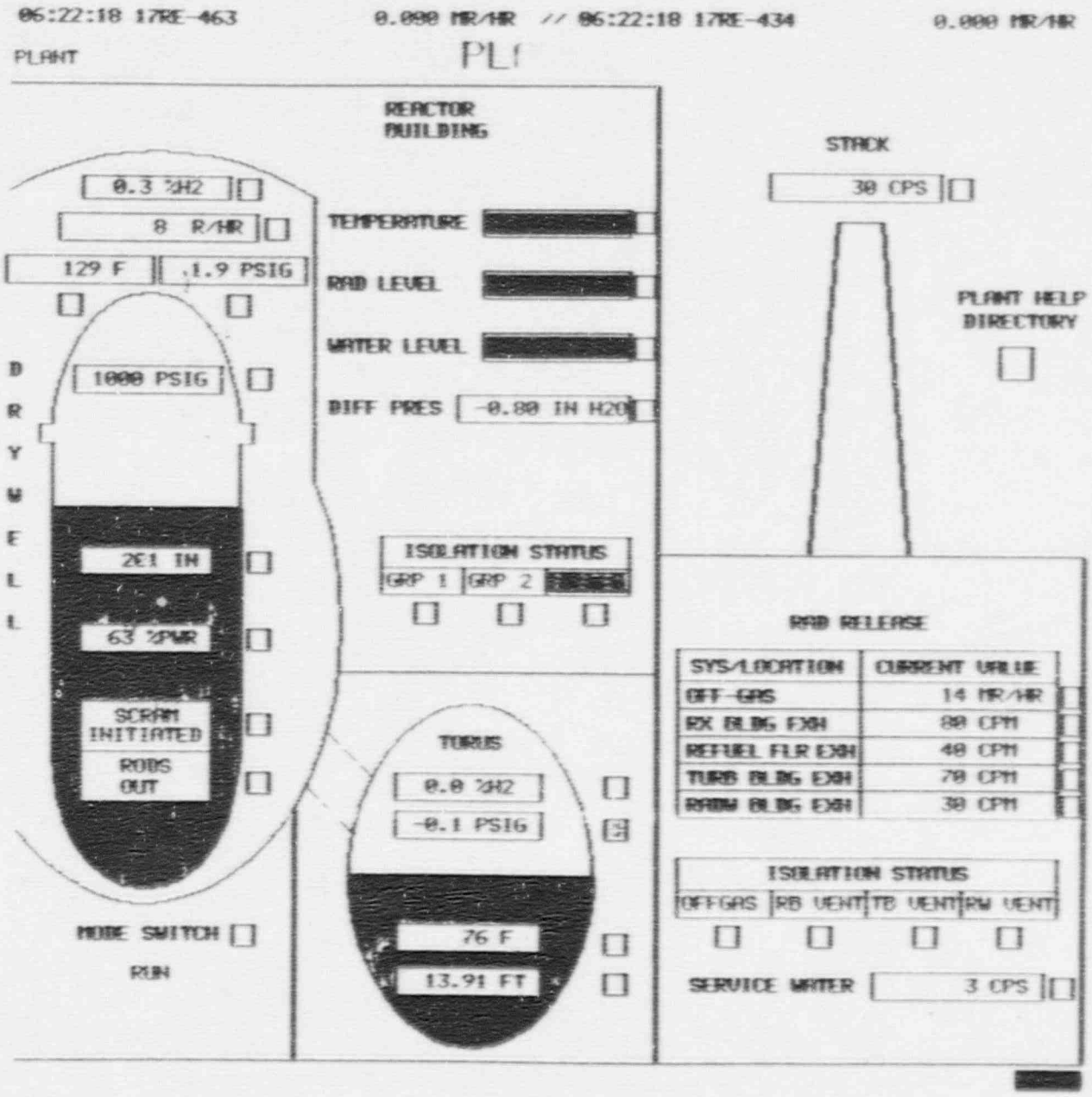
Exercise Time: 0705

Issued To: Control Room

Issued By: Lead Controller

MESSAGE

Plant parameters as indicated on SPDS display.


06:22:18 17RE-463
0.000 MR/HR
// 06:22:18 17RE-434
0.000 MR/HR
PLANT
PL1
REACTOR BUILDING
STACK
30 CPS
0.3 %H2
8 R/HR
129 F
1.9 PSIG
TEMPERATURE
RAD LEVEL
WATER LEVEL
DIFF PRES
-0.80 IN H20
PLANT HELP DIRECTORY
DRYWELL
1000 PSIG
ISOLATION STATUS
GRP 1
GRP 2
261 IN
63 %PWR
SCRAM INITIATED
RODS OUT
RAD RELEASE
SYS/LOCATION
CURRENT VALUE
OFF-GAS
14 MR/HR
RX BLDG EXH
80 CPM
REFUEL FLR EXH
40 CPM
TURB BLDG EXH
70 CPM
RADW BLDG EXH
30 CPM
TORUS
0.0 %H2
-0.1 PSIG
ISOLATION STATUS
OFF-GAS
RB VENT
TB VENT
RW VENT
76 F
13.91 FT
MODE SWITCH
RUN
SERVICE WATER
3 CPS

Form EAP-1.1.3
James A. Fitzpatrick Nuclear Power Plant
PART III - Plant Parameters

Sequence No. _____

APRM REACTOR POWER	61.93	%
IRM REACTOR POWER	5.000	%
SRM REACTOR POWER	0.9000E+06	CPS
RX WATER LEVEL	201.0	Inch TAF
FEEDWATER FLOW	7.027	MLB/HR
RCIC FLOW	0.0000E+00	GPM
REACTOR PRESSURE	1000.	PSIG
HPCI FLOW	0.0000E+00	GPM
LPCI A FLOW	0.0000E+00	GPM
LPCI B FLOW	0.0000E+00	GPM
"A" CORE SPRAY FLOW	0.0000E+00	GPM
"B" CORE SPRAY FLOW	0.0000E+00	GPM
DRYWELL SUMP LEVEL	15.00	Inch
STACK GAS RAD.	75.00	UCI/sec
RX BLDG VENT RAD.	19.00	UCI/sec
REFUEL FLR EXH RAD.	2.000	UCI/sec
TB BLDG VENT RAD.	9.000	UCI/sec
RW BLDG VENT RAD.	2.000	UCI/sec
SERVICE WATER RAD.	0.1500E-05	UCI/ml
STACK HI RANGE RAD.	0.2819	CI/sec
TB BLD HI RANGE RAD.	0.0000E+00	CI/sec
RW BLD HI RANGE RAD.	0.0000E+00	CI/sec
OFFGAS RAD.	14.00	mR/Hr
DRYWELL RAD.	8.000	R/Hr
HIGHEST MSL RAD MON	1500.	mR/Hr
DRYWELL PRESSURE	1.900	PSIG
DRYWELL TEMPERATURE	129.0	Deg F
TORUS WATER AVG TMP	76.00	Deg F
TORUS WATER LEVEL	13.91	FT
DRYWELL H2 CONC	0.7000	%
DRYWELL O2 CONC	1.800	%
CST LEVEL	250.0	Inch

Emergency Director Approval: _____ TIME: _____

FORM SAP 1.4 Page 3 of 28

MESSAGE SHEET

MESSAGE NO. 3

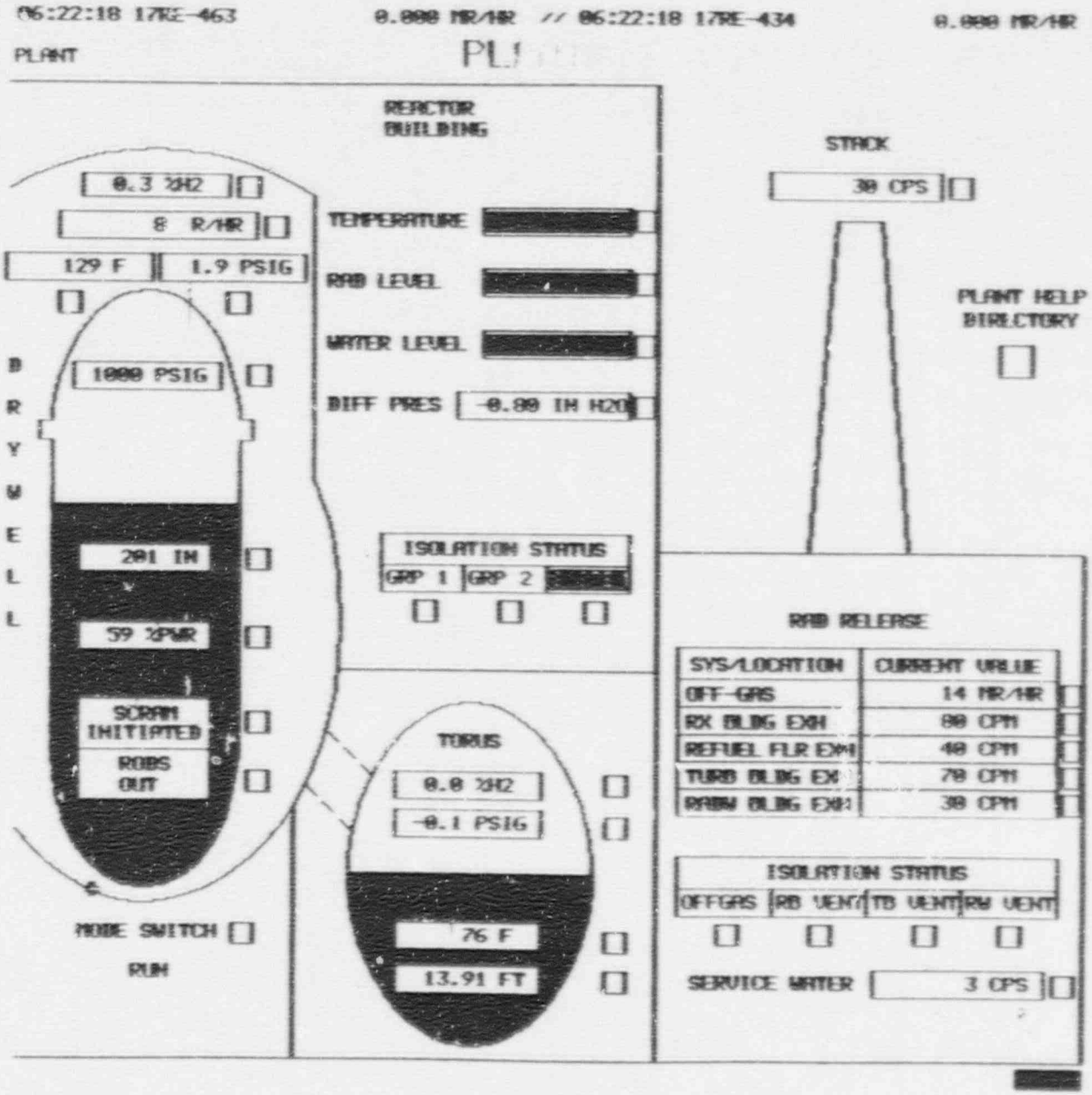
Exercise Time: 0715

Issued To: Control Room

Issued By: Lead Controller

MESSAGE

Plant parameters as indicated on SPDS display.


06:22:18 17RE-463
0.000 MR/HR
// 06:22:18 17RE-434
0.000 MR/HR
PLANT
REACTOR BUILDING
STACK
0.3 %H2
8 R/HR
129 F
1.9 PSIG
TEMPERATURE
RAD LEVEL
WATER LEVEL
DIFF PRES
-0.00 IN H2O
39 CPS
PLANT HELP DIRECTORY
DRYWELL
1000 PSIG
291 IN
59 %PWR
SCRAM INITIATED
RODS OUT
ISOLATION STATUS
GRP 1
GRP 2
RAD RELEASE
SYS/LOCATION
CURRENT VALUE
OFF-GAS
14 MR/HR
RX BLDG EXH
80 CPM
REFUEL FLR EXH
40 CPM
TURB BLDG EX
70 CPM
RADW BLDG EXH
30 CPM
TORUS
0.0 %H2
-0.1 PSIG
76 F
13.91 FT
ISOLATION STATUS
OFFGAS
RB VENT
TB VENT
RW VENT
MODE SWITCH
RUN
SERVICE WATER
3 CPS

06:22:18 17RE-463 0.000 MR/HR // 06:22:18 17RE-434 0.000 MR/HR

Form ERP-1.1.3
James A. Fitzpatrick Nuclear Power Plant
PART III - Plant Parameters

Sequence No. _____

APRM REACTOR POWER	58.58	%
IRM REACTOR POWER	5.000	%
SRM REACTOR POWER	0.9000E+05	CPS
RX WATER LEVEL	201.0	Inch TAF
FEEDWATER FLOW	6.825	MLB/HR
RCIC FLOW	0.0000E+00	GPM
REACTOR PRESSURE	1000.	PSIG
HPCI FLOW	0.0000E+00	GPM
LPCI A FLOW	0.0000E+00	GPM
LPCI B FLOW	0.0000E+00	GPM
"A" CORE SPRAY FLOW	0.0000E+00	GPM
"B" CORE SPRAY FLOW	0.0000E+00	GPM
DRYWELL SUMP LEVEL	15.00	Inch
STACK GAS RAD.	75.00	UCI/sec
RX BLDG VENT RAD.	19.00	UCI/sec
REFUEL FLR EXH RAD.	2.000	UCI/sec
TB BLDG VENT RAD.	9.000	UCI/sec
RW BLDG VENT RAD.	2.000	UCI/sec
SERVICE WATER RAD.	0.1500E-05	UCI/ml
STACK HI RANGE RAD.	0.2819	CI/sec
TB BLD HI RANGE RAD.	0.0000E+00	CI/sec
RW BLD HI RANGE RAD.	0.0000E+00	CI/sec
OFFGAS RAD.	14.00	mR/Hr
DRYWELL RAD.	8.000	R/Hr
HIGHEST MSL RAD MON	1500.	mR/Hr
DRYWELL PRESSURE	1.900	PSIG
DRYWELL TEMPERATURE	129.0	Deg F
TORUS WATER AVG TMP	76.00	Deg F
TORUS WATER LEVEL	13.91	FT
DRYWELL H2 CONC	0.3900	%
DRYWELL O2 CONC	1.000	%
CST LEVEL	250.0	Inch

Emergency Director Approval: _____ TIME: _____

MESSAGE SHEET

MESSAGE NO. 4

Exercise Time: 0730

Issued To: Control Room

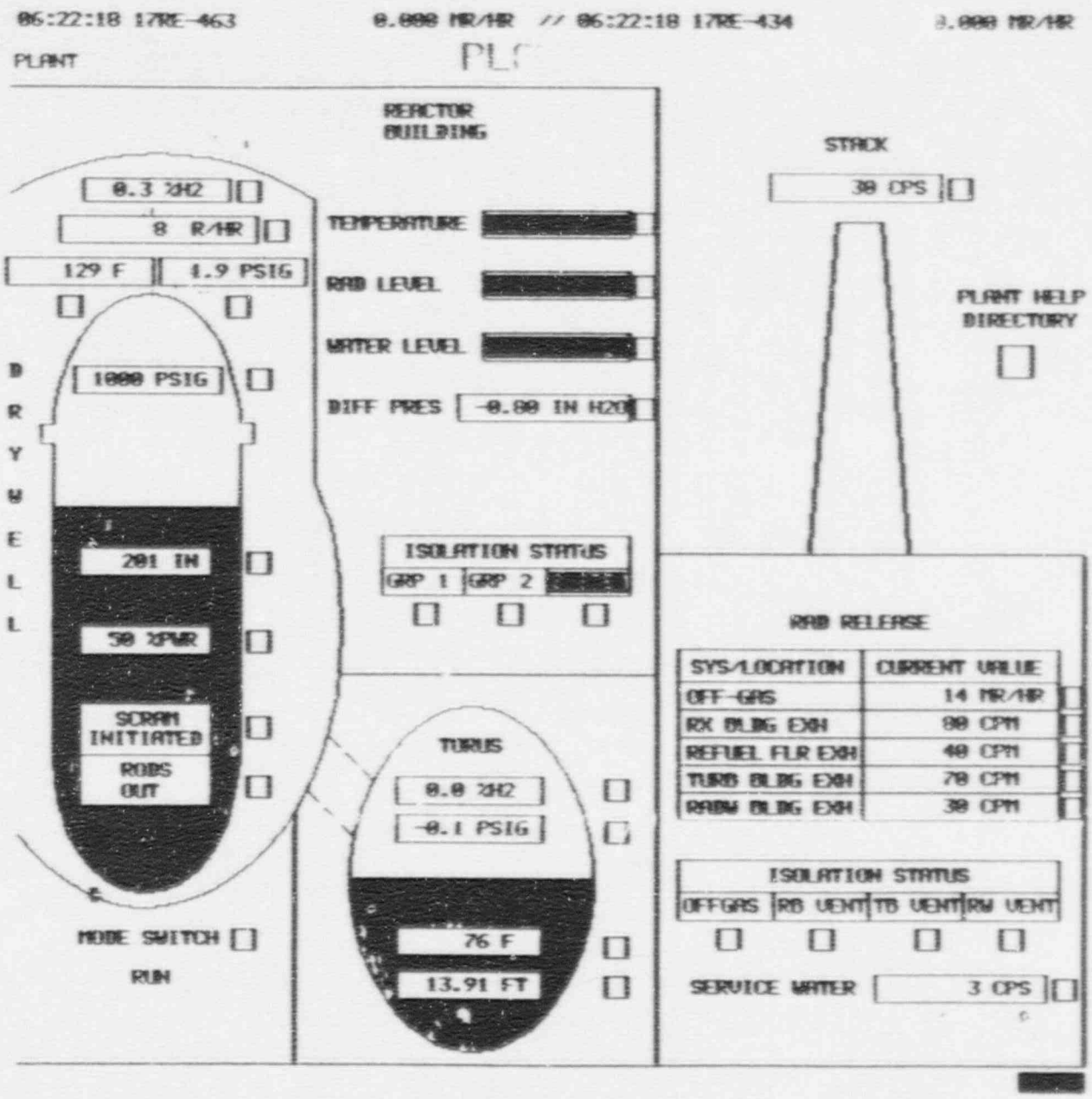
Issued By: Lead Controller

MESSAGE

The following annunciators alarm:

09-8-6-10 LHH-Fitz 115KV Line 3 Bkr 10022 Trip
09-8-6-14 115KV SYS VOLT LOSS

Plant parameters as indicated on SPDS display.


06:22:18 17RE-463
0.000 MR/HR
// 06:22:18 17RE-434
0.000 MR/HR
PLANT
REACTOR BUILDING
STACK
0.3 %H2
8 R/HR
129 F
1.9 PSIG
1000 PSIG
201 IN
50 %PWR
SCRAM INITIATED
RODS OUT
DRYWELL
TEMPERATURE
RAD LEVEL
WATER LEVEL
DIFF PRES
-0.00 IN H20
ISOLATION STATUS
GRP 1
GRP 2
39 CPS
PLANT HELP DIRECTORY
RAD RELEASE
SYS/LOCATION
CURRENT VALUE
OFF-GAS
14 MR/HR
RX BLDG EXH
80 CPM
REFUEL FLR EXH
40 CPM
TURB BLDG EXH
70 CPM
RADW BLDG EXH
30 CPM
ISOLATION STATUS
OFFGAS
RB VENT
TB VENT
RW VENT
SERVICE WATER
3 GPS
TORUS
0.0 %H2
-0.1 PSIG
76 F
13.91 FT
MODE SWITCH
RUN

06:22:18 17RE-463 0.000 MR/HR // 06:22:18 17RE-434 0.000 MR/HR

Form EAP-1.1.3
James A. Fitzpatrick Nuclear Power Plant
PART III - Plant Parameters

Sequence No. _____

APRM REACTOR POWER	49.77	%
IRM REACTOR POWER	5.000	%
SRM REACTOR POWER	0.9000E+06	CPS
RX WATER LEVEL	201.0	Inch THF
FEEDWATER FLOW	6.294	MLB/HR
RCIC FLOW	0.0000E+00	GPM
REACTOR PRESSURE	1000.	PSIG
HPCI FLOW	0.0000E+00	GPM
LPCI A FLOW	0.0000E+00	GPM
LPCI B FLOW	0.0000E+00	GPM
"A" CORE SPRAY FLOW	0.0000E+00	GPM
"B" CORE SPRAY FLOW	0.0000E+00	GPM
DRYWELL SUMP LEVEL	15.00	Inch
STACK GAS RAD.	75.00	UCI/sec
RX BLDG VENT RAD.	19.00	UCI/sec
REFUEL FLR EXH RAD.	2.000	UCI/sec
TB BLDG VENT RAD.	9.000	UCI/sec
RW BLDG VENT RAD.	2.000	UCI/sec
SERVICE WATER RAD.	0.1500E-05	UCI/ml
STACK HI RANGE RAD.	0.2819	CI/sec
TB BLD HI RANGE RAD.	0.0000E+00	CI/sec
RW BLD HI RANGE RAD.	0.0000E+00	CI/sec
OFFGAS RAD.	14.00	mR/Hr
DRYWELL RAD.	8.000	R/Hr
HIGHEST MSL RAD MON	1500.	mR/Hr
DRYWELL PRESSURE	1.900	PSIG
DRYWELL TEMPERATURE	129.0	Deg F
TORUS WATER AVG TMP	76.00	Deg F
TORUS WATER LEVEL	13.91	FT
DRYWELL H2 CONC	0.3000	%
DRYWELL 02 CONC	1.000	%
CST LEVEL	250.0	Inch

Energency Director Approval: _____ TIME: _____

FORM SAP 1.4 Page 5 of 28

MESSAGE SHEET

MESSAGE NO. 5

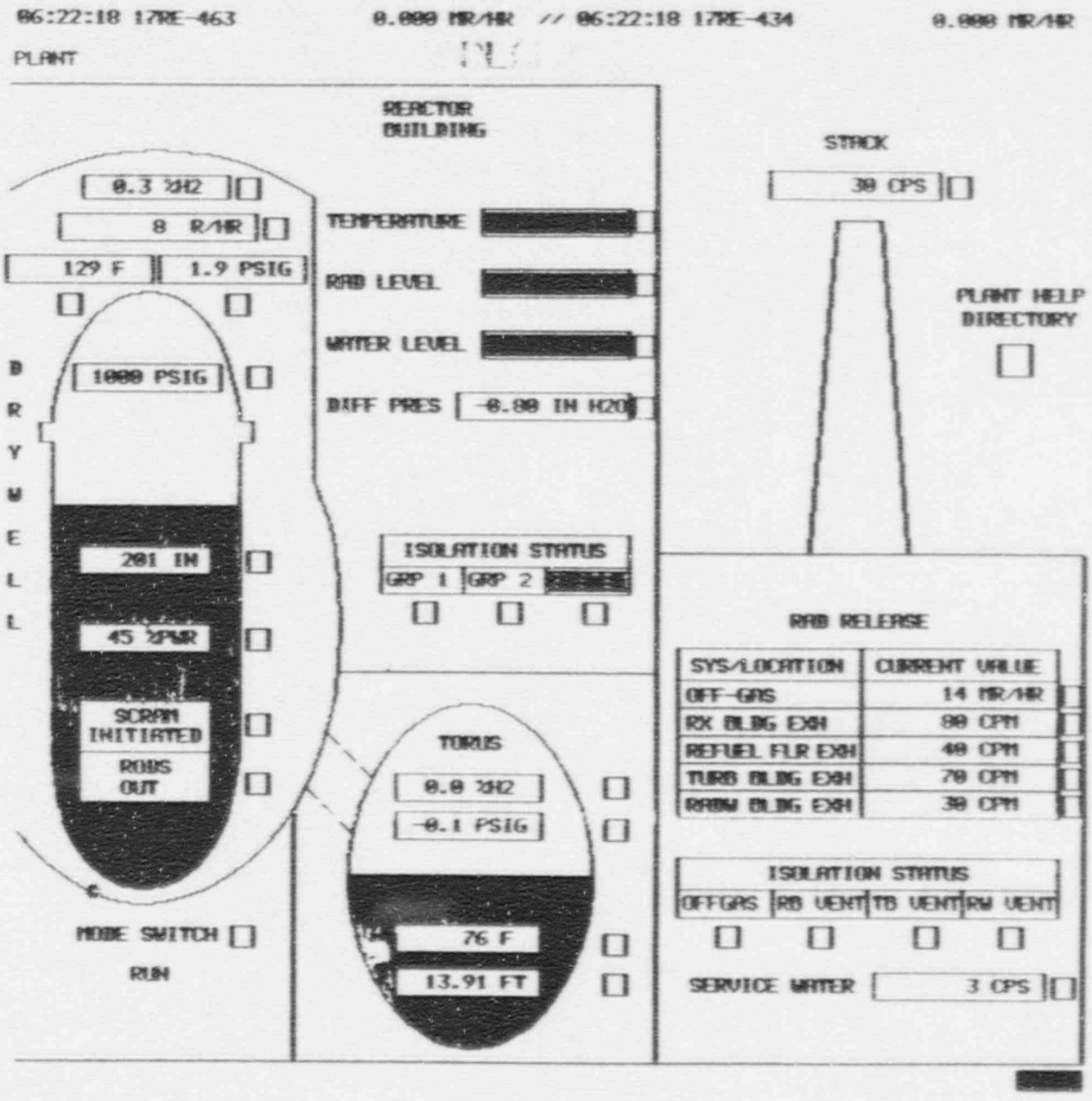
Exercise Time: 0745

Issued To: Control Room

Issued By: Lead Controller

MESSAGE

Plant parameters as indicated on SPDS display.


06:22:18 17RE-463
0.000 MR/HR
// 06:22:18 17RE-434
0.000 MR/HR
PLANT
REACTOR BUILDING
STACK
0.3 %H2
8 R/HR
129 F
1.9 PSIG
1000 PSIG
DRYWELL
TEMPERATURE
RAD LEVEL
WATER LEVEL
DIFF PRES
-0.00 IN H20
39 CPS
PLANT HELP DIRECTORY
ISOLATION STATUS
GRP 1
GRP 2
201 IN
45 %PWR
SCRAM INITIATED
RODS OUT
TORUS
0.0 %H2
-0.1 PSIG
76 F
13.91 FT
MODE SWITCH
RUN
RAD RELEASE
SYS/LOCATION
CURRENT VALUE
OFF-GAS
14 MR/HR
RX BLDG EXH
00 CPM
REFUEL FLR EXH
40 CPM
TURB BLDG EXH
70 CPM
RADW BLDG EXH
30 CPM
ISOLATION STATUS
OFFGAS
RB VENT
TB VENT
RW VENT
SERVICE WATER
3 GPS

Form ERP-1.1.3
James A. Fitzpatrick Nuclear Power Plant
PART III - Plant Parameters

Sequence No. _____

APRM REACTOR POWER	45.00	%
IRM REACTOR POWER	5.000	%
SRM REACTOR POWER	0.9000E+06	CPS
RX WATER LEVEL	201.0	Inch TAF
FEEDWATER FLOW	5.950	MLB/HR
RCIC FLOW	0.0000E+00	GPM
REACTOR PRESSURE	1000.	PSIG
HPCI FLOW	0.0000E+00	GPM
LPCI A FLOW	0.0000E+00	GPM
LPCI B FLOW	0.0000E+00	GPM
"A" CORE SPRAY FLOW	0.0000E+00	GPM
"B" CORE SPRAY FLOW	0.0000E+00	GPM
DRYWELL SUMP LEVEL	15.00	Inch
STACK GAS RAD.	75.00	UCI/sec
RX BLDG VENT RAD.	19.00	UCI/sec
REFUEL FLR EXH RAD.	2.000	UCI/sec
TB BLDG VENT RAD.	9.000	UCI/sec
RW BLDG VENT RAD.	2.000	UCI/sec
SERVICE WATER RAD.	0.1500E-05	UCI/ml
STACK HI RANGE RAD.	0.2819	CI/sec
TB BLD HI RANGE RAD.	0.0000E+00	CI/sec
RW BLD HI RANGE RAD.	0.0000E+00	CI/sec
OFFGAS RAD.	14.00	mR/Hr
DRYWELL RAD.	8.000	R/Hr
HIGHEST MSL RAD MON	1500.	mR/Hr
DRYWELL PRESSURE	1.900	PSIG
DRYWELL TEMPERATURE	129.0	Deg F
TORUS WATER AVG TMP	76.00	Deg F
TORUS WATER LEVEL	13.91	FT
DRYWELL H2 CONC	0.3900	%
DRYWELL 02 CONC	1.800	%
CST LEVEL	250.0	Inch

Emergency Director Approval: _____ TIME: _____

MESSAGE SHEET

MESSAGE NO. 6

Exercise Time: 0800

Issued To: Control Room

Issued By: Lead Controller

MESSAGE

The following annunciators alarm:

09-3-1-9 Fuel Pool Cool & Cln Up Trouble
09-3-1-10 Core Spray A or B Disch Line Not Full
09-3-1-18 RHR A or B Disch Line Not Full
09-3-1-19 ARM Downscale
09-3-1-20 Refuel Area ARM Rad Hi
09-3-1-29 New Fuel Storage ARM Rad Hi
09-3-1-30 Radw Bldg ARM Rad Hi
09-3-1-39 Turb Bldg ARM Rad Hi
09-3-1-40 Rx Bldg ARM Rad Hi
09-3-2-7 Off Gas Smpl Flow Hi or Lo
09-3-2-10 Off Gas Timer Initiated
09-3-2-17 Off Gas Rad Mon Downscale or Inop
09-3-2-19 Rx Bldg Vent Rad Mon Downscale
09-3-3-11 Main Stm Rad Mon Downscale
09-3-3-22 HPCI Stm Leak Det Logic Pwr Fail
09-4-0-1 Cont Hi Range Rad Mon A Fail or Pwr Loss
09-4-0-4 Cont Hi Range Rad Mon B Fail or Pwr Loss
09-4-0-21 PASS Gas Smpl Ht Trace A Trouble
09-4-0-22 PASS Gas Smpl Ht Trace B Trouble
09-4-0-23 PASS Exh Fan Trouble
09-4-1-16 SRV Leaking
09-4-1-19 Rx Bldg Equip Sump A Temp Hi
09-4-1-20 Rx Bldg Equip Sump B Temp Hi
09-4-2-1 DW Equip Drn Isol Vlv Closed
09-4-2-2 DW Floor Drn Isol Vlv Closed
09-4-2-3 PCIS Sys A Vlv Overload or Pwr Loss
09-4-2-6 SRV Sonic Mon Alarm Hi
09-4-2-11 DW Equip Sump Temp Hi or Cool Wtr Flow Lo
09-4-2-13 PCIS Sys B Vlv Overload or Pwr Loss
09-4-2-20 RWR MG A Drv Mtr Bkr Trip
09-4-2-21 DW Equip Sump HI-HI or LO-LO
09-4-2-36 TIP ARM Rad Hi
09-4-2-37 RWR MG A Oil Mist Elim Diff Press Hi
09-4-3-1 RWR MG A Gen Lockout
09-4-3-2 RWR Loop A Out of Serv
09-4-3-5 RWR Loop B Out of Serv

MESSAGE SHEET

MESSAGE NO. 6

Exercise Time: 0800 (cont)

Issued To: Control Room

Issued By: Lead Controller

MESSAGE

Annunciators continued:

09-4-3-10 RWR MG B Gen Lockout
09-4-3-11 RWR MG A Scoop Tube Lock
09-4-3-19 RWR MG B Drv Mtr Bkr Trip
09-4-3-20 RWR MG B Scoop Tube Lock
09-4-3-21 RWR MG A Gen Aux Lockout
09-4-3-30 RWR MG B Gen Aux Lockout
09-4-3-31 RWR MG A Speed Cntrl Sig Failed
09-4-3-36 RWR MG B Oil Mist Elim Diff Press Hi
09-4-3-40 RWR MG B Speed Cntrl Sig Failed
09-5-1-1 SDIV A or B Hi Lvl Trip
09-5-1-2 MSIVS Not Full Open Trip
09-5-1-3 RPS A Auto Scram
09-5-1-4 RPS B Auto Scram
09-5-1-5 RPS MG A Trouble
09-5-1-6 PCIS Sys A Rx Lvl LO-LO-LO
09-5-1-7 PCIS Sys B Rx Lvl LO-LO-LO
09-5-1-9 CRD Charging Wtr Press Lo
09-5-1-10 CRD Drv Wtr Filter Diff Press Hi
09-5-1-13 RPS A Man Scram
09-5-1-14 RPS B Man Scram
09-5-1-15 RPS Mg B Trouble
09-5-1-16 PCIS Sys A Stm Tun Temp Hi
09-5-1-17 PCIS Sys B Stm Tun Temp Hi
09-5-1-21 RPS Hi DW Press Trip
09-5-1-22 RPS Hi Rx Press Trip
09-5-1-24 Fdtwr Cntrl Lo Flow Vlv Elect Sig Fail
09-5-1-26 PCIS Sys A Main Stm Line Flow Hi
09-5-1-27 PCIS Sys B Main Stm Line Flow Hi
09-5-1-28 Rx Wtr Lvl Alarm Hi or Lo
09-5-1-31 RPS Rx Vessel Lo Lvl Trip
09-5-1-32 RPS Main Stm Line Hi Rad Trip
09-5-1-36 Sys A Stm Line Press Lo
09-5-1-37 Sys B Stm Line Press Lo
09-5-1-41 Neutron Mon Sys Trip
09-5-1-43 CRD Accum Press Lo or Lvl Hi
09-5-1-44 SDIV A or B Not Drained

MESSAGE SHEET

MESSAGE NO. 6

Exercise Time: 0800 (cont)

Issued To: Control Room

Issued By: Lead Controller

MESSAGE

Annunciators continued:

09-5-1-51 TCV Fast Closure Trip
09-5-1-53 TSV Closure Trip
09-5-1-54 Scram Air Hdr Press Hi or Lo
09-5-1-55 PCIS Sys A Group I Isol
09-5-1-56 PCIS Sys B Group I Isol
09-5-1-58 Interposing Relay Sys on DC Pwr
09-5-2-2 Rod Withdrawal Block
09-5-2-3 Rod Drift
09-5-2-6 EHC Lo Press Trip
09-5-2-9 Main Turb Trip
09-5-2-16 Main Turb Exh Hood Hi Temp Trip
09-5-2-20 Main Turb Exh Hood Temp Hi
09-5-2-22 RSCS In Serv
09-5-2-25 Flow Ref Off Norm
09-5-2-26 Main Turb Lo Vac Trip
09-5-2-27 Main Turb Loss of Stator Coolant Trip
09-5-2-30 Main Turb Bellows Fail or Lo Vac Trip
09-5-2-32 IRM Downscale
09-5-2-34 APRM Downscale
09-5-2-35 RBM Downscale
09-5-2-36 Main Turb Hi Vib Trip
09-5-2-44 APRM Upscale
09-5-2-45 RBM Upscale or Inop
09-5-2-54 APRM Trip Sys A Inop or Upscale Hi
09-5-2-55 APRM Trip Sys B Inop or Upscale Trip
09-5-2-57 ATTS RPS Div A1 or A2 Pwr Supp Fail or TU Out of File
09-5-2-58 ATTS RPS Div A1 or A2 Gross Fail or TU Inop
09-5-2-59 ATTS RPS Div B1 or B2 Pwr Supp Fail or TU Out of File
09-5-2-60 ATTS RPS Div B1 or B2 Gross Fail or TU Inop
09-6-1-3 Circ Wtr Pmp 36P-1A Overload or Trip
09-6-1-10 Screen Wash Bstr Pmp 46P-6A or 6B Trip or Cntrl Pwr Loss
09-6-1-11 Circ Wtr Pmp 36P-1B Overload or Trip
09-6-1-14 Perimeter Drn Pmp Trouble
09-6-1-18 Trvlg Wtr Screen Trip or Cntrl Pwr Loss
09-6-1-19 Circ Wtr Pmp 36P-1C Overload or Trip
09-6-1-24 Stack Dilution Fan 26FN-2A or 2B Fail

MESSAGE SHEET

MESSAGE NO. 6

Exercise Time: 0800 (cont)

Issued To: Control Room

Issued By: Lead Controller

MESSAGE

Annunciators continued:

09-6-1-29 CNDSR Vac Lo
09-6-1-30 SJAE Stm Press Lo
09-6-1-31 Serv Air Hdr Press Lo
09-6-2-2 RBC Hx Disch Temp Hi or Lo
09-6-2-9 Cond Xfer Pmp 33P-13A Lockout or Trip
09-6-2-12 RBC Conduct Hi
09-6-2-19 Cond Xfer Pmp 33P-13B Lockout or Trip
09-6-2-22 RBC Makeup Tk Lvl Hi or Lo
09-6-2-29 Cond Xfer Hdr Press Lo
09-6-2-30 CNDSR Hotwell Lvl Hi or Lo
09-6-2-31 RBC Hdr Press Lo
09-6-2-32 RBC Hdr Press LO-LO
09-6-2-34 Serv Wtr Hdr Press Lo
09-6-2-36 TBC Hdr Press Lo
09-6-2-38 Breathing Air Hdr Press Lo
09-6-2-39 Serv Air Hdr Isol Vlv Closed
09-6-2-40 Breathing Air Hdr Isol Vlv Closed
09-6-3-20 RFP A & B Seal Wtr Diff Press Lo
09-6-3-30 SJAE or SPE Drn Tk Lvl Hi or Lo
09-6-3-31 Cond Pmp Disch Hdr Press Lo
09-6-3-32 Cond Demin Sys Trouble
09-6-4-1 RFPT A Vac Lo
09-6-4-2 RFPT A Trip
09-6-4-3 RFPT A Oil Clr Disch Oil Temp Hi
09-6-4-4 RFPT A or B Aux Oil Pmp Trouble
09-6-4-6 RFPT A or B Turning Gear Trouble
09-6-4-8 RFPT B Vac Lo
09-6-4-9 RFPT B Trip
09-6-4-10 RFPT B Oil Clr Disch Oil Temp Hi
09-6-4-15 RFPT A or B Emerg Brg Oil Pmp Running
09-7-1-3 Main Gen Auto Reg Trip to Man
09-7-1-5 Unit Prim Lockout
09-7-1-15 Main XSFMR T-18 Temp Hi
09-7-1-26 Main Gen Exciter Filed Bkr Trip
09-7-1-28 Main Gen Field Grd Will Alarm Only
09-7-1-31 Main XSFMR T-18 Trouble

MESSAGE SHEET

MESSAGE NO. 6

Exercise Time: 0800 (cont)

Issued To: Control Room

Issued By: Lead Controller

MESSAGE

Annunciators continued:

09-7-2-1 Main Gen Disc 10031 Open
09-7-2-3 Fitz-Edic 345KV Line 1 Bkr 10052 Trip
09-7-2-7 Fitz-Scriba 345KV Line 10 Bkr 10042 Trip
09-7-2-8 Main Turb Brg Oil Vapor Extractor Trouble
09-7-2-10 Reboiler Lvl Cntrl Trouble
09-7-2-16 EHC Fluid Pwr Unit Tk Trouble
09-7-2-18 Reboiler Trouble
09-7-2-21 345KV Bkr 10042 Ext Dry Time or Loss of AC for Htrs
09-7-2-32 Main Gen Core Mon Trouble
09-7-3-1 Main Turb Brg 3 Lift Pmp Trouble
09-7-3-2 Main Turb Brg 4 Lift Pmp Trouble
09-7-3-3 Main Turb Mtr Drvn Suct Oil Pmp Trouble
09-7-3-8 Main Gen Stator Wtr Sys Trouble
09-7-3-13 Main Turb Brg 5 Lift Pmp Trouble
09-7-3-14 Main Turb Brg 6 Lift Pmp Trouble
09-7-3-19 Main Gen Emerg Seal Oil Pmp Running
09-7-3-22 Moist Sep Drn Tk 2A Lvl Lo
09-7-3-24 2nd Stage RHTR Drn Tk 4A Lvl Lo
09-7-3-25 Main Turb Brg 7 Lift Pmp Trouble
09-7-3-26 Main Turb Brg 8 Lift Pmp Trouble
09-7-3-29 EHC Pmp A or B Trip or Cntrl Pwr Loss
09-7-3-32 SPE Fan Overload Cntrl Volts Lo
09-7-3-38 Main Turb Turning Gear Oil Pmp Running
09-7-3-40 Main Stm Leads Drn Pot Lvl Hi
09-7-3-43 SPE Vac Lo
09-7-3-44 Seal Stm Hdr Press Lo
09-7-3-46 Moist Sep Drn Tk 2B Lvl Lo
09-7-3-48 2nd Stage RHTR Drn Tk 4B Lvl Lo
09-8-1-4 Computer Cntrl Volt Trouble
09-8-1-8 UPS on DC Drive
09-8-2-4 EDG C Load Bkr 10512 Closed
09-8-2-8 Bus 10300-10500 Tie Bkr 10514 Trip or Cntrl Pwr Loss
09-8-2-9 EDG A Eng Trouble or Shutdown
09-8-3-9 10100 or 10200 or 10300 or 10400 Prolong UV Trip
09-8-3-17 Bus 10300-10500 4KV Tie Bkr 10304 Trip
09-8-3-18 L13 L23 L33 L43 4KV Supp Fdr Bkr 10340 Trip

MESSAGE SHEET

MESSAGE NO. 6

Exercise Time: 0800 (cont)

Issued To: Control Room

Issued By: Lead Controller

MESSAGE

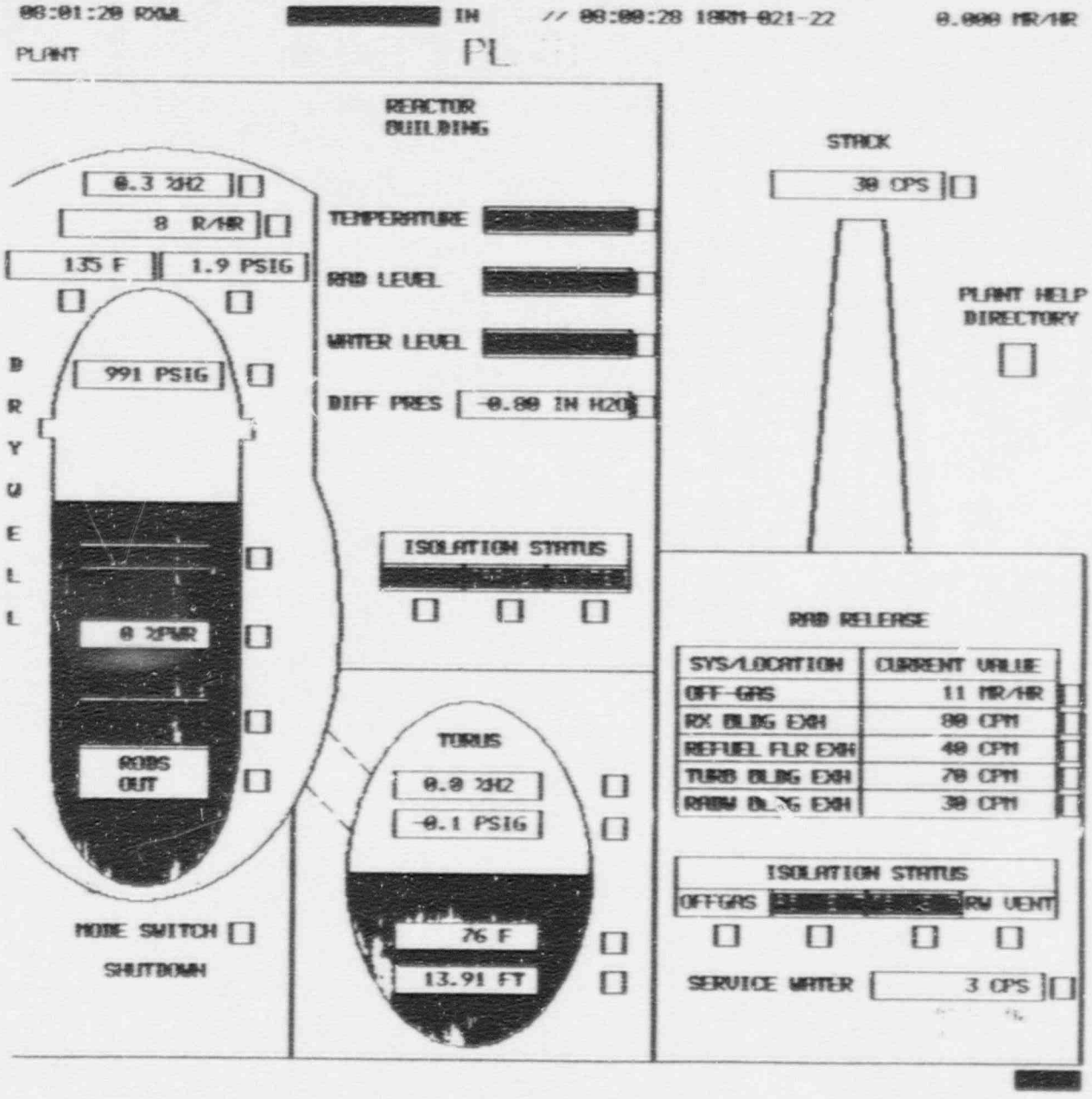
Annunciators continued:

09-8-3-23 L14 L24 L34 L44 4KV Supp Fdr Bkr 10440 Trip
09-8-3-24 416OV Bus 10500 Degraded Voltage Timer Initiated
09-8-3-26 Bus 10300 Norm Supp Bkr 10302 Trip
09-8-3-32 Bus 10400 Norm Supp Bkr 10402 Trip
09-8-4-1 Bus 10400-10600 Tie Bkr 10614 Trip or Cntrl Pwr Loss
09-8-4-3 EDG B Load Bkr 10602 Closed
09-8-4-6 EDG D Load Bkr 10612 Closed
09-8-4-11 EDG B Eng Trouble or Shutdown
09-8-4-14 EDG D Eng Trouble or Shutdown
09-8-5-3 Bus 10100 Norm Supp Bkr 10102 Trip
09-8-5-13 Bus 10700 Supp Bkr 10702 Trip
09-8-5-15 Bus 10200 Norm Supp Bkr 10202 Trip
09-8-6-1 LHH-Fitz 115KV Line 3 Prim Relay Trip
09-8-6-10 LHH-Fitz 115KV Line 3 Bkr 10022 Air Loss or AC Pwr Loss
09-8-6-14 115KV SYS Volt Loss
09-8-6-15 NMP-Fitz 115KV Line 4 Bkr 10012 Trip

HPCI/RCIC auto initiate and control level.

"B", "C" and "D" EDG's start and load the 10500 and 10600 buses.

Plant parameters as indicated on SPDS display.


08:01:20 RXML
IN
// 08:00:28 1801-821-22
0.000 MR/HR
PLANT
PL
REACTOR BUILDING
STACK
0.3 %H2
8 R/HR
135 F
1.9 PSIG
TEMPERATURE
RAD LEVEL
WATER LEVEL
DIFF PRES
-0.80 IN H20
30 CPS
PLANT HELP DIRECTORY
DRYWELL
991 PSIG
0 %PWR
RODS OUT
ISOLATION STATUS
TORUS
0.0 %H2
-0.1 PSIG
76 F
13.91 FT
RAD RELEASE
SYS/LOCATION
CURRENT VALUE
OFF-GAS
11 MR/HR
RX BLDG EXH
80 CPM
REFUEL FLR EXH
40 CPM
TURB BLDG EXH
70 CPM
RADW BLDG EXH
30 CPM
ISOLATION STATUS
OFFGAS
RW VENT
MODE SWITCH
SHUTDOWN
SERVICE WATER
3 GPS

08:01:20 RMM [redacted] IM // 08:00:28 1801-021-22 0.000 MR/HR

Form EPP-1.1.3
James A. Fitzpatrick Nuclear Power Plant
PART III - Plant Parameters

Sequence No. _____

APRM REACTOR POWER	0.0000E+00	%
IRM REACTOR POWER	0.0000E+00	%
SRM REACTOR POWER	0.0000E+00	CPS
RX WATER LEVEL	[redacted]	Inch TAF
FEEDWATER FLOW	0.3000	MLB/HR
RCIC FLOW	0.0000E+00	GPM
REACTOR PRESSURE	998.2	PSIG
HPCI FLOW	0.0000E+00	GPM
LPCI A FLOW	0.0000E+00	GPM
LPCI B FLOW	0.0000E+00	GPM
"A" CORE SPRAY FLOW	0.0000E+00	GPM
"B" CORE SPRAY FLOW	0.0000E+00	GPM
DRYWELL SUMP LEVEL	15.00	Inch
STACK GAS RAD.	75.00	UCI/sec
RX BLDG VENT RAD.	19.00	UCI/sec
REFUEL FLR EXH RAD.	2.000	UCI/sec
TB BLDG VENT RAD.	9.000	UCI/sec
RW BLDG VENT RAD.	2.000	UCI/sec
SERVICE WATER RAD.	0.1500E-05	UCI/ml
STACK HI RANGE RAD.	0.2819	CI/sec
TB BLD HI RANGE RAD.	0.0000E+00	CI/sec
RW BLD HI RANGE RAD.	0.0000E+00	CI/sec
OFFGAS RAD.	13.38	mR/Hr
DRYWELL RAD.	7.994	R/Hr
HIGHEST MSL RAD MON	1586.	mR/Hr
DRYWELL PRESSURE	1.900	PSIG
DRYWELL TEMPERATURE	132.6	Deg F
TORUS WATER AVG TMP	76.00	Deg F
TORUS WATER LEVEL	13.91	FT
DRYWELL H2 CONC	0.3000	%
DRYWELL O2 CONC	1.000	%
CST LEVEL	258.0	Inch

Emergency Director Approval: _____ TIME: _____

[redacted]

MESSAGE SHEET

MESSAGE NO. 7

Exercise Time: 0801

Issued To: Control Room

Issued By: Lead Controller

MESSAGE

The following annunciators are sealed in:

09-3-3-2 DIV I Ambient Temp Hi
09-3-3-12 Div II Ambient Temp Hi
09-5-1-16 PCIS Sys A Stm Tun Temp Hi
09-5-1-17 PCIS Sys B Stm Tun Temp Hi

09-4 panel dual position indications for 29MOV-74, -77 and -79.

FORM SAP 1.4 Page 13 of 28

MESSAGE SHEET

MESSAGE NO. 8

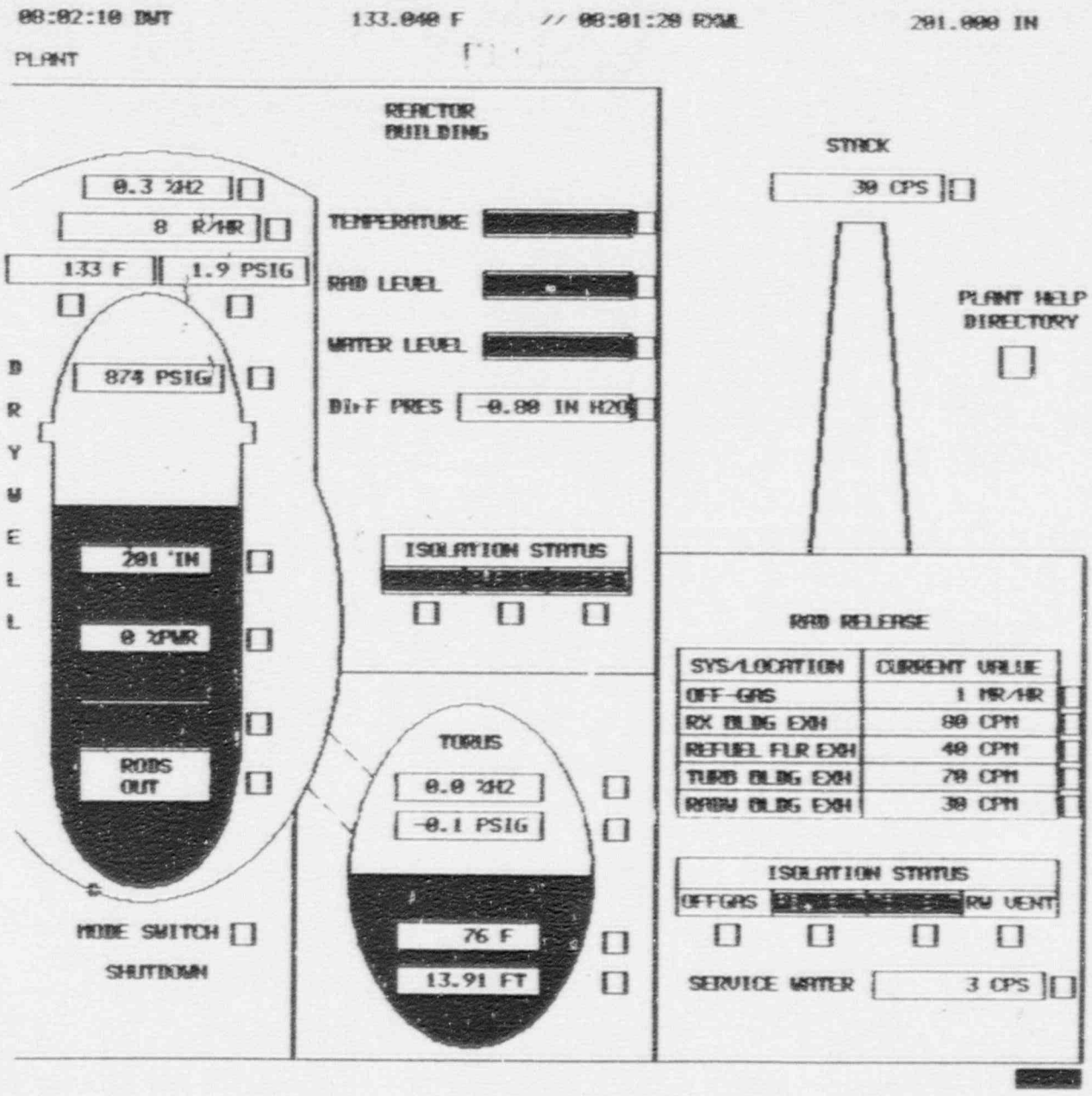
Exercise Time: 0815

Issued To: Control Room

Issued By: Lead Controller

MESSAGE

Plant parameters as indicated on SPDS display.


08:02:10 EST
133.040 F
// 08:01:20 RXML
201.000 IN
PLANT
REACTOR BUILDING
0.3 %H2
8 R/HR
133 F
1.9 PSIG
874 PSIG
201 IN
0 %PWR
RODS OUT
DRYWELL
TEMPERATURE
RAD LEVEL
WATER LEVEL
DIFF PRES
-0.80 IN H2O
ISOLATION STATUS
TORUS
0.0 %H2
-0.1 PSIG
76 F
13.91 FT
MODE SWITCH
SHUTDOWN
STACK
30 CPS
PLANT HELP DIRECTORY
RAD RELEASE
SYS/LOCATION
CURRENT VALUE
OFF-GAS
1 MR/HR
RX BLDG EXH
80 CPM
REFUEL FLR EXH
40 CPM
TURB BLDG EXH
70 CPM
RADW BLDG EXH
30 CPM
ISOLATION STATUS
OFFGAS
RW VENT
SERVICE WATER
3 GPS

08:02:10 DWT 132.980 F // 08:01:20 RXWL 201.000 IN

Form EAP-1.1.3
James A. Fitzpatrick Nuclear Power Plant
PART III - Plant Parameters

Sequence No. __________

APRM REACTOR POWER	0.0000E+00	%
IRM REACTOR POWER	0.0000E+00	%
SRM REACTOR POWER	0.0000E+00	CPS
RX WATER LEVEL	201.0	Inch TAF
FEEDWATER FLOW	0.3000	MLB/HR
RCIC FLOW	380.0	GPM
REACTOR PRESSURE	866.7	PSIG
HPCI FLOW	0.0000E+00	GPM
LPCI A FLOW	0.0000E+00	GPM
LPCI B FLOW	0.0000E+00	GPM
"A" CORE SPRAY FLOW	0.0000E+00	GPM
"B" CORE SPRAY FLOW	0.0000E+00	GPM
DRYWELL SUMP LEVEL	15.00	Inch
STACK GAS RAD.	75.00	UCI/sec
RX BLDG VENT RAD.	19.00	UCI/sec
RFUEL FLR EXH RAD.	2.000	UCI/sec
TB BLDG VENT RAD.	9.000	UCI/sec
RW BLDG VENT RAD.	2.000	UCI/sec
SERVICE WATER RAD.	0.1500E-05	UCI/ml
STACK HI RANGE RAD.	0.2819	CI/sec
TB BLD HI RANGE RAD.	0.0000E+00	CI/sec
RW BLD HI RANGE RAD.	0.0000E+00	CI/sec
OFFGAS RAD.	1.000	mR/Hr
DRYWELL RAD.	7.570	R/Hr
HIGHEST MSL RAD MON	1960.	mR/Hr
DRYWELL PRESSURE	1.900	PSIG
DRYWELL TEMPERATURE	133.0	Deg F
TORUS WATER AVG TMP	76.37	Deg F
TORUS WATER LEVEL	13.91	FT
DRYWELL H2 CONC	0.3000	%
DRYWELL O2 CONC	1.800	%
CST LEVEL	250.0	Inch

Emergency Director Approval: ____________________ TIME: __________

[redacted]

MESSAGE SHEET

MESSAGE NO. 9

Exercise Time: 0830

Issued To: Control Room

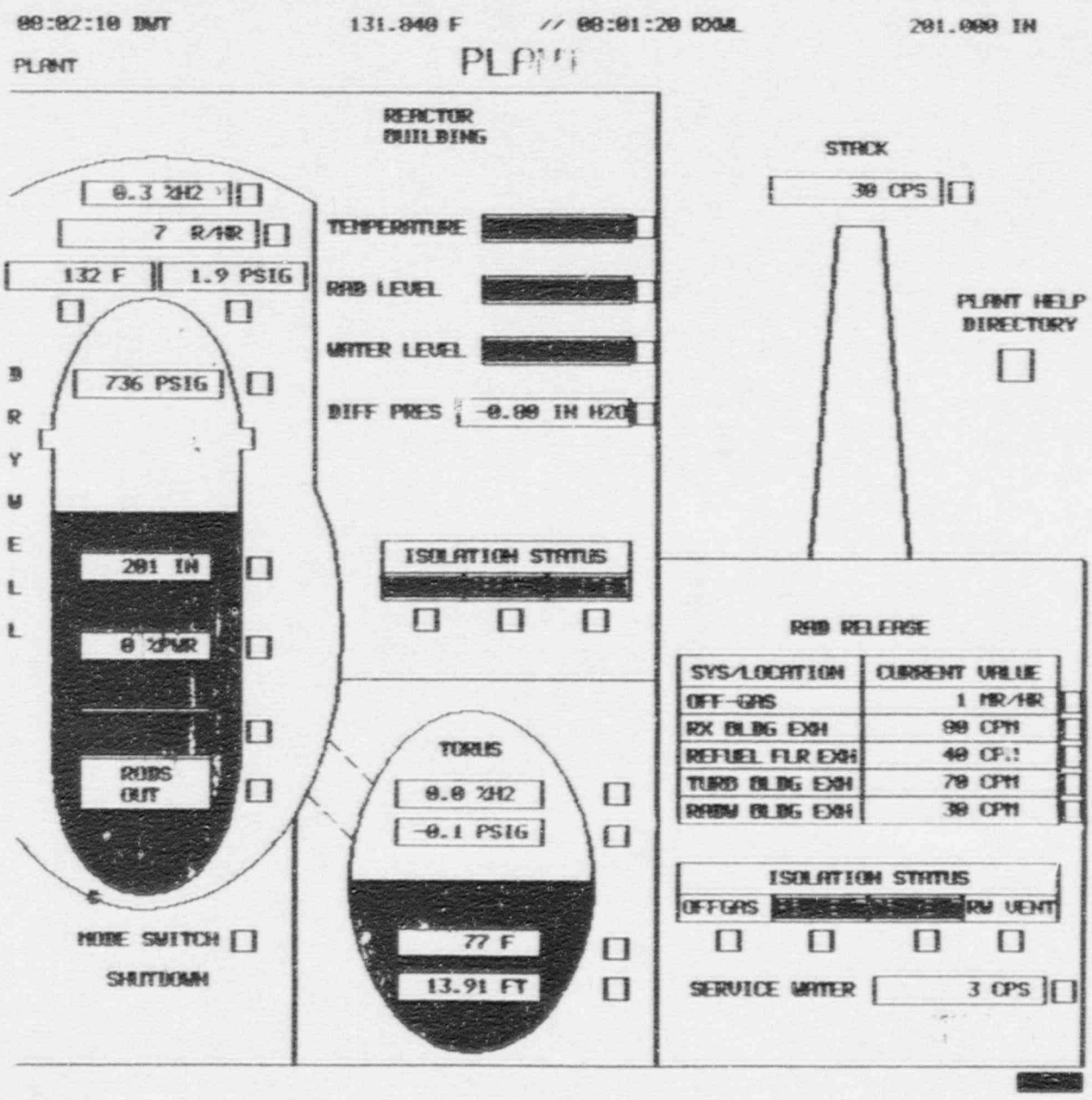
Issued By: Lead Controller

MESSAGE

The following annunciators alarm:

09-7-3-42 Hydrogen Seal Oil Trouble

Plant parameters as indicated on SPDS display.


08:02:10 EST
131.840 F
// 08:01:20 RAM
201.000 IN
PLANT
PLANT
REACTOR BUILDING
0.3 %H2
7 R/HR
132 F
1.9 PSIG
736 PSIG
201 IN
0 %PWR
RODS OUT
DRYWELL
TEMPERATURE
RAD LEVEL
WATER LEVEL
DIFF PRES
-0.00 IN H2O
ISOLATION STATUS
MODE SWITCH
SHUTDOWN
TORUS
0.0 %H2
-0.1 PSIG
77 F
13.91 FT
STACK
30 CPS
PLANT HELP DIRECTORY
RAD RELEASE
SYS/LOCATION
CURRENT VALUE
OFF-GAS
1 MR/HR
RX BLDG EXH
99 CPM
REFUEL FLR EXH
40 CPM
TURB BLDG EXH
70 CPM
RADW BLDG EXH
30 CPM
ISOLATION STATUS
OFFGAS
RW VENT
SERVICE WATER
3 GPS

08:02:10 EST 131.000 F // 08:01:20 RCGL 201.000 IN

Form EAP-1.1.3
James A. Fitzpatrick Nuclear Power Plant
PART III - Plant Parameters

Sequence No. _____

APRM REACTOR POWER	0.0000E+00	%
IRM REACTOR POWER	0.0000E+00	%
SRM REACTOR POWER	0.0000E+00	CPS
RX WATER LEVEL	201.0	Inch TAF
FEEDWATER FLOW	0.3000	MLB/HR
RCIC FLOW	300.0	GPM
REACTOR PRESSURE	730.2	PSIG
HPCI FLOW	0.0000E+00	GPM
LPCI A FLOW	0.0000E+00	GPM
LPCI B FLOW	0.0000E+00	GPM
"A" CORE SPRAY FLOW	0.0000E+00	GPM
"B" CORE SPRAY FLOW	0.0000E+00	GPM
DRYWELL SUMP LEVEL	15.00	Inch
STACK GAS RAD.	75.00	UCI/sec
RX BLDG VENT RAD.	19.00	UCI/sec
REACT FLR EXH RAD.	2.000	UCI/sec
TB BLDG VENT RAD.	9.000	UCI/sec
RW BLDG VENT RAD.	2.000	UCI/sec
SERVICE WATER RAD.	0.1500E-05	UCI/ml
STACK HI RANGE RAD.	0.2819	CI/sec
TB BLD HI RANGE RAD.	0.0000E+00	CI/sec
RW BLD HI RANGE RAD.	0.0000E+00	CI/sec
OFFGAS RAD.	1.000	mR/Hr
DRYWELL RAD.	7.134	R/Hr
HIGHEST MSL RAD MON	2428.	mR/Hr
DRYWELL PRESSURE	1.960	PSIG
DRYWELL TEMPERATURE	131.8	Deg F
TORUS WATER AVG TMP	76.74	Deg F
TORUS WATER LEVEL	13.91	FT
DRYWELL H2 CONC	0.3000	%
DRYWELL O2 CONC	1.000	%
CST LEVEL	250.0	Inch

Energency Director Approval: _____ TIME: _____

FORM SAP 1.4 Page 15 of 28

MESSAGE SHEET

MESSAGE NO. 10

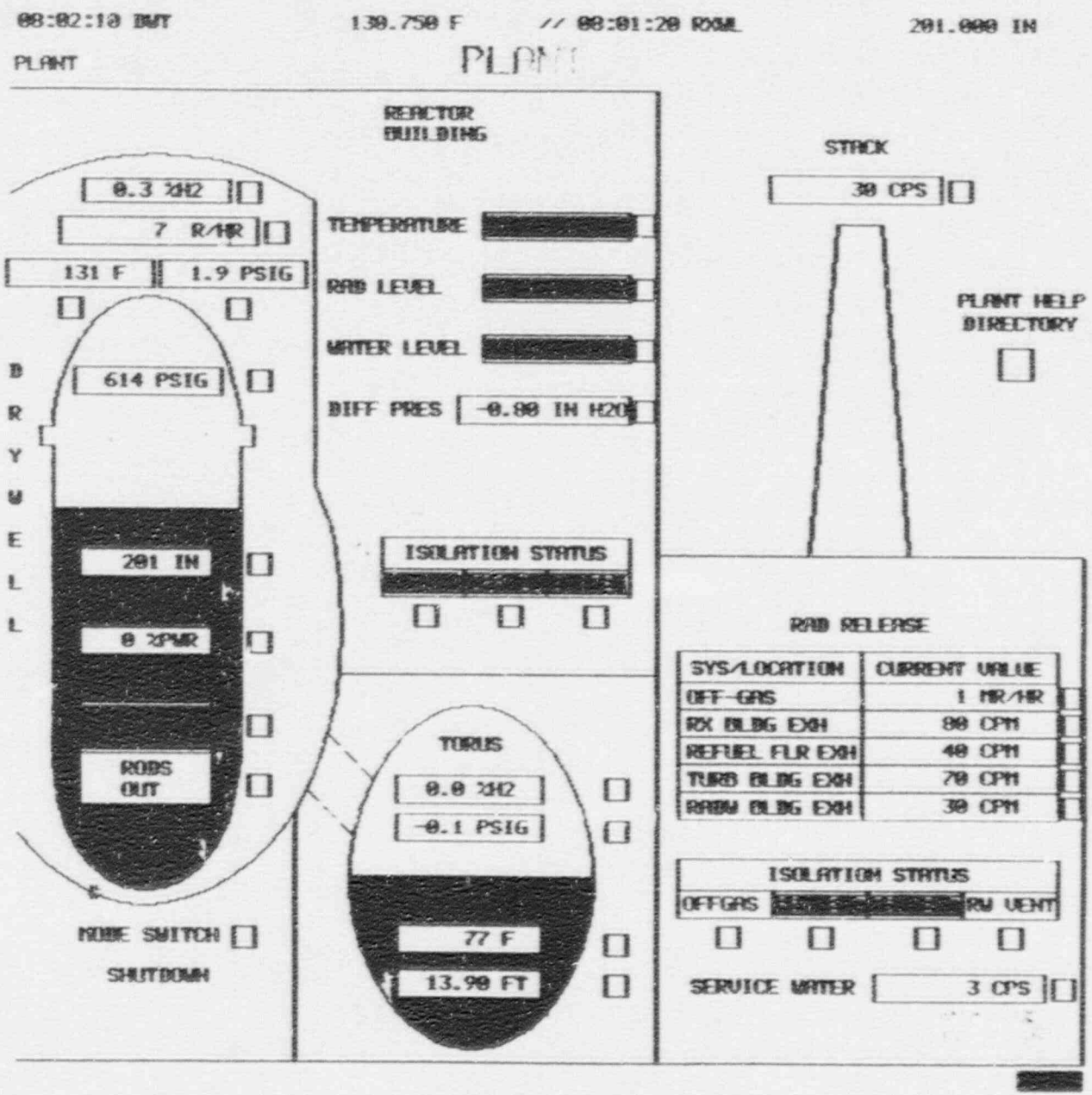
Exercise Time: 0845

Issued To: Control Room

Issued By: Lead Controller

MESSAGE

Plant parameters as indicated on SPDS display.


08:02:10 BMT
139.750 F
// 08:01:20 RXML
201.000 IN
PLANT
PLANT
REACTOR BUILDING
0.3 %H2
7 R/HR
131 F
1.9 PSIG
614 PSIG
201 IN
0 %PWR
RODS OUT
DRYWELL
TEMPERATURE
RAD LEVEL
WATER LEVEL
DIFF PRES
-0.80 IN H20
ISOLATION STATUS
MODE SWITCH
SHUTDOWN
TORUS
0.0 %H2
-0.1 PSIG
77 F
13.90 FT
STACK
30 CPS
PLANT HELP DIRECTORY
RAD RELEASE
SYS/LOCATION
CURRENT VALUE
OFF-GAS
1 MR/HR
RX BLDG EXH
80 CPM
REFUEL FLR EXH
40 CPM
TURB BLDG EXH
70 CPM
RADW BLDG EXH
30 CPM
ISOLATION STATUS
OFFGAS
RW VENT
SERVICE WATER
3 CPS

Form EPP-1.1.3
James A. Fitzpatrick Nuclear Power Plant
PART III - Plant Parameters

Sequence No. _____

APRM REACTOR POWER	0.0000E+00	%
IRM REACTOR POWER	0.0000E+00	%
SRM REACTOR POWER	0.0000E+00	CPS
RX WATER LEVEL	201.0	Inch, TAF
FEEDWATER FLOW	0.3000	MLB/HR
RCIC FLOW	300.0	GPM
REACTOR PRESSURE	687.0	PSIG
HPCI FLOW	0.0000E+00	GPM
LPCI A FLOW	0.0000E+00	GPM
LPCI B FLOW	0.0000E+00	GPM
"A" CORE SPRAY FLOW	0.0000E+00	GPM
"B" CORE SPRAY FLOW	0.0000E+00	GPM
DRYWELL SUMP LEVEL	15.00	Inch
STACK GAS RAD.	75.00	UCI/sec
RX BLDG VENT RAD.	19.00	UCI/sec
REFUEL FLR EXH RAD.	2.000	UCI/sec
TB BLDG VENT RAD.	9.000	UCI/sec
RW BLDG VENT RAD.	2.000	UCI/sec
SERVICE WATER RAD.	0.1500E-05	UCI/ml
STACK HI RANGE RAD.	0.2819	CI/sec
TB BLD HI RANGE RAD.	0.0000E+00	CI/sec
RW BLD HI RANGE RAD.	0.0000E+00	CI/sec
OFFGAS RAD.	1.000	mR/Hr
DRYWELL RAD.	6.734	R/Hr
HIGHEST MSL RAD MON	2850.	mR/Hr
DRYWELL PRESSURE	1.900	PSIG
DRYWELL TEMPERATURE	139.7	Deg F
TORUS WATER AVG TMP	77.00	Deg F
TORUS WATER LEVEL	13.90	FT
DRYWELL H2 CONC	0.3000	%
DRYWELL 02 CONC	1.000	%
CST LEVEL	250.0	Inch

Emergency Director Approval: _____ TIME: _____

FORM SAP 1.4 Page 16 of 28

MESSAGE SHEET

MESSAGE NO. 11

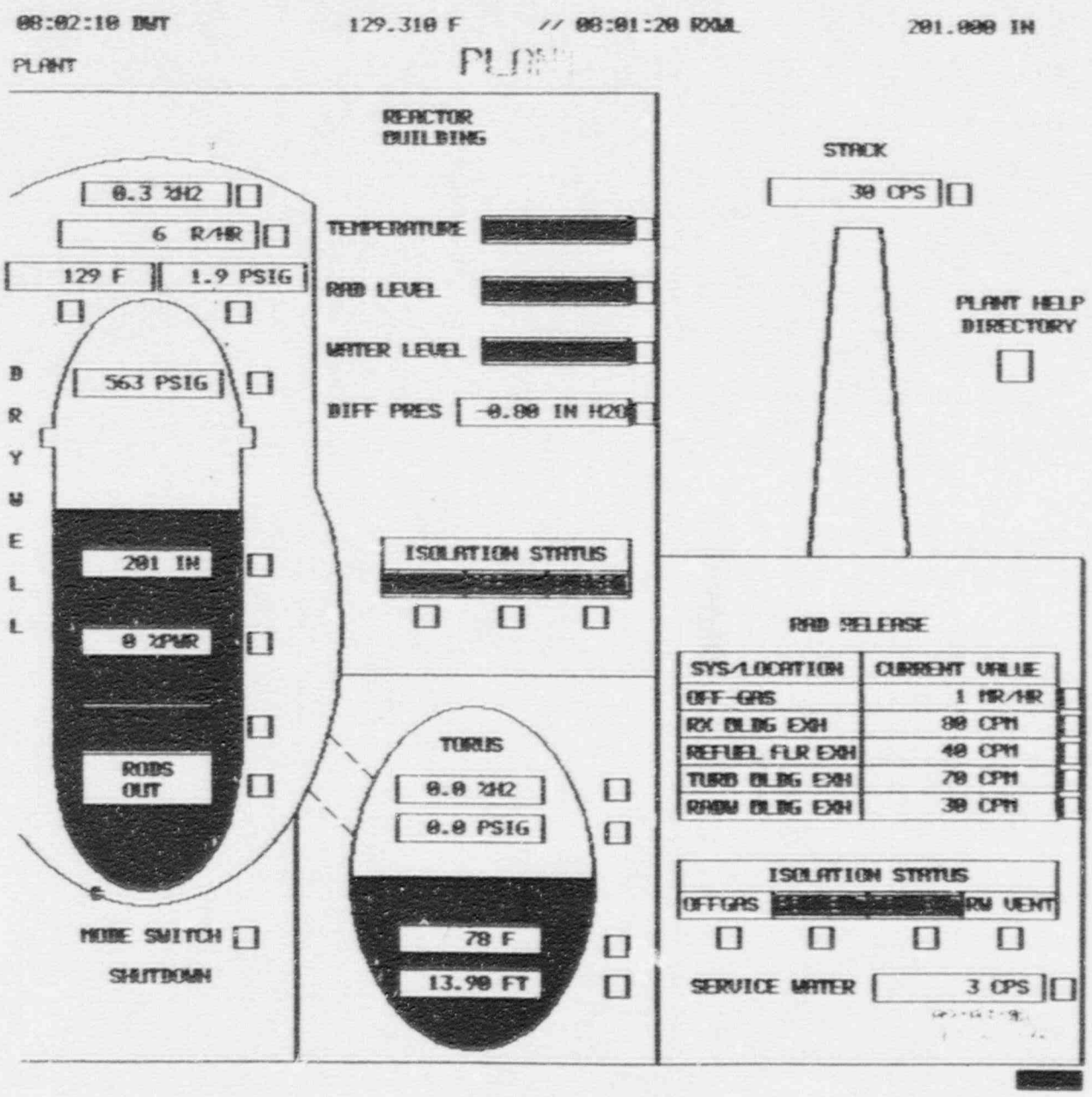
Exercise Time: 0900

Issued To: Control Room

Issued By: Lead Controller

MESSAGE

Plant parameters as indicated on SPDS display.


08:02:10 EST
129.310 F
// 08:01:20 RXML
201.000 IN
PLANT
PLANT
REACTOR BUILDING
0.3 %H2
6 R/HR
129 F
1.9 PSIG
563 PSIG
201 IN
0 %PWR
RODS OUT
DRYWELL
TEMPERATURE
ROD LEVEL
WATER LEVEL
DIFF PRES
-0.00 IN H2O
ISOLATION STATUS
TORUS
0.0 %H2
0.0 PSIG
78 F
13.90 FT
MODE SWITCH
SHUTDOWN
STACK
30 CPS
PLANT HELP DIRECTORY
RAD RELEASE
SYS/LOCATION
CURRENT VALUE
OFF-GAS
1 MR/HR
RX BLDG EXH
80 CPM
REFUEL FLR EXH
40 CPM
TURB BLDG EXH
70 CPM
RADW BLDG EXH
30 CPM
ISOLATION STATUS
OFFGAS
RW VENT
SERVICE WATER
3 CPS

08:02:10 EST 129.390 F // 08:01:20 RCML 201.000 IN

Form EEP-1.1.3
James A. Fitzpatrick Nuclear Power Plant
PART III - Plant Parameters

Sequence No. _____

APRM REACTOR POWER	0.0000E+00	%
IRM REACTOR POWER	0.0000E+00	%
SRM REACTOR POWER	6.0000E+00	CPS
RX WATER LEVEL	201.0	Inch TAF
FEEDWATER FLOW	0.3000	MLB/HR
RCIC FLOW	300.0	GPM
REACTOR PRESSURE	565.8	PSIG
HPCI FLOW	0.0000E+00	GPM
LPCI A FLOW	0.0000E+00	GPM
LPCI B FLOW	0.0000E+00	GPM
"A" CORE SPRAY FLOW	0.0000E+00	GPM
"B" CORE SPRAY FLOW	0.0000E+00	GPM
DRYWELL SUMP LEVEL	15.00	Inch
STACK GAS RAD.	75.00	UCI/sec
RX BLDG VENT RAD.	19.00	UCI/sec
REFUEL FLR EXH RAD.	2.000	UCI/sec
TB BLDG VENT RAD.	9.000	UCI/sec
RW BLDG VENT RAD.	2.000	UCI/sec
SERVICE WATER RAD.	0.1500E-05	UCI/ml
STACK HI RANGE RAD.	0.2819	CI/sec
TB BLD HI RANGE RAD.	0.1858	CI/sec
RW BLD HI RANGE RAD.	0.0000E+00	CI/sec
OFFGAS RAD.	1.000	mR/Hr
DRYWELL RAD.	6.250	R/Hr
HIGHEST MSL RAD MEAS	337%.	mR/Hr
DRYWELL PRESSURE	1.900	PSIG
DRYWELL TEMPERATURE	129.4	Deg F
TORUS WATER AVG TMP	77.50	Deg F
TORUS WATER LEVEL	13.90	FT
DRYWELL H2 CONC	0.3000	%
DRYWELL O2 CONC	1.800	%
CST LEVEL	250.0	Inch

Emergency Director Approval: _____ TIME: _____

MESSAGE SHEET

MESSAGE NO. 12

Exercise Time: 0915

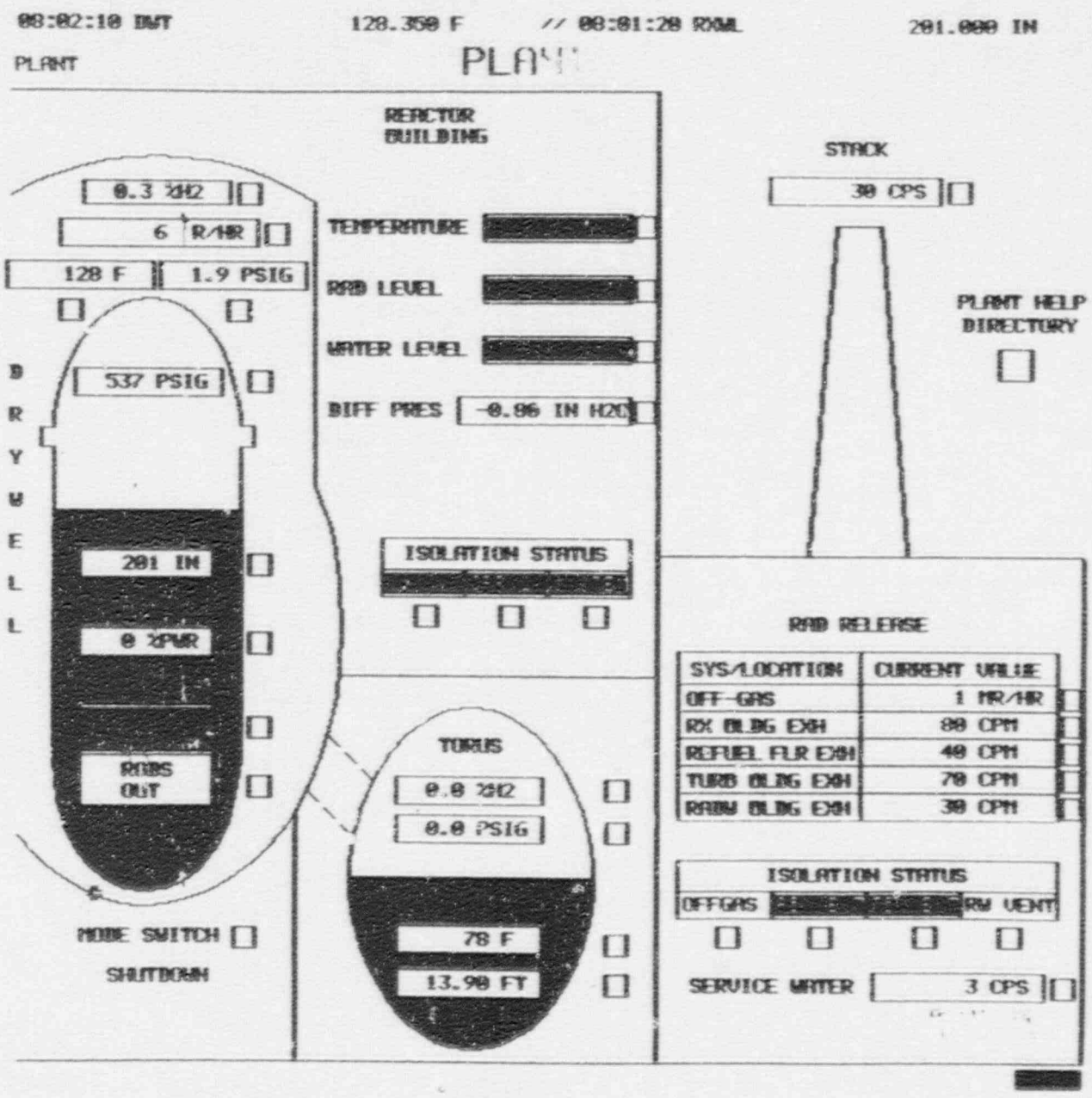
Issued To: Control Room

Issued By: Lead Controller

MESSAGE

NMP Line #4 is restored and T-3 transformer can be placed back in service.

Plant parameters as indicated on SPDS display.


08:02:10 EST
128.359 F
// 08:01:20 RXML
201.000 IN
PLANT
PLANT
REACTOR BUILDING
0.3 %H2
6 R/HR
128 F
1.9 PSIG
537 PSIG
201 IN
0 %PWR
RODS OUT
TEMPERATURE
ROD LEVEL
WATER LEVEL
DIFF PRES
-0.86 IN H2O
ISOLATION STATUS
DRYWELL
TORUS
0.0 %H2
0.0 PSIG
78 F
13.98 FT
MODE SWITCH
SHUTDOWN
STACK
30 CPS
PLANT HELP DIRECTORY
RAD RELEASE
SYS/LOCATION
CURRENT VALUE
OFF-GAS
1 MR/HR
RX BLDG EXH
80 CPM
REFUEL FLR EXH
40 CPM
TURB BLDG EXH
70 CPM
RADW BLDG EXH
30 CPM
ISOLATION STATUS
OFFGAS
RW VENT
SERVICE WATER
3 CPS

08:02:10 EST 128.200 F // 08:01:20 PDML 201.000 IN

Form ERP-1.1.3
James A. Fitzpatrick Nuclear Power Plant
PART III - Plant Parameters

Sequence No. _____

APRM REACTOR POWER	0.0000E+00	%
IRM REACTOR POWER	0.0000E+00	%
SRM REACTOR POWER	0.0000E+00	CPS
RX WATER LEVEL	201.0	Inch TAF
FEEDWATER FLOW	0.3000	MLB/HR
RCIC FLOW	380.0	GPM
REACTOR PRESSURE	534.8	PSIG
HPCI FLOW	0.0000E+00	GPM
LPCI A FLOW	0.0000E+00	GPM
LPCI B FLOW	0.0000E+00	GPM
"A" CORE SPRAY FLOW	0.0000E+00	GPM
"B" CORE SPRAY FLOW	0.0000E+00	GPM
DRYWELL SUMP LEVEL	15.00	Inch
STACK GAS RAD.	75.00	UCI/sec
RX BLDG VENT RAD.	19.00	UCI/sec
REFUEL FLR EXH RAD.	2.000	UCI/sec
TB BLDG VENT RAD.	9.000	UCI/sec
RW BLDG VENT RAD.	2.000	UCI/sec
SERVICE WATER RAD.	0.1500E-05	UCI/ml
STACK HI RANGE RAD.	0.2819	CI/sec
TB BLD HI RANGE RAD.	3.727	CI/sec
RW BLD HI RANGE RAD.	0.0000E+00	CI/sec
OFFGAS RAD.	1.000	mR/Hr
DRYWELL RAD.	5.852	R/Hr
HIGHEST MSL RAD MON	3881.	mR/Hr
DRYWELL PRESSURE	1.900	PSIG
DRYWELL TEMPERATURE	128.3	Deg F
TORUS WATER AVG TMP	77.84	Deg F
TORUS WATER LEVEL	13.90	FT
DRYWELL H2 CONC	0.3800	%
DRYWELL O2 CONC	1.800	%
CST LEVEL	250.0	Inch

Emergency Director Approval: _____ TIME: _____

MESSAGE SHEET

MESSAGE NO. 13

Exercise Time: 0930

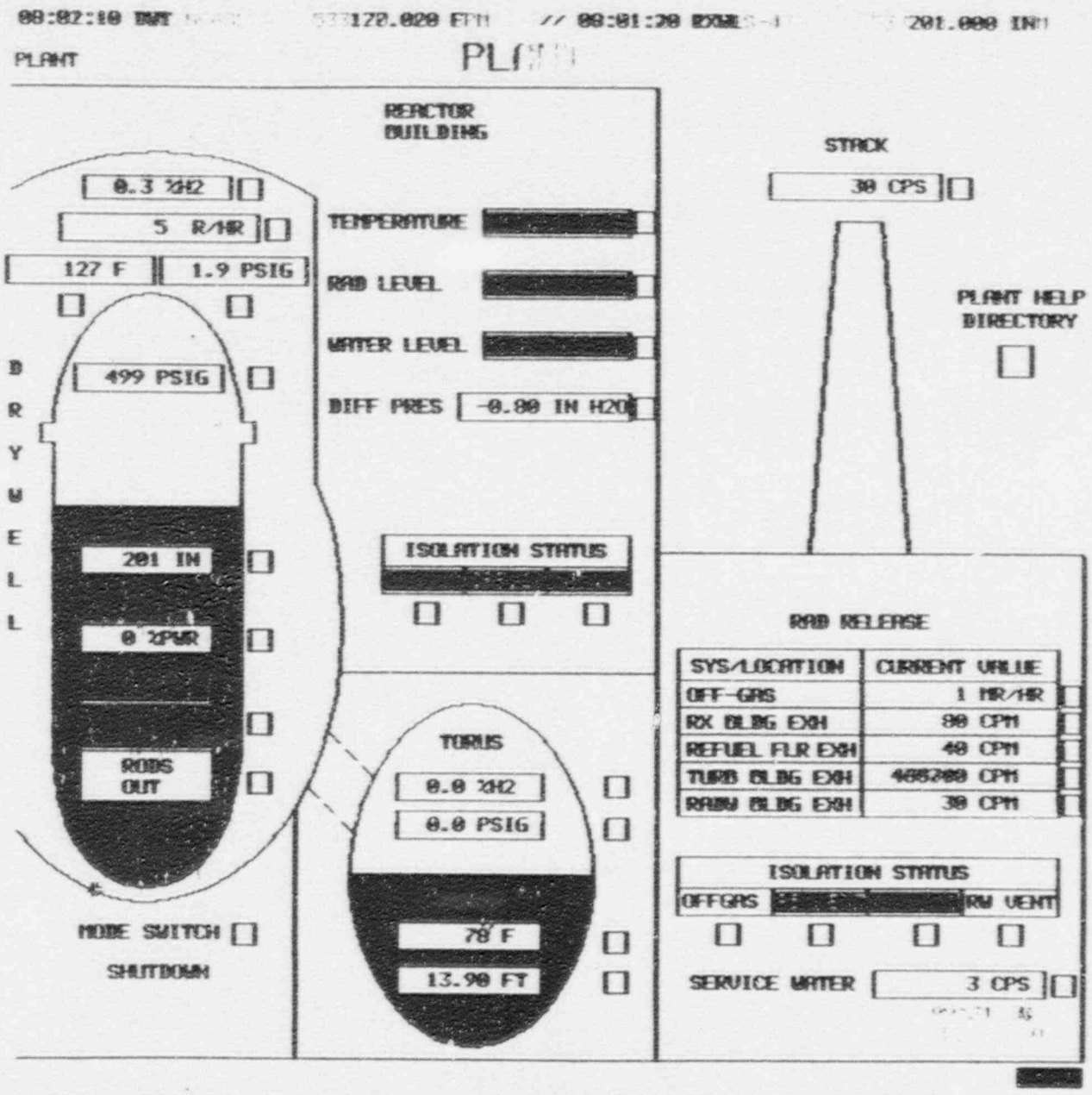
Issued To: Control Room

Issued By: Lead Controller

MESSAGE

An explosion is heard coming from the Turbine Building.

Plant parameters as indicated on SPDS display.


PLANT
REACTOR BUILDING
STACK
0.3 %H2
5 R/HR
127 F
1.9 PSIG
TEMPERATURE
RAD LEVEL
WATER LEVEL
DIFF PRES
-0.00 IN H2O
30 CPS
PLANT HELP DIRECTORY
DRYWELL
499 PSIG
201 IN
0 %PWR
RODS OUT
ISOLATION STATUS
RAD RELEASE
SYS/LOCATION
CURRENT VALUE
OFF-GAS
1 MR/HR
RX BLDG EXH
80 CPM
REFUEL FLR EXH
40 CPM
TURB BLDG EXH
465280 CPM
RADW BLDG EXH
30 CPM
TORUS
0.0 %H2
0.0 PSIG
78 F
13.90 FT
ISOLATION STATUS
OFFGAS
RW VENT
MODE SWITCH
SHUTDOWN
SERVICE WATER
3 GPS

09:31:58 1801-021-22 [redacted] MR/HR // 09:31:58 1801-021-07 [redacted] MR/HR

Form EAP-1.1.3
James A. Fitzpatrick Nuclear Power Plant
PART III - Plant Parameters

Sequence No. _____

APRM REACTOR POWER	0.0000E+00	%
IRM REACTOR POWER	0.0000E+00	%
SRM REACTOR POWER	0.0000E+00	CPS
RX WATER LEVEL	201.0	Inch TAF
FEEDWATER FLOW	0.3906	MLB/HR
RCIC FLOW	390.0	GPM
REACTOR PRESSURE	493.6	PSIG
HPCI FLOW	0.0000E+00	GPM
LPCI A FLOW	0.0000E+00	GPM
LPCI B FLOW	0.0000E+00	GPM
"A" CORE SPRAY FLOW	0.0000E+00	GPM
"B" CORE SPRAY FLOW	0.0000E+00	GPM
DRYWELL SUMP LEVEL	15.00	Inch
STACK GAS RAD.	75.00	UCI/sec
RX BLDG VENT RAD.	19.00	UCI/sec
REFUEL FLR EXH RAD.	2.000	UCI/sec
TB BLDG VENT RAD.	[redacted]	UCI/sec
RW BLDG VENT RAD.	2.000	UCI/sec
SERVICE WATER RAD.	0.1500E-05	UCI/ml
STACK HI RANGE RAD.	0.2819	CI/sec
TB BLD HI RANGE RAD.	[redacted]	CI/sec
RW BLD HI RANGE RAD.	0.0000E+00	CI/sec
OFFGAS RAD.	1.000	mR/Hr
DRYWELL RAD.	5.360	R/Hr
HIGHEST MSL RAD MON	4329.	mR/Hr
DRYWELL PRESSURE	1.900	PSIG
DRYWELL TEMPERATURE	126.9	Deg F
TORUS WATER AVG TMP	78.26	Deg F
TORUS WATER LEVEL	13.90	FT
DRYWELL H2 CONC	0.3000	%
DRYWELL O2 CONC	1.000	%
CST LEVEL	259.0	Inch

Emergency Director Approval: _____ TIME: _____

[redacted]

MESSAGE SHEET

MESSAGE NO. 14

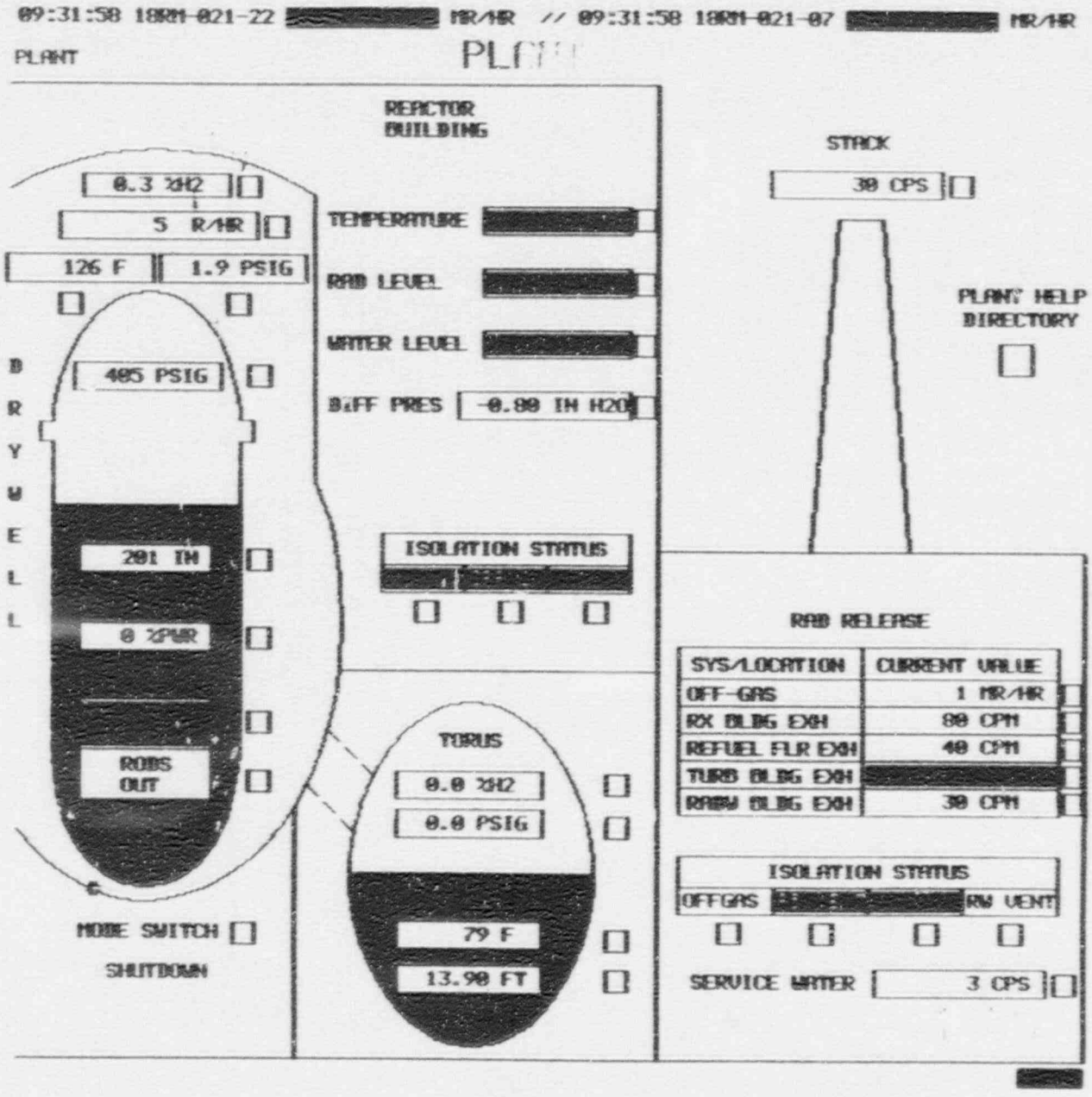
Exercise Time: 0945

Issued To: Control Room

Issued By: Lead Controller

MESSAGE

Plant parameters as indicated on SPDS display.


09:31:58 1881-021-22 MR/HR // 09:31:58 1881-021-07 MR/HR
PLANT
PLANT
REACTOR BUILDING
STACK
0.3 %H2
5 R/HR
126 F
1.9 PSIG
TEMPERATURE
RAD LEVEL
WATER LEVEL
DIFF PRES
-0.00 IN H20
30 CPS
PLANT HELP DIRECTORY
DRYWELL
485 PSIG
201 IN
0 %PWR
RODS OUT
ISOLATION STATUS
TORUS
0.0 %H2
0.0 PSIG
79 F
13.90 FT
MODE SWITCH
SHUTDOWN
RAD RELEASE
SYS/LOCATION
CURRENT VALUE
OFF-GAS
1 MR/HR
RX BLDG EXH
80 CPM
REFUEL FLR EXH
40 CPM
TURB BLDG EXH
RADW BLDG EXH
30 CPM
ISOLATION STATUS
OFFGAS
RW VENT
SERVICE WATER
3 GPS

09:31:58 1RM1-021-22 [redacted] MR/HR // 09:31:58 1RM1-021-07 [redacted] MR/HR

Form EPP-1.1.3
James A. Fitzpatrick Nuclear Power Plant
PART III - Plant Parameters

Sequence No. _____

APRM REACTOR POWER	0.0000E+00	%
IRM REACTOR POWER	0.0000E+00	%
SRM REACTOR POWER	0.0000E+00	CPS
RX WATER LEVEL	201.0	Inch Taf
FEEDWATER FLOW	0.3000	MLB/HR
RCIC FLOW	300.0	GPM
REACTOR PRESSURE	409.1	PSIG
HPCI FLOW	0.0000E+00	GPM
LPCI A FLOW	0.0000E+00	GPM
LPCI B FLOW	0.0000E+00	GPM
"A" CORE SPRAY FLOW	0.0000E+00	GPM
"B" CORE SPRAY FLOW	0.0000E+00	GPM
DRYWELL SUMP LEVEL	15.00	Inch
STACK GAS RAD.	75.00	UCI/sec
RX BLDG VENT RAD.	19.00	UCI/sec
REFUEL FLR EXH RAD.	2.000	UCI/sec
TB BLDG VENT RAD.	[redacted]	UCI/sec
RW BLDG VENT RAD.	2.000	UCI/sec
SERVICE WATER RAD.	0.1500E-05	UCI/ml
STACK HI RANGE RAD.	0.2819	CI/sec
TB BLD HI RANGE RAD.	[redacted]	CI/sec
RW BLD HI RANGE RAD.	0.0000E+00	CI/sec
OFFGAS RAD.	1.000	mR/Hr
DRYWELL RAD.	4.914	R/Hr
HIGHEST MSL RAD MON	4886.	mR/Hr
DRYWELL PRESSURE	1.900	PSIG
DRYWELL TEMPERATURE	125.7	Deg F
TORUS WATER AVG TMP	78.64	Deg F
TORUS WATER LEVEL	13.90	FT
DRYWELL H2 CONC	0.3000	%
DRYWELL O2 CONC	1.800	%
CST LEVEL	250.0	Inch

Emergency Director Approval: _____ TIME: _____

[redacted]

MESSAGE SHEET

MESSAGE NO. 15

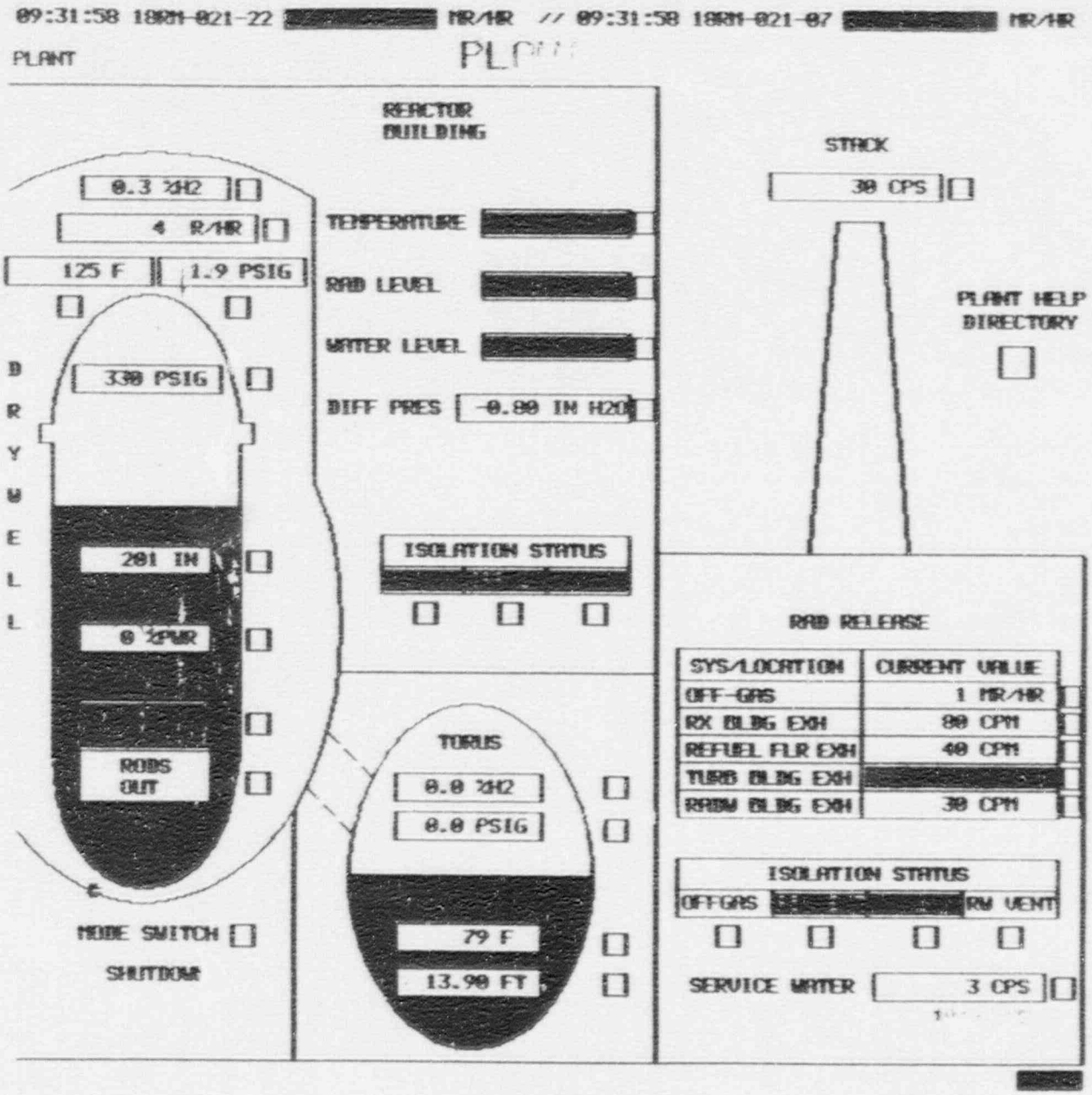
Exercise Time: 1000

Issued To: Control Room

Issued By: Lead Controller

MESSAGE

Plant parameters as indicated on SPDS display.


09:31:58 1801-021-22 MR/HR // 09:31:58 1801-021-07 MR/HR
PLANT
REACTOR BUILDING
0.3 %H2
4 R/HR
125 F
1.9 PSIG
TEMPERATURE
RAD LEVEL
WATER LEVEL
DIFF PRES
-0.80 IN H20
330 PSIG
DRYWELL
281 IN
0 %PWR
RODS OUT
ISOLATION STATUS
TORUS
0.0 %H2
0.0 PSIG
79 F
13.90 FT
MODE SWITCH
SHUTDOWN
STACK
30 CPS
PLANT HELP DIRECTORY
RAD RELEASE
SYS/LOCATION
CURRENT VALUE
OFF-GAS
1 MR/HR
RX BLDG EXH
80 CPM
REFUEL FLR EXH
40 CPM
TURB BLDG EXH
RADW BLDG EXH
30 CPM
ISOLATION STATUS
OFFGAS
RW VENT
SERVICE WATER
3 GPS

09:31:58 1881-021-22 [redacted] MR/HR // 09:31:58 1881-021-07 [redacted] MR/HR

Form EAP-1.1.3
James A. Fitzpatrick Nuclear Power Plant
PART III - Plant Parameters

Sequence No. _____

APRM REACTOR POWER	0.0000E+00	%
IRM REACTOR POWER	0.0000E+00	%
SRM REACTOR POWER	0.0000E+00	CPS
RX WATER LEVEL	201.0	Inch TAF
FEEDWATER FLOW	0.3000	MLB/HR
RCIC FLOW	254.6	GPM
REACTOR PRESSURE	324.1	PSIG
HPCI FLOW	0.0000E+00	GPM
LPCI A FLOW	123.4	GPM
LPCI B FLOW	0.0000E+00	GPM
"A" CORE SPRAY FLOW	0.0000E+00	GPM
"B" CORE SPRAY FLOW	0.0000E+00	GPM
DRYWELL SUMP LEVEL	15.00	Inch
STACK GAS RAD.	75.00	UCI/sec
RX BLDG VENT RAD.	19.00	UCI/sec
REFUEL FLR EXH RAD.	2.000	UCI/sec
TB BLDG VENT RAD.	[redacted]	UCI/sec
RW BLDG VENT RAD.	2.000	UCI/sec
SERVICE WATER RAD.	0.1500E-05	UCI/ml
STACK HI RANGE RAD.	0.2819	CI/sec
TB BLD HI RANGE RAD.	[redacted]	CI/sec
RW BLD HI RANGE RAD.	0.0000E+00	CI/sec
OFFGAS RAD.	1.000	mR/Hr
DRYWELL RAD.	4.468	R/Hr
HIGHEST MSL RAD MON	5286.	mR/Hr
DRYWELL PRESSURE	1.900	PSIG
DRYWELL TEMPERATURE	124.5	Deg F
TORUS WATER AVG TMP	79.03	Deg F
TORUS WATER LEVEL	13.90	FT
DRYWELL H2 CONC	0.3900	%
DRYWELL O2 CONC	1.800	%
CST LEVEL	250.0	Inch

Emergency Director Approval: _____ TIME: _____

[redacted]

MESSAGE SHEET

MESSAGE NO. 16

Exercise Time: 1015

Issued To: Control Room

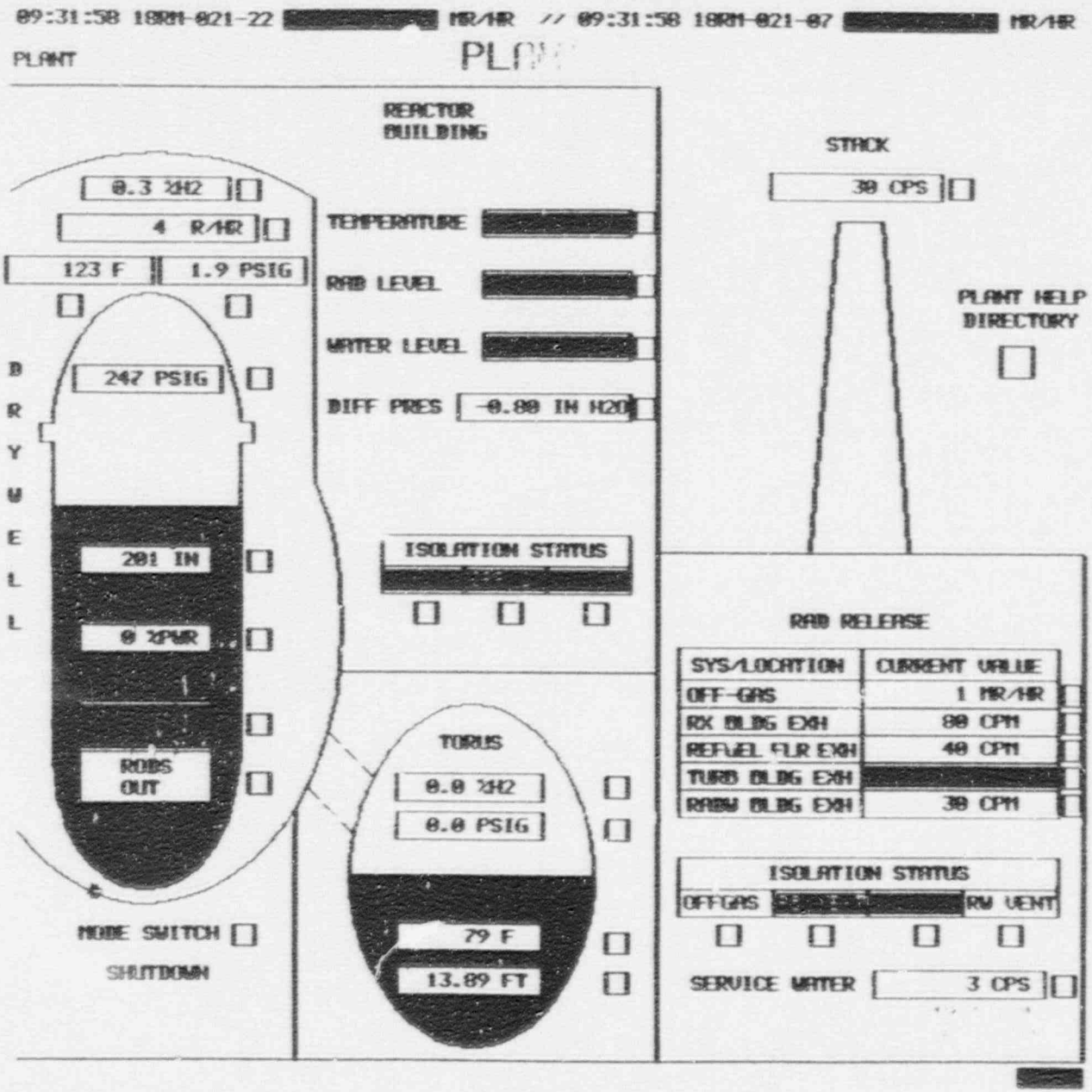
Issued By: Lead Controller

MESSAGE

The following annunciators alarm:

09-8-1-20 125VDC Batt Chgr A Current or Output Lo

Plant parameters as indicated on SPDS display.


09:31:58 1RM1-021-22 [redacted] MR/HR // 09:31:58 1RM1-021-07 [redacted] MR/HR
PLANT
PLANT
REACTOR BUILDING
STACK
0.3 %H2
4 R/HR
123 F
1.9 PSIG
TEMPERATURE
RAD LEVEL
WATER LEVEL
DIFF PRES
-0.00 IN H20
30 CPS
PLANT HELP DIRECTORY
DRYWELL
247 PSIG
201 IN
0 %PWR
RODS OUT
ISOLATION STATUS
RAD RELEASE
SYS/LOCATION
CURRENT VALUE
OFF-GAS
1 MR/HR
RX BLDG EXH
80 CPM
REFUEL FLR EXH
40 CPM
TURB BLDG EXH
RADW BLDG EXH
30 CPM
TORUS
0.0 %H2
0.0 PSIG
79 F
13.89 FT
ISOLATION STATUS
OFFGAS
RW VENT
MODE SWITCH
SHUTDOWN
SERVICE WATER
3 CPS

09:31:58 1881-821-22 [redacted] MR/HR // 09:31:58 1881-821-07 [redacted] MR/HR

Form EAP-1.1.3
James A. Fitzpatrick Nuclear Power Plant
PART III - Plant Parameters

Sequence No. _____

APRM REACTOR POWER	0.0000E+00	%
IRM REACTOR POWER	0.0000E+00	%
SRM REACTOR POWER	0.0000E+00	CPS
RX WATER LEVEL	201.0	Inch TAF
FEEDWATER FLOW	0.3000	MLB/HR
RCIC FLOW	0.0000E+00	GPM
REACTOR PRESSURE	241.7	PSIG
HPCI FLOW	0.0000E+00	GPM
LPCI A FLOW	342.5	GPM
LPCI B FLOW	0.0000E+00	GPM
"A" CORE SPRAY FLOW	0.0000E+00	GPM
"B" CORE SPRAY FLOW	0.0000E+00	GPM
DRYWELL SUMP LEVEL	15.00	Inch
STACK GAS RAD.	75.00	UCI/sec
RX BLDG VENT RAD.	19.00	UCI/sec
REFUEL FLR EXH RAD.	2.000	UCI/sec
TB BLDG VENT RAD.	[redacted]	UCI/sec
RW BLDG VENT RAD.	2.000	UCI/sec
SERVICE WATER RAD.	0.1500E-05	UCI/ml
STACK HI RANGE RAD.	0.2819	CI/sec
TB BLD HI RANGE RAD.	[redacted]	CI/sec
RW BLD HI RANGE RAD.	0.0000E+00	CI/sec
OFFGAS RAD.	1.000	mR/Hr
DRYWELL RAD.	4.831	R/Hr
HIGHEST MSL RAD MON	5752.	mR/Hr
DRYWELL PRESSURE	1.900	PSIG
DRYWELL TEMPERATURE	123.3	Deg F
TORUS WATER AVG TMP	79.40	Deg F
TORUS WATER LEVEL	13.89	FT
DRYWELL H2 CONC	0.3000	%
DRYWELL O2 CONC	1.800	%
CST LEVEL	250.0	Inch

Energency Director Approval: _____ TIME: _____

[redacted]

FORM SAP 1.4 Page 22 of 28

MESSAGE SHEET

MESSAGE NO. 17

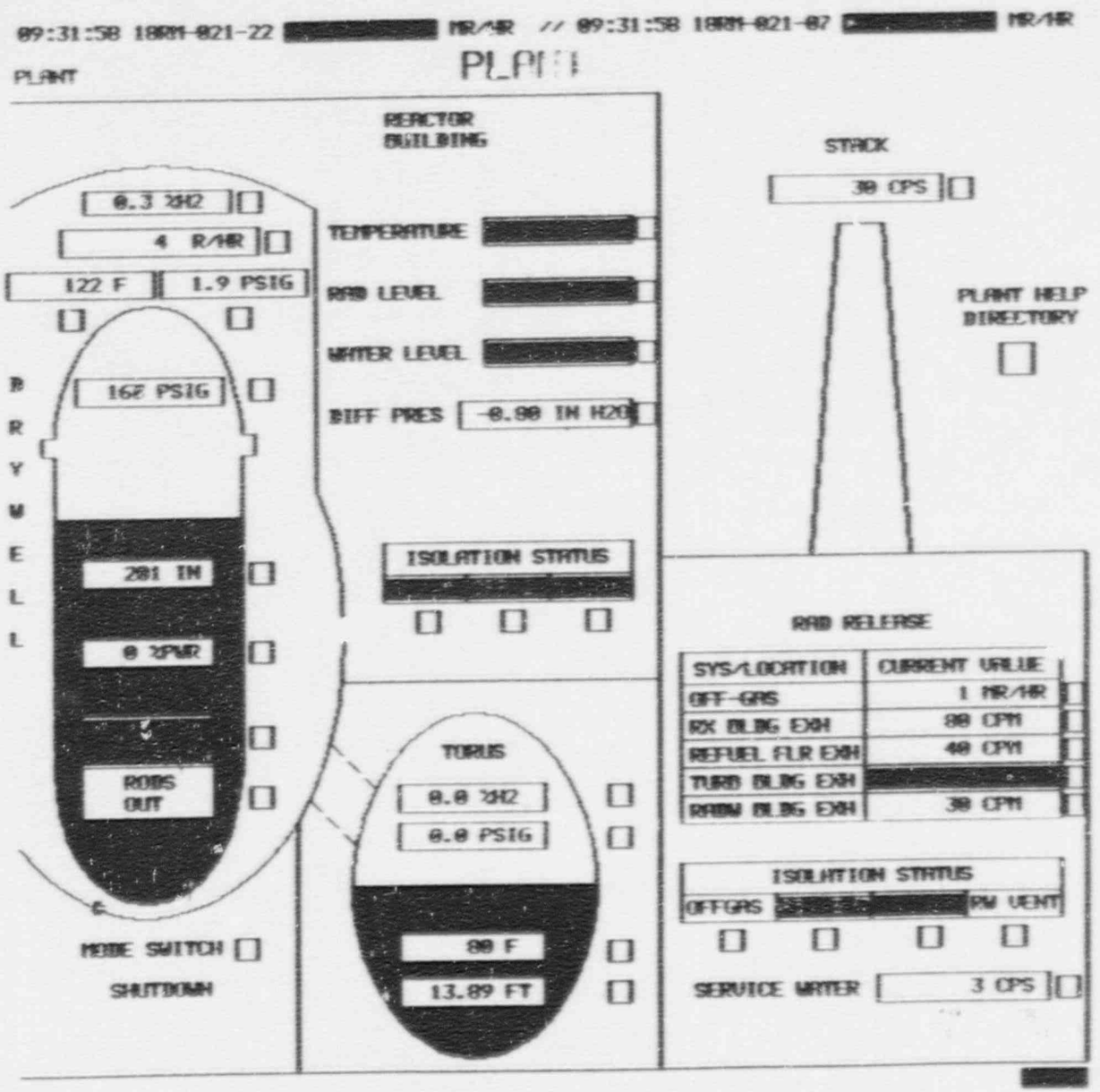
Exercise Time: 1030

Issued To: Control Room

Issued By: Lead Controller

MESSAGE

Plant parameters as indicated on SPDS display.


09:31:58 1881-821-22 [redacted] MR/HR // 09:31:58 1881-821-07 [redacted] MR/HR
PLANT
PLANT
REACTOR BUILDING
STACK
0.3 %H2
4 R/HR
122 F
1.9 PSIG
TEMPERATURE
RAD LEVEL
WATER LEVEL
DIFF PRES
-0.00 IN H20
30 CPS
PLANT HELP DIRECTORY
PRYWELL
162 PSIG
281 IN
0 %PWR
RODS OUT
ISOLATION STATUS
TORUS
0.0 %H2
0.0 PSIG
89 F
13.89 FT
MODE SWITCH
SHUTDOWN
RAD RELEASE
SYS/LOCATION
CURRENT VALUE
OFF-GAS
1 MR/HR
RX BLDG EXH
80 CPM
REFUEL FLR EXH
40 CPM
TURB BLDG EXH
RADW BLDG EXH
30 CPM
ISOLATION STATUS
OFFGAS
RW VENT
SERVICE WATER
3 CPS

09:31:58 1RRM-021-22 [redacted] MR/HR // 09:31:58 1RRM-021-07 [redacted] MR/HR

Form EIP-1.1.3
James A. Fitzpatrick Nuclear Power Plant
PART III - Plant Parameters

Sequence No. _____

APRM REACTOR POWER	0.0000E+00	%
IRM REACTOR POWER	0.0000E+00	%
SRM REACTOR POWER	0.0000E+00	CPS
RX WATER LEVEL	201.9	Inch TAF
FEEDWATER FLOW	0.3000	MLB/HR
RCIC FLOW	0.0000E+00	GPM
REACTOR PRESSURE	161.4	PSIG
HPCI FLOW	0.0000E+00	GPM
LPCI A FLOW	295.4	GPM
LPCI B FLOW	0.0000E+00	GPM
"A" CORE SPRAY FLOW	0.0000E+00	GPM
"B" CORE SPRAY FLOW	0.0000E+00	GPM
DRYWELL SUMP LEVEL	15.00	Inch
STACK GAS RAD.	75.00	UCI/sec
RX BLDG VENT RAD.	19.00	UCI/sec
REFUEL FLR EXH RAD.	2.000	UCI/sec
TB BLDG VENT RAD.	[redacted]	UCI/sec
RW BLDG VENT RAD.	2.000	UCI/sec
SERVICE WATER RAD.	0.1500E-05	UCI/ml
STACK HI RANGE RAD.	0.2819	CI/sec
TB BLD HI RANGE RAD.	[redacted]	CI/sec
RW BLD HI RANGE RAD.	0.0000E+00	CI/sec
OFFGAS RAD.	1.000	mR/Hr
DRYWELL RAD.	3.600	R/Hr
HIGHEST MSL RAD MON	6285.	mR/Hr
DRYWELL PRESSURE	1.900	PSIG
DRYWELL TEMPERATURE	122.1	Deg F
TORUS WATER AVG TMP	79.76	Deg F
TORUS WATER LEVEL	13.89	FT
DRYWELL H2 CONC	0.3000	%
DRYWELL O2 CONC	1.800	%
CST LEVEL	250.0	Inch

Emergency Director Approval: _____ TIME: _____

[redacted]

MESSAGE SHEET

MESSAGE NO. 18

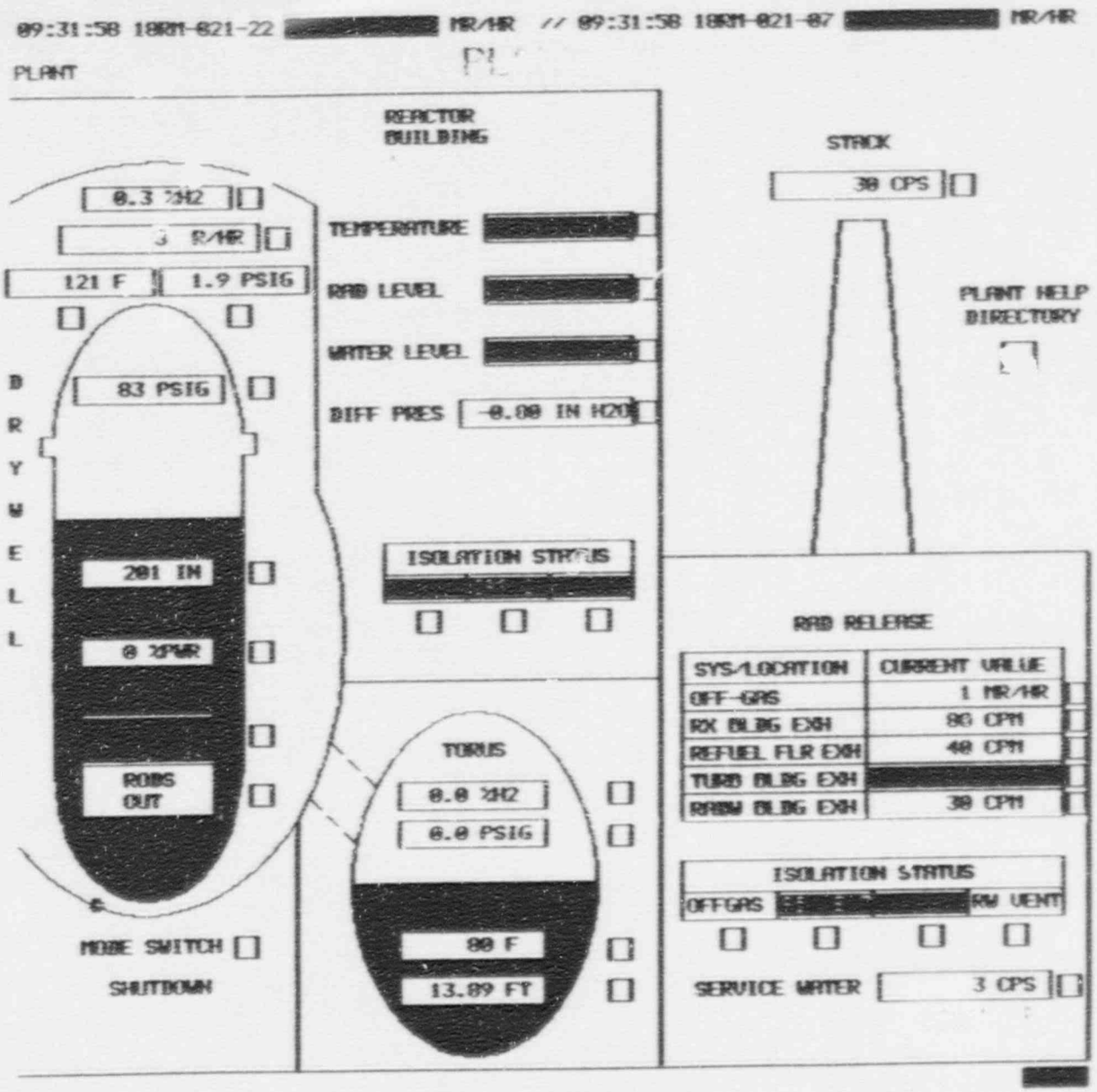
Exercise Time: 1045

Issued To: Control Room

Issued By: Lead Controller

MESSAGE

Plant parameters as indicated on SPDS display.


09:31:58 1881-821-22 MR/HR // 09:31:58 1881-821-87 MR/HR
PLANT
REACTOR BUILDING
STACK
30 CPS
0.3 %H2
3 R/HR
TEMPERATURE
121 F
1.9 PSIG
RAD LEVEL
WATER LEVEL
PLANT HELP DIRECTORY
DRYWELL
83 PSIG
DIFF PRES
-0.00 IN H20
ISOLATION STATUS
201 IN
0 %PWR
RODS OUT
TORUS
0.0 %H2
0.0 PSIG
RAD RELEASE
SYS/LOCATION
CURRENT VALUE
OFF-GAS
1 MR/HR
RX BLDG EXH
80 CPM
REFUEL FLR EXH
40 CPM
TURB BLDG EXH
RADW BLDG EXH
30 CPM
ISOLATION STATUS
OFFGAS
RW VENT
MODE SWITCH
SHUTDOWN
80 F
13.89 FT
SERVICE WATER
3 GPS

09:31:58 18RM-821-22 [redacted] MR/HR // 09:31:58 18RM-821-07 [redacted] MR/HR

Form EPP-1.1.3
James A. Fitzpatrick Nuclear Power Plant
PART III - Plant Parameters

Sequence No. _____

APRM REACTOR POWER	0.0000E+00	%
IRM REACTOR POWER	0.0000E+00	%
SRM REACTOR POWER	0.0000E+00	CPS
RX WATER LEVEL	201.0	Inch TAF
FEEDWATER FLOW	0.3000	MLB/HR
RCIC FLOW	0.0000E+00	GPM
REACTOR PRESSURE	77.20	PSIG
HPCI FLOW	0.0000E+00	GPM
LPCI A FLOW	246.4	GPM
LPCI B FLOW	0.0000E+00	GPM
"A" CORE SPRAY FLOW	0.0000E+00	GPM
"B" CORE SPRAY FLOW	0.0000E+00	GPM
DRYWELL SUMP LEVEL	15.00	Inch
STACK GAS RAD.	75.00	UCI/sec
RX BLDG VENT RAD.	19.00	UCI/sec
REFUEL FLR EXH RAD.	2.000	UCI/sec
TB BLDG VENT RAD.	[redacted]	UCI/sec
RW BLDG VENT RAD.	2.000	UCI/sec
SERVICE WATER RAD.	0.1500E-05	UCI/ml
STACK HI RANGE RAD.	0.2819	CI/sec
TB BLD HI RANGE RAD.	[redacted]	CI/sec
RW BLD HI RANGE RAD.	0.0000E+00	CI/sec
OFFGAS RAD.	1.000	mR/Hr
DRYWELL RAD.	3.162	R/Hr
HIGHEST MSL RAD MON	6678.	mR/Hr
DRYWELL PRESSURE	1.900	PSIG
DRYWELL TEMPERATURE	120.9	Deg F
TORUS WATER AVG TMP	80.14	Deg F
TORUS WATER LEVEL	13.89	FT
DRYWELL H2 CONC	0.3000	%
DRYWELL O2 CONC	1.800	%
CST LEVEL	250.0	Inch

Emergency Director Approval: _____ TIME: _____

[redacted]

MESSAGE SHEET

MESSAGE NO. 19

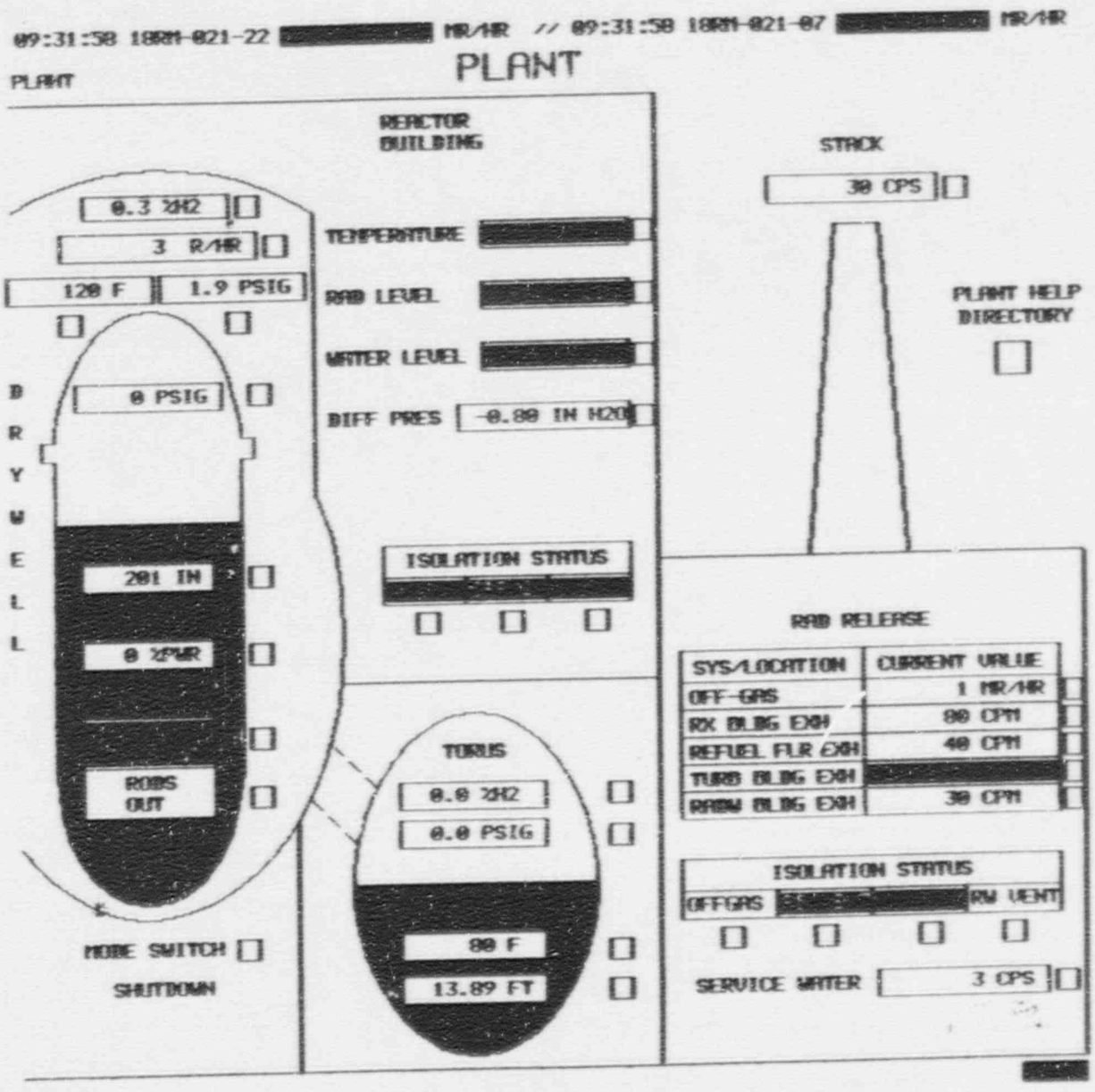
Exercise Time: 1100

Issued To: Control Room

Issued By: Lead Controller

MESSAGE

Plant parameters as indicated on SPDS display.


09:31:58 1881-821-22 MR/HR // 09:31:58 1881-821-07 MR/HR
PLANT
PLANT
REACTOR BUILDING
0.3 %H2
3 R/HR
120 F
1.9 PSIG
TEMPERATURE
RAD LEVEL
WATER LEVEL
DIFF PRES
-0.80 IN H2O
0 PSIG
DRYWELL
201 IN
0 %PWR
RODS OUT
ISOLATION STATUS
TORUS
0.0 %H2
0.0 PSIG
80 F
13.89 FT
MODE SWITCH
SHUTDOWN
STACK
30 CPS
PLANT HELP DIRECTORY
RAD RELEASE
SYS/LOCATION
CURRENT VALUE
OFF-GAS
1 MR/HR
RX BLDG EXH
80 CPM
REFUEL FLR EXH
40 CPM
TURB BLDG EXH
RADW BLDG EXH
30 CPM
ISOLATION STATUS
OFFGAS
RW VENT
SERVICE WATER
3 GPS

09:31:58 1881-821-22 [redacted] MR/HR // 09:31:58 1881-821-87 [redacted] MR/HR

Form EMP-1.1.3
James A. Fitzpatrick Nuclear Power Plant
PART III - Plant Parameters

Sequence No. _____

APRM REACTOR POWER	0.0000E+00	%
IRM REACTOR POWER	0.0000E+00	%
SRM REACTOR POWER	0.0000E+00	CPS
RX WATER LEVEL	201.0	Inch TAF
FEEDWATER FLOW	0.3000	MLB/HR
RCIC FLOW	0.0000E+00	GPM
REACTOR PRESSURE	0.0000E+00	PSIG
HPCI FLOW	0.0000E+00	GPM
LPCI A FLOW	197.3	GPM
LPCI B FLOW	0.0000E+00	GPM
"A" CORE SPRAY FLOW	0.0000E+00	GPM
"B" CORE SPRAY FLOW	0.0000E+00	GPM
DRYWELL SUMP LEVEL	15.00	Inch
STACK GAS RAD.	75.00	UCI/sec
RX BLDG VENT RAD.	19.00	UCI/sec
REFUEL FLR EXH RAD.	2.000	UCI/sec
TB BLDG VENT RAD.	[redacted]	UCI/sec
RW BLDG VENT RAD.	2.000	UCI/sec
SERVICE WATER RAD.	0.1500E-05	UCI/ml
STACK HI RANGE RAD.	0.2819	CI/sec
TB BLD HI RANGE RAD.	[redacted]	CI/sec
RW BLD HI RANGE RAD.	0.0000E+00	CI/sec
OFFGAS RAD.	1.000	mR/Hr
DRYWELL RAD.	2.727	R/Hr
HIGHEST MSL RAD MON	7150.	mR/Hr
DRYWELL PRESSURE	1.900	PSIG
DRYWELL TEMPERATURE	119.7	Deg F
TORUS WATER AVG TMP	80.52	Deg F
TORUS WATER LEVEL	13.89	FT
DRYWELL H2 CONC	0.3000	%
DRYWELL O2 CONC	1.000	%
CST LEVEL	250.0	Inch

Emergency Director Approval: _____ TIME: _____

[redacted]

MESSAGE SHEET

MESSAGE NO. 20

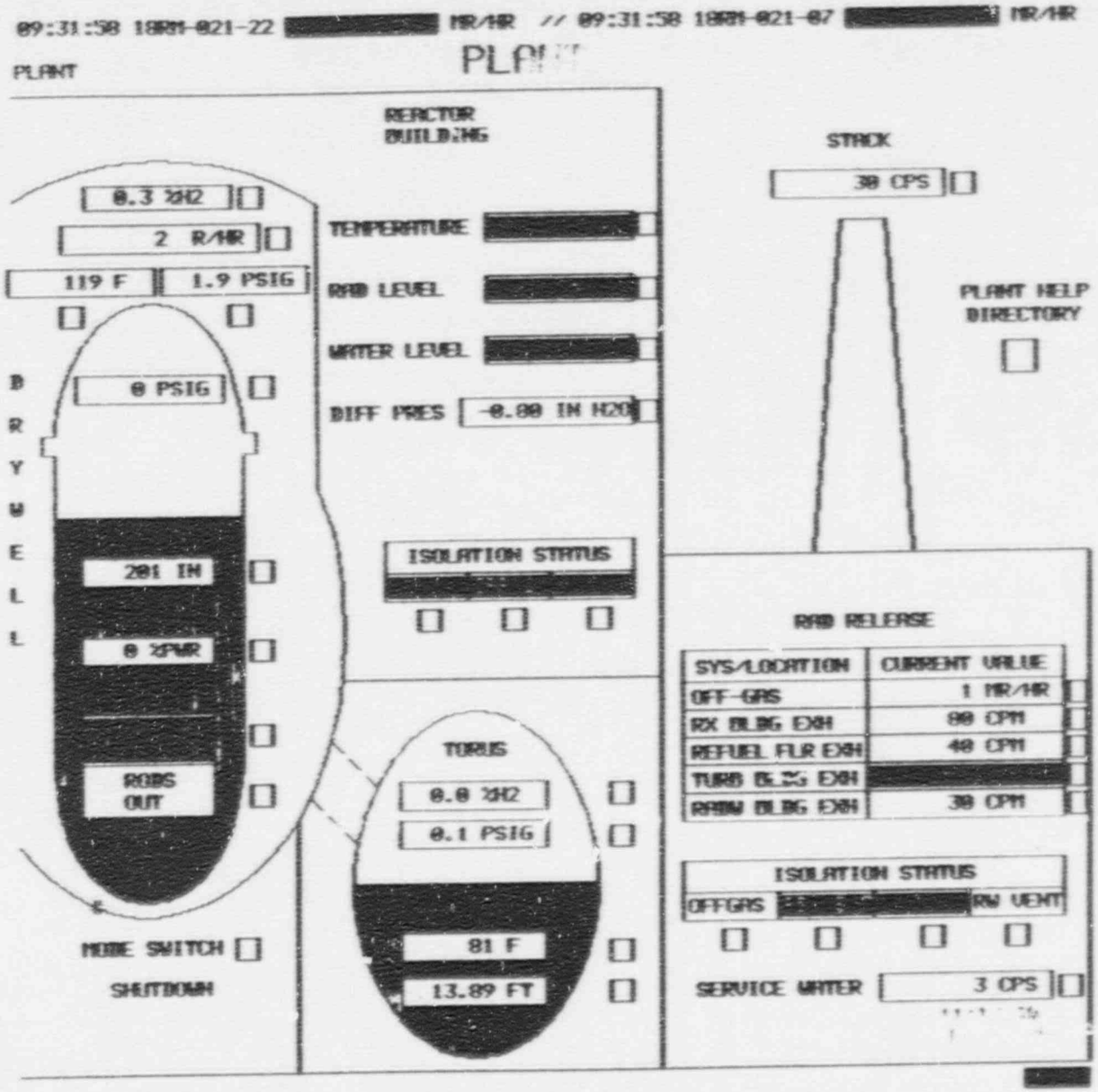
Exercise Time: 1115

Issued To: Control Room

Issued By: Lead Controller

MESSAGE

Plant parameters as indicated on SPDS display.


09:31:58 1881-021-22 MR/HR // 09:31:58 1881-021-87 MR/HR
PLANT
REACTOR BUILDING
STACK
0.3 %H2
2 R/HR
TEMPERATURE
119 F
1.9 PSIG
RHR LEVEL
WATER LEVEL
DRYWELL
0 PSIG
DIFF PRES
-0.00 IN H2O
PLANT HELP DIRECTORY
ISOLATION STATUS
201 IN
0 %PWR
RHR RELEASE
SYS/LOCATION
CURRENT VALUE
OFF-GAS
1 MR/HR
RX BLDG EXH
80 CPM
REFUEL FLR EXH
40 CPM
TURB BLDG EXH
RADW BLDG EXH
30 CPM
RODS OUT
TORUS
0.0 %H2
0.1 PSIG
ISOLATION STATUS
OFFGAS
RW VENT
MODE SWITCH
SHUTDOWN
81 F
13.89 FT
SERVICE WATER
3 CPS

09:31:58 10RM-021-22 [redacted] MR/HR // 09:31:58 10RM-021-07 [redacted] MR/HR

Form EAP-1.1.3
James A. Fitzpatrick Nuclear Power Plant
PART III - Plant Parameters

Sequence No. ___ _____

APRM REACTOR POWER	0.0000E+00	%
IRM REACTOR POWER	0.0000E+00	%
SRM REACTOR POWER	0.0000E+00	CPS
RX WATER LEVEL	201.0	Inch TAF
FEEDWATER FLOW	0.3900	MLB/HR
RCIC FLOW	0.0000E+00	GPM
REACTOR PRESSURE	0.0000E+00	PSIG
HPCI FLOW	0.0000E+00	GPM
LPCI A FLOW	147.2	GPM
LPCI B FLOW	0.0000E+00	GPM
"A" CORE SPRAY FLOW	0.0000E+00	GPM
"B" CORE SPRAY FLOW	0.0000E+00	GPM
DRYWELL SUMP LEVEL	15.00	Inch
STACK GAS RAD.	75.00	UCI/sec
RX BLDG VENT RAD.	19.00	UCI/sec
REFUEL FLR EXH RAD.	2.000	UCI/sec
TB BLDG VENT RAD.	[redacted]	UCI/sec
RW BLDG VENT RAD.	2.000	UCI/sec
SERVICE WATER RAD.	0.1500E-05	UCI/ml
STACK HI RANGE RAD.	0.2819	CI/sec
TB BLD HI RANGE RAD.	[redacted]	CI/sec
RW BLD HI RANGE RAD.	0.0000E+00	CI/sec
OFFGAS RAD.	1.000	mR/Hr
DRYWELL RAD.	2.276	R/Hr
HIGHEST MSL RAD MON	7632.	mR/Hr
DRYWELL PRESSURE	1.900	PSIG
DRYWELL TEMPERATURE	118.5	Deg F
TORUS WATER AVG TMP	89.91	Deg F
TORUS WATER LEVEL	13.89	FT
DRYWELL H2 CONC	0.3900	%
DRYWELL 02 CONC	1.000	%
CST LEVEL	250.0	Inch

Emergency Director Approval: _____ TIME: _____

[redacted]

MESSAGE SHEET

MESSAGE NO. 21

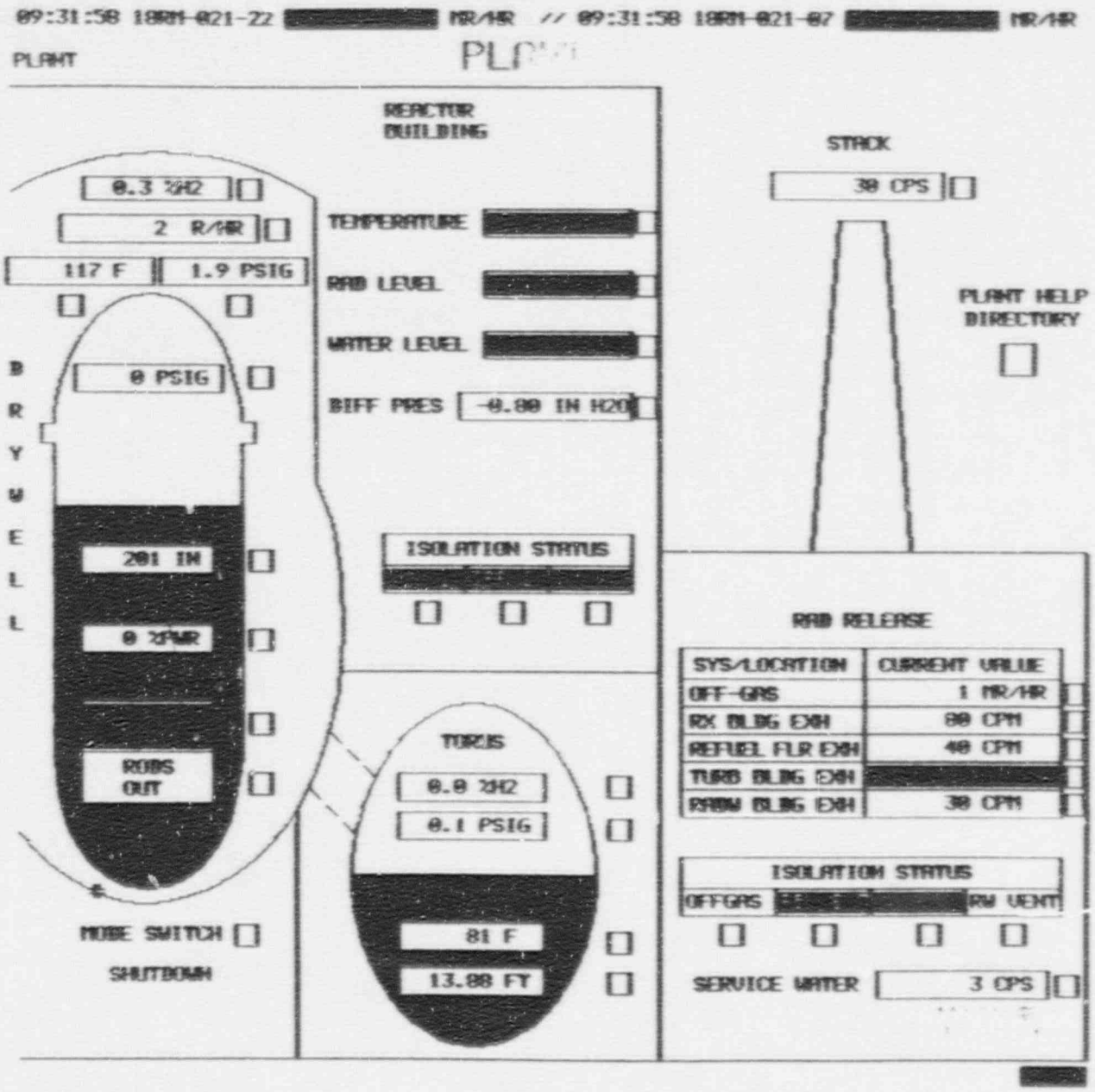
Exercise Time: 1130

Issued To: Control Room

Issued By: Lead Controller

MESSAGE

Plant parameters as indicated on SPDS display.


09:31:58 1881-021-22 MR/HR // 09:31:58 1881-021-07 MR/HR
PLANT
REACTOR BUILDING
0.3 %H2
2 R/HR
117 F
1.9 PSIG
TEMPERATURE
RAD LEVEL
WATER LEVEL
DIFF PRES
-0.00 IN H20
0 PSIG
201 IN
0 %PWR
RODS OUT
ISOLATION STATUS
TORUS
0.0 %H2
0.1 PSIG
81 F
13.00 FT
MODE SWITCH
SHUTDOWN
STACK
30 CPS
PLANT HELP DIRECTORY
RAD RELEASE
SYS/LOCATION
CURRENT VALUE
OFF-GAS
1 MR/HR
RX BLDG EXH
00 CPM
REFUEL FLR EXH
40 CPM
TURB BLDG EXH
RADW BLDG EXH
30 CPM
ISOLATION STATUS
OFFGAS
RW VENT
SERVICE WATER
3 GPS

09:31:58 18RM-021-22 [redacted] MR/HR // 09:31:58 18RM-021-07 [redacted] MR/HR

Form EAP-1.1.3
James A. Fitzpatrick Nuclear Power Plant
PART III - Plant Parameters

Sequence No. _____

APRM REACTOR POWER	0.0000E+00	%
IRM REACTOR POWER	0.0000E+00	%
SRM REACTOR POWER	0.0000E+00	CPS
RX WATER LEVEL	201.0	Inch TAF
FEEDWATER FLOW	0.3000	MLB/HR
RCIC FLOW	0.0000E+00	GPM
REACTOR PRESSURE	0.0000E+00	PSIG
HPCI FLOW	0.0000E+00	GPM
LPCI A FLOW	100.2	GPM
LPCI B FLOW	0.0000E+00	GPM
"A" CORE SPRAY FLOW	0.0000E+00	GPM
"B" CORE SPRAY FLOW	0.0000E+00	GPM
DRYWELL SUMP LEVEL	15.00	Inch
STACK GAS RAD.	75.00	UCI/sec
RX BLDG VENT RAD.	19.00	UCI/sec
REFUEL FLR EXH RAD.	2.000	UCI/sec
TB BLDG VENT RAD.	[redacted]	UCI/sec
RW BLDG VENT RAD.	2.000	UCI/sec
SERVICE WATER RAD.	0.1500E-05	UCI/ml
STACK HI RANGE RAD.	0.2819	CI/sec
TB BLD HI RANGE RAD.	[redacted]	CI/sec
RW BLD HI RANGE RAD.	0.0000E+00	CI/sec
OFFGAS RAD.	1.000	mR/Hr
DRYWELL RAD.	1.854	R/Hr
HIGHEST MSL RAD MON	0004.	mR/Hr
DRYWELL PRESSURE	1.900	PSIG
DRYWELL TEMPERATURE	117.3	Deg F
TORUS WATER AVG TMP	81.27	Deg F
TORUS WATER LEVEL	13.00	FT
DRYWELL H2 CONC	0.3000	%
DRYWELL O2 CONC	1.000	%
CST LEVEL	250.0	Inch

Emergency Director Approval: _____ TIME: _____

[redacted]

FORM SAP 1.4 Page 27 of 28

MESSAGE SHEET

MESSAGE NO. 22

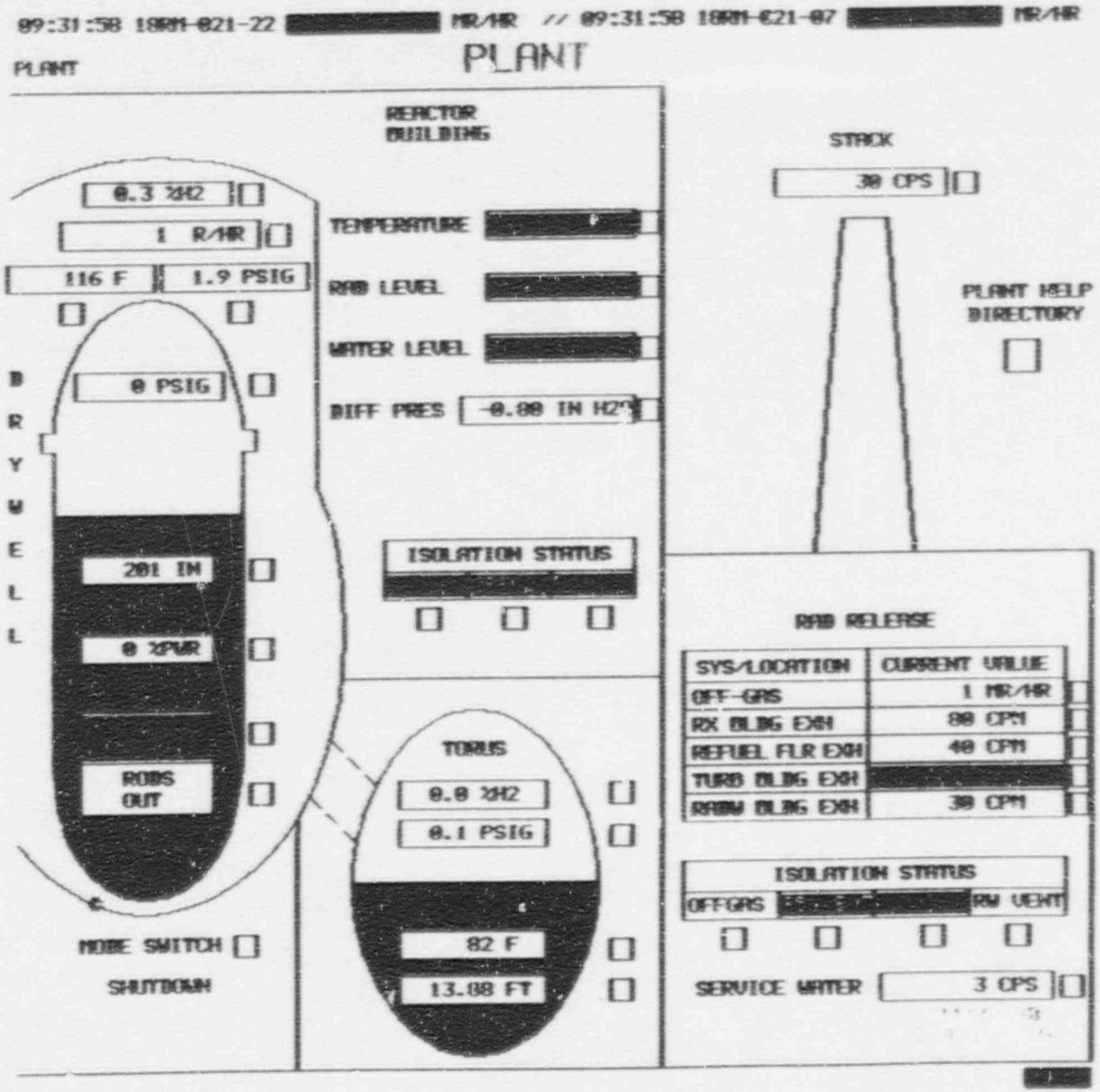
Exercise Time: 1145

Issued To: Control Room

Issued By: Lead Controller

MESSAGE

Plant parameters as indicated on SPDS display.


09:31:58 1881-021-22 MR/HR // 09:31:58 1881-021-07 MR/HR
PLANT
PLANT
REACTOR BUILDING
0.3 %H2
1 R/HR
116 F
1.9 PSIG
TEMPERATURE
RAD LEVEL
WATER LEVEL
DIFF PRES
-0.00 IN H2O
DRYWELL
0 PSIG
201 IN
0 %PWR
RODS OUT
ISOLATION STATUS
TORUS
0.0 %H2
0.1 PSIG
82 F
13.08 FT
MODE SWITCH
SHUTDOWN
STACK
30 CPS
PLANT HELP DIRECTORY
RAD RELEASE
SYS/LOCATION
CURRENT VALUE
OFF-GAS
1 MR/HR
RX BLDG EXH
80 CPM
REFUEL FLR EXH
40 CPM
TURB BLDG EXH
RADW BLDG EXH
30 CPM
ISOLATION STATUS
OFFGAS
RW VENT
SERVICE WATER
3 CPS

09:31:58 10RM-021-22 [redacted] MR/HR // 09:31:58 10RM-021-67 [redacted] MR/HR

Form EIP-1.1.3
James A. Fitzpatrick Nuclear Power Plant
PART III - Plant Parameters

Sequence No. _____

APRM REACTOR POWER	0.0000E+00	%
IRM REACTOR POWER	0.0000E+00	%
SRM REACTOR POWER	0.0000E+00	CPS
RX WATER LEVEL	201.0	Inch TAF
FEEDWATER FLOW	0.3900	MLB/HR
RCIC FLOW	0.0000E+00	GPM
REACTOR PRESSURE	0.0000E+00	PSIG
HPCI FLOW	0.0000E+00	GPM
LPCI A FLOW	52.88	GPM
LPCI B FLOW	0.0000E+00	GPM
"A" CORE SPRAY FLOW	0.0000E+00	GPM
"B" CORE SPRAY FLOW	0.0000E+00	GPM
DRYWELL SUMP LEVEL	15.00	Inch
STACK GAS RAD.	75.00	UCI/sec
RX BLDG VENT RAD.	19.00	UCI/sec
REFUEL FLR EXH RAD.	2.000	UCI/sec
TB BLDG VENT RAD.	[redacted]	UCI/sec
RW BLDG VENT RAD.	2.000	UCI/sec
SERVICE WATER RAD.	0.1500E-05	UCI/ml
STACK HI RANGE RAD.	0.2819	CI/sec
TB BLD HI RANGE RAD.	[redacted]	CI/sec
RW BLD HI RANGE RAD.	0.0000E+00	CI/sec
OFFGAS RAD.	1.000	mR/Hr
DRYWELL RAD.	1.439	R/Hr
HIGHEST MSL RAD MON	8539.	mR/Hr
DRYWELL PRESSURE	1.900	PSIG
DRYWELL TEMPERATURE	116.2	Deg F
TORUS WATER AVG TMP	81.63	Deg F
TORUS WATER LEVEL	13.88	FT
DRYWELL H2 CONC	0.3800	%
DRYWELL O2 CONC	1.800	%
CST LEVEL	250.0	Inch

Emergency Director Approval: _____ TIME: _____

[redacted]

MESSAGE SHEET

MESSAGE NO. 23

Exercise Time: 1200

Issued To: Control Room

Issued By: Lead Controller

MESSAGE

Plant is stable. Commence discussions on terminating the emergency and entering recovery.

SECTION 7

James A. FitzPatrick Nuclear Power Plant
1990 Partial Scale Emergency Exercise

December 19, 1990

DATA

I. Plant Process Computer Data

II. ARMs and In-Plant Maps

Chemistry Data

Offsite Radiological Data

SECTION 7

James A. FitzPatrick Nuclear Power Plant
1990 Partial Scale Emergency Exercise

December 19, 1990

Plant Process Computer (SPDS) Data

The following displays are examples of process computer logs and the SPDS displays that are available in the emergency facilities for personnel to obtain plant data information. These process computer logs are updated on a one minute basis and SPDS is a real-time display.

Process Computer Logs 4, 6 and 7 (ERDS/NYS Part III Data) and the SPDS Plant, RRC, SCR and TBRWRD displays are the only ones which will contain actual scenario data during the course of the training drill. This data is the same that can be found in Section 6 of this package on the Part III forms and SPDS displays.

FitzPatrick Nuclear Plant EOF/[illegible] Plant Status Logs

---- Master Menu ----

(1) Elog#1 - General Plant Status
(2) Elog#2 - Containment Temp/Pressure
(3) Elog#3 - Power & Steam Systems
(4) Elog#4 - Area Rad Monitors
(5) Elog#5 - Vessel/RCS Integrity
(6) Elog#6 - Ventilation Rad Monitors
(7) NYS Part-III Data Sheet
(8) One-hour Trends
(9) 15-Minute Met Data
(q) Quit and Exit System.

Enter 1,2,3,4,5,6,7,8 or q:

Fitzpatrick Plant Elog #1 Data at Time: 15:40

CORE THERMAL POWER MWTH-	-22.581
GROSS GEN POWER (MW)-	0.000
GROSS GEN POWER % RATED-	0.000
TOTAL CORE FLOW M#/HR-	27.913
TOTAL CORE FLOW % RATED-	36.361
RX TEMP/RECIRC A DELTA T-	-0.078
CNDSR 33C-10A VAC IN HG-	24.584
COND INLT/DISCH TUNNL DT-	4.557
REACTOR PRESSURE PSIG-	143.171
RX WATER LEVEL Inch TAF-	204.082
CLNUP SYS INLT TEMP DEGF-	333.599
RX WTR OUT NONREGEN HX T-	81.535
RB CLG WTR HX OUT DEGF-	73.678
DRYWELL PRESSURE PSIG-	0.155
SUPR POOL PRESSURE PSIG-	0.178
DRYWELL TO TORUS DELTA P-	-0.015
TORUS WATER LEVEL FEET-	13.887
SUPR POOL AIR TEMP DEG F-	84.926
DWLL OXYGEN(A) 0/30 %VOL-	21.997
STACK HI RANGE RAD mR/Hr-	0.176

Press <RETURN> for Main Menu [TAB,?]:

Fitzpatrick Plant Elog #2 Data at Time: 15:41

DRYWELL PRESSURE PSIG-	0.154	DWLL AREA 0(RTD-101)DEGF-	93.91
SUPR POOL PRESSURE PSIG-	0.176	DWLL AREA 0(RTD-120)DEGF-	89.86
DRYWELL TO TORUS DELTA P-	-0.016	DWLL AREA 1(RTD-119)DEGF-	89.06
DRYWELL TEMPERATURE DegF-	97.817	DWLL AREA 1(RTD-102)DEGF-	94.67
SUPR POOL AIR TEMP DEG F-	84.964	DWLL AREA 2(RTD-103)DEGF-	97.97
TORUS WATER LEVEL FT -	13.900	DWLL AREA 2(RTD-104)DEGF-	97.60
TORUS WATER AVG TMP DegF-	75.644	DWLL AREA 3(RTD-105)DEGF-	98.66
DRYWELL SUMP LEVEL Inch-	15.000	DWLL AREA 3(RTD-106)DEGF-	108.37
DRYWELL RAD Monitor R/Hr-	1.925	DWLL AREA 4(RTD-107)DEGF-	107.39
DRYWELL H2 CONC % -	0.354	DWLL AREA 4(RTD-108)DEGF-	112.47
DRYWELL O2 CONC % -	23.957	DWLL AREA 5(RTD-109)DEGF-	98.39
TORUS H2 CONC % -	0.000	DWLL AREA 5(RTD-117)DEGF-	109.59
TORUS O2 CONC % -	21.997	DWLL AREA 6(RTD-110)DEGF-	108.75
		DWLL AREA 6(RTD-111)DEGF-	108.37
		DWLL AREA 7(RTD-118)DEGF-	86.13
		DWLL AREA 8(RTD-113)DEGF-	84.11
		DWLL AREA 8(RTD-114)DEGF-	85.80
		DRYWELL WGTD AVG TEMP-	97.81

Press <RETURN> for Main Menu [TAB,?]:

Fitzpatrick Plant Elog #3 Data at Time: 15:41

APRM A FLUX LEVEL -	6.391	CRD DRIVE WATER FLO M#/H-	0.03
APRM B FLUX LEVEL -	5.038	FDWTR LOOP A FLOW M#/HR-	0.48
APRM C FLUX LEVEL -	5.618	FDWTR LOOP B FLOW M#/HR-	0.48
APRM D FLUX LEVEL -	4.127	CLNUP SYS A FLOW M#/HR-	0.04
APRM E FLUX LEVEL -	3.428	CLNUP SYS B FLOW M#/HR-	0.04
APRM F FLUX LEVEL -	5.817	CLNUP SYS INLT TEMP DEGF-	333.96
RX TEMP/RECIRC A DELTA T-	-0.341	CLNUP SYS OUTL TEMP DEGF-	293.65
TBCLCW CLR OUTLET DEG F-	74.852	RELIEF VALVE RV-2-71A T-	169.15
CNDSR 33C-10A VAC IN HG-	24.608	RELIEF VALVE RV-2-71B T-	150.04
TOTAL CORE FLOW M#/HR-	27.882	RELIEF VALVE RV-2-71C T-	140.47
CORE DIFFERENTIAL PRESS -	1.951	RELIEF VALVE RV-2-71D T-	121.62
RECIRC PMP MTR A PWR MW-	0.057	RELIEF VALVE RV-2-71E T-	181.01
RECIRC PMP MTR B PWR MW-	0.067	RELIEF VALVE RV-2-71F T-	133.04
TOTAL STEAM FLOW M#/HR-	0.534	RELIEF VALVE RV-2-71G T-	118.50
REACTOR PRESSURE PSIG-	142.804	RELIEF VALVE RV-2-71H T-	127.70
MAIN STEAM PRESSURE -	0.000	RELIEF VALVE RV-2-71J T-	128.12
MAIN STEAM PRESSURE -	1200.000	RELIEF VALVE RV-2-71K T-	137.93
COND INLT/DISCH TUNNL DT-	4.557	RELIEF VALVE RV-2-71L T-	202.30
COND PPS DISCH HDR PRESS-	240.741	RECIRC A1 INLT TEMP DEGF-	362.49
COND BSTR PMP SUCTN PSIG-	193.079	RECIRC B1 INLT TEMP DEGF-	363.82

Press <RETURN> for Main Menu [TAB,?]:

Fitzpatrick Plant Elog #4 Data at Time: 15:41

12	SPENT FUEL POOL mR/Hr-	0.546	01	CHEMISTRY LAB mR/Hr-	0.09
14	NEW FUEL VAULT mR/Hr-	1.043	02	CLOTHNG CHG AREAmR/Hr-	0.03
30	SOUTH REFUEL FLRmR/Hr-	188.848	03	CONTROL ROOM R/Hr-	0.01
13	RXBLD 344'SOUTH mR/Hr-	2.405	04	TB 300 HP-END mR/Hr-	0.57
15	RXCU PRECOAT TNKmR/Hr-	8.029	05	TB 300 LP-END mR/Hr-	0.39
17	FUEL POOL PUMP mR/Hr-	74.957	06	TB 252 HOGG PUMPmR/Hr-	0.10
18	CONTAM EQPT STORmR/Hr-	2.519	07	RX FEEDPUMP AREAmR/Hr-	0.49
16	RWCU HX ROOM mR/Hr-	1.955	08	RDWST CONTROL RMmR/Hr-	0.34
20	RX SAMPLE AREA mR/Hr-	7.559	09	RW284 FILTR/VLV mR/Hr-	6.60
19	RWCU PUMP AREA mR/Hr-	1.720	10	RW272 DRUM STOREmR/Hr-	1.78
21	RBCLC HX AREA mR/Hr-	0.408	11	RW252 RDWST PUMPmR/Hr-	12.29
23	RX BLDG EL 272' mR/Hr-	1.097	22	COND DEMIN VALVEmR/Hr-	0.66
24	TIP DRIVE PLATF mR/Hr-	2.637			
25	EAST CRD HCV mR/Hr-	0.897			
28	CRD REMOVL HATCHmR/Hr-	4.981			
26	WEST CRD HCV mR/Hr-	1.456			
27	EAST CRES EL227'mR/Hr-	12.778			
29	WEST CRES EL227'mR/Hr-	6.854			

Press <RETURN> for Main Menu [TAB,?]:

Fitzpatrick Plant Elog #5 Data at Time: 15:41

APRM REACTOR POWER % -	-0.923
RX WATER LEVEL Inch TAF-	204.132
REACTOR PRESSURE PSIG-	143.144
RX TEMP VIA STEAM TABLES-	362.152
FEEDWATER FLOW MLB/HR-	0.959
HPCI PUMP FLOW GPM-	1.406
LPCI A FLOW GPM-	-0.375
LPCI B FLOW GPM-	-3.124
LOWER CORESPRAY FLOW GPM-	0.000
UPPER CORESPRAY FLOW GPM-	0.000
DRYWELL SUMP LEVEL Inch-	15.000
SUPR POOL AIR TEMP DEG F-	84.964
SUPR POOL PRESSURE PSIG-	0.176
TORUS WATER LEVEL FT -	13.900
TORUS WATER AVG TMP DegF-	75.644
DRYWELL TEMPERATURE DegF-	97.817
DRYWELL PRESSURE PSIG-	0.154
PRIMARY CONT. H2 CONC % -	0.354
PRIMARY CONT. O2 CONC % -	23.957

Press <RETURN> for Main Menu [TAB,?]:

Fitzpatrick Plant Elog #6 Data at Time: 15:41

STACK HI RANGE RAD	mR/Hr-	0.182	MAIN STEAM RAD A	mR/Hr-	7.3
T-BLDG HI RNGE RAD	mR/Hr-	0.244	MAIN STEAM RAD B	mR/Hr-	8.1
RDWST HI RANGE RAD	mR/Hr-	0.100	MAIN STEAM RAD C	mR/Hr-	7.8
DWLL HI RNGE RAD(A)	R/HR-	1.523	MAIN STEAM RAD D	mR/Hr-	7.8
DWLL HI RNGE RAD(B)	R/HR-	1.925	RADWASTE EFFLUENT	CPS-	131.4
A XHST BELOW RFL FLR	CPM-	41.017	SERV WTR DISCHARGE	CPS-	3.5
B XHST BELOW RFL FLR	CPM-	173.776	INTAKE WATER TEMP	Deg F-	59.0
RFEUL FLR VENT A RAD	CPM-	68.122			
RFEUL FLR VENT B RAD	CPM-	39.570			
T-BLDG XHST A RAD	CPM-	32.347			
T-BLDG XHST B RAD	CPM-	34.198			
RDWST BLD XHST RAD A	CPM-	27.372			
RDWST BLD XHST RAD B	CPM-	31.221			
STACK MON A RAD	CPS-	5.601			
STACK MON B RAD	CPS-	33.564			
RX BLDG EXHAUST FLOW	CFM-	54153.406			
TB BLDG EXHAUST FLOW	CFM-	58993.875			
RW BLDG EXHAUST FLOW	CFM-	30132.855			
SBGT FLOW	CFM-	6198.150			

Press <RETURN> for Main Menu [TAB;?]:

Fitzpatrick Plant NYS Part III Data at Time: 15:41

APRM REACTOR POWER	% -	-0.923	TB BLDG VENT RAD.	uCi/ml-	3.420
IRM REACTOR POWER	% -	17.763	RW BLDG VENT RAD.	uCi/ml-	1.405
SRM REACTOR POWER	CPS-	823.658	STACK HI RANGE RAD.	Ci/s-	0.461
RX WATER LEVEL	Inch TAF-	204.132	TB BLD HIRANGE RAD.	Ci/s-	5.51
FEEDWATER FLOW	MLB/HR-	0.959	RW BLD HIRANGE RAD.	Ci/s-	0.685
RCIC FLOW	GPM-	0.000	SERVICEWATER RAD.	uCi/ml-	0.000
REACTOR PRESSURE	PSIG-	143.144	OFFGAS RAD.	mR/Hr-	4.031
HPCI PUMP FLOW	GPM-	1.406	DRYWELL RAD Monitor	R/Hr-	1.929
LPCI A FLOW	GPM-	-0.375	HIGEST MSL RAD MON	mR/Hr-	8.165
LPCI B FLOW	GPM-	-3.124	DRYWELL PRESSURE	PSIG-	0.154
UPPER CORESPRAY FLOW	GPM-	0.000	DRYWELL TEMPERATURE	DegF-	97.81
LOWER CORESPRAY FLOW	GPM-	0.000	TORUS WATER AVG TMP	DegF-	75.644
DRYWELL SUMP LEVEL	Inch-	15.000	TORUS WATER LEVEL	FT -	13.900
STACK GAS RAD.	uCi/ml-	83.911	DRYWELL H2 CONC	% -	0.354
RX BLDG VENT RAD.	uCi/ml-	17.378	DRYWELL O2 CONC	% -	23.957
RFUELFLR EXH RAD.	uCi/ml-	1.703	CS TANK LEVEL	Inch-	310.344

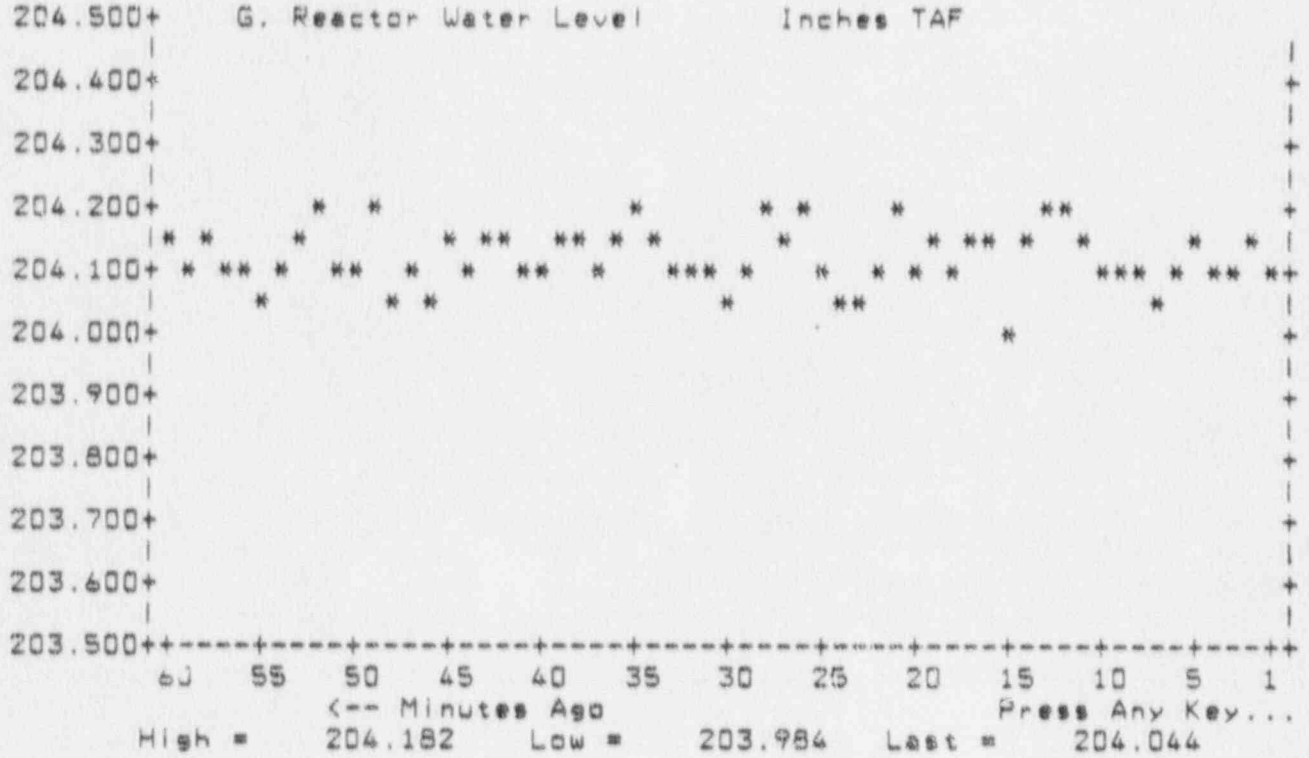
Press <RETURN> to Continue... P to Print

James A. FitzPatrick Nuclear Plant
Part III - Plant Parameters
Form EAP-1.1.3

Description	Units	Value
APRM REACTOR POWER	% -	-0.923
IRM REACTOR POWER	% -	17.763
SRM REACTOR POWER	CPS-	823.658
RX WATER LEVEL	Inch TAF-	204.132
FEEDWATER FLOW	MLB/HR-	0.959
RCIC FLOW	GPM-	0.000
REACTOR PRESSURE	PSIG-	143.144
HPCI PUMP FLOW	GPM-	1.406
LPCI A FLOW	GPM-	-0.375
LPCI B FLOW	GPM-	-3.124
UPPER CORESPRAY FLOW	GPM-	0.000
LOWER CORESPRAY FLOW	GPM-	0.000
DRYWELL SUMP LEVEL	Inch-	15.000
STACK GAS RAD.	uCi/ml-	83.911
RX BLDG VENT RAD.	uCi/ml-	17.378
RFUELFLR EXH RAD.	uCi/ml-	1.703
TB BLDG VENT RAD.	uCi/ml-	3.420
RW BLDG VENT RAD.	uCi/ml-	1.405
STACK HI RANGE RAD.	Ci/s-	0.461
TB BLD HIRANGE RAD.	Ci/s-	5.513
RW BLD HIRANGE RAD.	Ci/s-	0.685
SERVICEWATER RAD.	uCi/ml-	0.000
OFFGAS RAD.	mR/Hr-	4.031
DRYWELL RAD Monitor	R/Hr-	1.925
HIGEST MSL RAD MON	mR/Hr-	8.165
DRYWELL PRESSURE	PSIG-	0.154
DRYWELL TEMPERATURE	DegF-	97.817
TORUS WATER AVG TMP	DegF-	75.644
TORUS WATER LEVEL	FT -	13.900
DRYWELL H2 CONC	% -	0.354
DRYWELL O2 CONC	% -	23.957
CS TANK LEVEL	Inch-	310.344

Emergency Director Approval: _____
TIME: 15:41

```
    Trending - One (1) Hour History
==============================================
     A. Core Thermal Power        MW Thermal
     B. Gross Gen. Power          % Rated
     C. Gross Gen. Power          MWatt
     D. Cond Inlet/Dsch Tunnel    DT  Deg F
     E. Cinup Sys Inlet Temp.     Deg F
     F. Reactor Pressure          PSIG
     G. Reactor Water Level       Inches TAF
     H. Condsr 33C-10A Vacuum     Inches Hg
     I. RBCLC Water HX Outlet     Deg F.
     J. TBCLW Clr Outlet Temp.    Deg F.
     K. Drywell Pressure          PSIG
     L. Torus Pressure            PSIG
     M. Torus Wtr Lvl             Feet
     N. Torus Wtr Temp            Deg F.
     O. Drywell Oxygen (0/30%)    % Volume
     P. Main Steam Pressure       PSIG
     Q. Vessel Bottom Drain T.    Deg F.
     R. Stack  Hi Range Rad.      mR/Hr
     S. T-Bldg Hi Range Rad.      mR/Hr
     T. Rdwst  Hi Range Rad.      mR/Hr.
==============================================
  Enter Choice [A-T], or 0 to Quit:
```



```
*** 15-Minute Average Met Data *****
      Date:  6/21/90  Time: 13:45
========== Main Tower ==============
 30 ft wind speed (mph)      =    1.7
 30 ft wind direction (deg)  =  348.
 30 ft sigma-theta (deg)     =   27.3
 30 ft temperature (deg f)   =   65.4
100 ft wind speed (mph)      =    0.0
100 ft wind direction (deg)  =    0.
100 ft delta t   (deg f)     =    0.0
200 ft wind speed (mph)      =    4.0
200 ft wind direction (deg)  =  315.
200 ft sigma-theta (deg)     =    5.7
200 ft delta t   (deg f)     =    2.2
 ========== Backup Tower ========:
96 ft wind speed (mph)       =    5.0
96 ft wind direction (deg)   =  328.
========== Inland Tower ============
30 ft wind speed (mph)       =    0.0
30 ft wind direction (deg)   =    0.
30 ft sigma-theta (deg)      =    0.0
```

Press RETURN for Main Menu:

SECTION 7

James A. FitzPatrick Nuclear Power Plant
1990 Partial Scale Emergency Exercise

December 19, 1990

ARMs and In-Plant Maps

Timeline of exercise ARM readings are attached. These values are for controllers use only.

Also attached are in-plant maps which may be used for reference when repair teams are dispatched into the plant. These in-plant maps are to be used when providing all in-plant radiological data. The controllers shall provide all non-radiological data (i.e. status of repairs, valve indications, pressure readings, etc.) to drill participants in accordance with Section 6 of this scenario package and the damage repair reports.

Any area that is entered which does not appear on these in-plant maps shall have radiological conditions which are normal to that area under operating conditions.

Any area that is entered at a time which does not appear on these in-plant maps shall also have radiological conditions which are normal to that area under operating conditions.

SECTION 7

AREA RADIATION MONITORS DATA SHEET

(All readings are mR/hr)

ARM #	ARM Location	0700 to 0815	0815 to 0900	0900 to 1100	1100 to 1130	1130 to 1200
1	AB 272' Chem Lab	Normal	Normal	Normal	Normal	Normal
2	AB Change Area	Normal	Normal	Normal	Normal	Normal
3	AB CR	Normal	Normal	Normal	Normal	Normal
4	TB 300' HP End	Normal	145	Upscale Hi	195	150
5	TB 300' LP End	Normal	118	Upscale Hi	138	110
6	TB 252' Hogging	Normal	85	Upscale Hi	105	100
7	TB 252' Feed Pump	Normal	150	Upscale Hi	150	120
8	RW CR 284'	Normal	Normal	Normal	Normal	Normal
9	RW Filter/Vlv 279'	Normal	Normal	Normal	Normal	Normal
10	RW Drum Area 272	Normal	Normal	Normal	Normal	Normal
11	RW Pmp Rm 252'	Normal	Normal	Normal	Normal	Normal
12	RxB 369' Fuel Pool	Normal	Normal	Normal	Normal	Normal
13	RxB 344' S. End	Normal	Normal	Normal	Normal	Normal
14	369' Fuel Vault	Normal	Normal	Normal	Normal	Normal
15	326' Cleanup Tank	Normal	Normal	Normal	Normal	Normal
16	300' Clnup Hx	Normal	Normal	Normal	Normal	Normal
17	326' Pool Pmps	Normal	Normal	Normal	Normal	Normal
18	326' Decon Storage	Normal	Normal	Normal	Normal	Normal
19	300' Rx Clnup Pmp	Normal	Normal	Normal	Normal	Normal
20	300' Rx Sample Sta	Normal	Normal	Normal	Normal	Normal
21	300' CLCW Hx	Normal	Normal	Normal	Normal	Normal
22	TB 252' Demin Vlv	Normal	160	Upscale Hi	180	150
23	RB 272' Access	Normal	Normal	Normal	Normal	Normal
24	RB 284' TIP	Normal	Normal	Normal	Normal	Normal
25	272' East CRD	Normal	Normal	Normal	Normal	Normal
26	272' West CRD	Normal	Normal	Normal	Normal	Normal
27	227' Equip Sump	Normal	Normal	Normal	Normal	Normal
28	272' CRD Hatch	Normal	Normal	Normal	Normal	Normal
29	227' West Sump	Normal	Normal	Normal	Normal	Normal
30	369' S. Fuel Fl	Normal	Normal	Normal	Normal	Normal

SECTION 7

INPLANT
AIR SAMPLE DATA

(Field Readings in R/hr)
(Concentrations in μCi/cc)

NUCLIDE	0-1	1.1-10.0	10.1-20.0	20.1-50.0	50.1-100	(R/hr)
Kr-85	8.4E-06	8.4E-05	2.5E-04	4.0E-04	1.2E-03	
Kr-85m	8.4E-05	8.4E-04	2.5E-03	4.0E-03	1.2E-02	
Kr-87	1.5E-04	1.5E-03	4.3E-03	7.0E-03	2.1E-02	
Kr-88	2.1E-04	2.1E-03	6.2E-03	1.0E-02	3.0E-02	
Xe-131m	2.5E-04	2.5E-03	7.4E-03	1.2E-02	3.6E-02	
Xe-133	7.8E-04	7.8E-03	2.3E-02	3.7E-02	1.1E-01	
Xe-133m	1.1E-04	1.0E-03	3.1E-03	5.0E-03	1.5E-02	
Xe-135	8.4E-05	8.4E-04	2.5E-03	4.0E-03	1.2E-02	
Xe-135m	1.3E-04	1.3E-03	3.7E-03	6.0E-03	1.8E-02	
I-131	8.4E-08	8.4E-07	2.5E-06	4.0E-06	1.2E-05	
I-133	1.2E-07	1.2E-06	3.5E-06	5.7E-06	1.7E-05	
I-135	7.1E-08	7.1E-07	2.1E-06	3.4E-06	1.0E-05	
TOT. CONC.	1.8E-03	1.8E-02	5.29E-02	8.54E-02	2.55E-01	(μCi/cc)

NOTE: Information on this sheet is for controller use only. This page will not be handed to drill players.

RES SURVEY LOG SHEET

FORM NO SSL-1

☐ RWP NO. _____
☐ RPOP-5 _____
☐ _____

SURVEY NO. _____

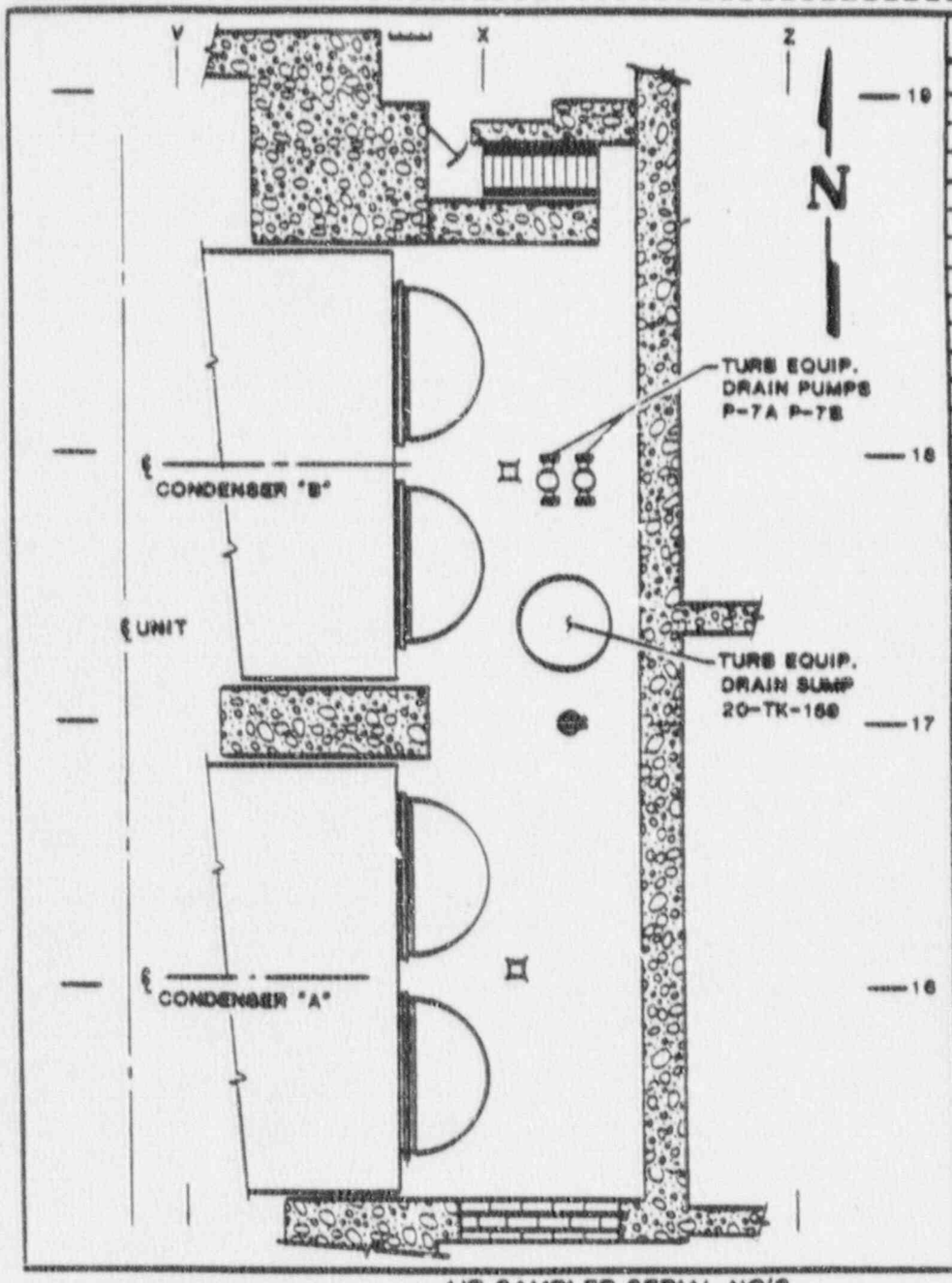
DATE _____

TIME _____

BUILDING / ELEVATION TB 244-1

SYSTEM & COMPONENT I D / ELEVATION _____

DESCRIPTION / COMMENT _____



RADIATION		T.M.
1	Normal	0705
2	Normal	0815
3	120 mR/hr	0910
4	1600 "	1000
5	150 "	1130
6	120 "	1200
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CONTAMINATION		
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☐ AIRBORNE SURVEY AIR SAMPLER SERIAL NO'S. _____ ; _____

AIR SAMPLE RESULTS: _____ ; _____ ; _____ ; _____ ; _____ ; _____ ; _____
(% EFFECTIVE MPC)

SURVEYOR(S) _____ EXPOSURE _____ (mR) REVIEW _____
(SIGNATURE)

_____ _____ (mR)
(SIGNATURE)(S)

REV 0 (SEE OTHER SIDE FOR INSTRUMENT DATA & SURVEY LOG KEY)

RES SURVEY LOG SHEET

FORM NO SSL-1

☐ RWP NO. _____
☐ RPOP-5 _____
☐ _____

SURVEY NO.

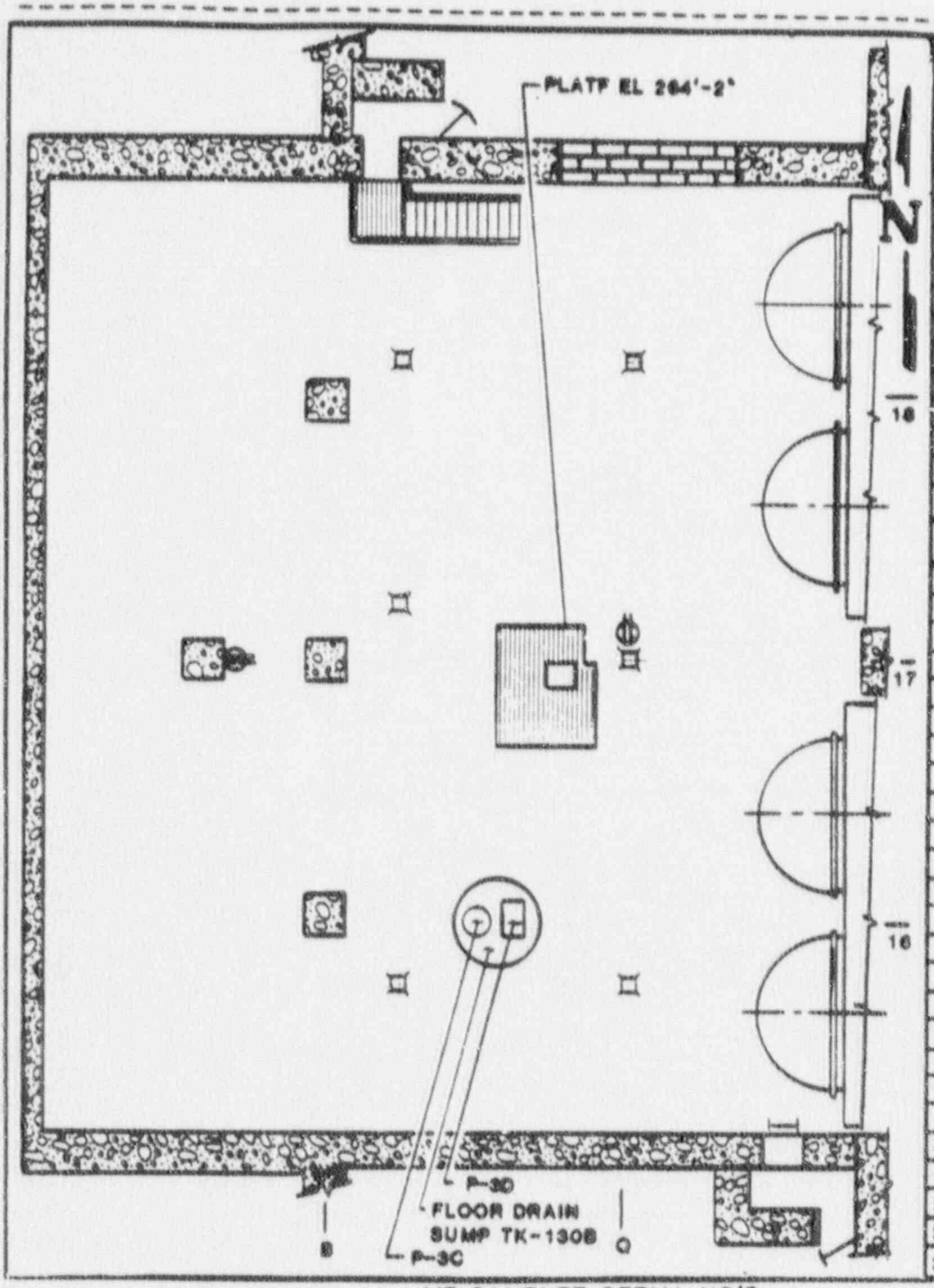
BUILDING / ELEVATION TB 244-2

DATE _____

SYSTEM & COMPONENT I D / ELEVATION _____

TIME _____

DESCRIPTION / COMMENT _____



RADIATION/mR		
1	Normal	0700
2	120 mR/hr	09'0
3	1600 "	1000
4	150 "	1130
5	120 "	1200
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☐ AIRBORNE SURVEY

AIR SAMPLER SERIAL NO'S. _____ ; _____

AIR SAMPLE RESULTS: _____ ; _____ ; _____ ; _____ ; _____ ; _____ ; _____

(% EFFECTIVE MPC)

SURVEYOR(S) _____ EXPOSURE _____ (mR) REVIEW _____

(SIGNATURE)(S) _____ (mR) (SIGNATURE)

REV 1

(SEE OTHER SIDE FOR INSTRUMENT DATA & SURVEY LOG KEY)

RES SURVEY LOG SHEET

FORM NO SSL-1

☐ RWP NO. _____
☐ RPOP-5 _____
☐ _____

SURVEY NO. _____

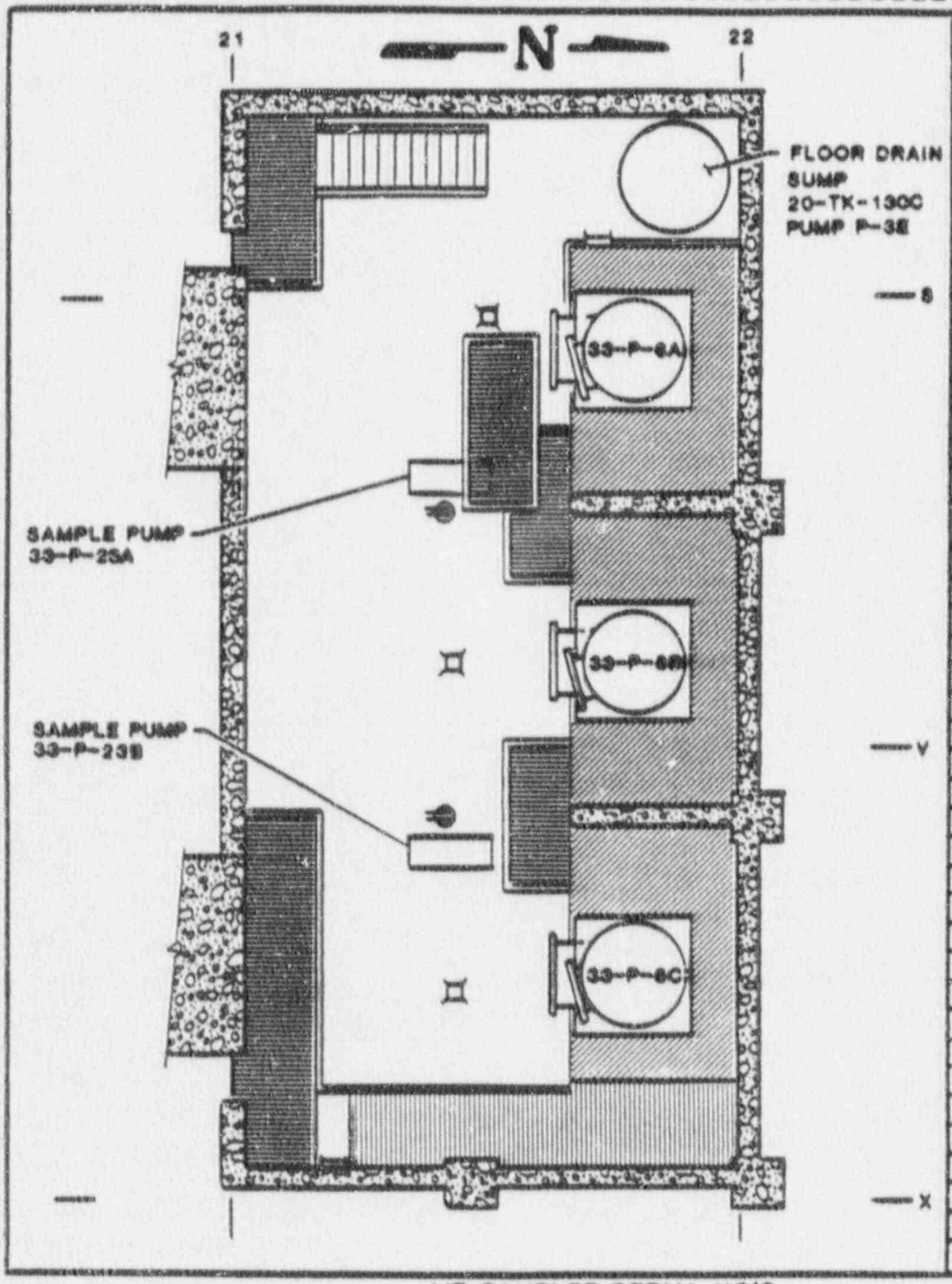
DATE _____

TIME _____

BUILDING / ELEVATION TB 244-3

SYSTEM & COMPONENT I D / ELEVATION _____

DESCRIPTION / COMMENT _____



RADIATION		
1	Normal	07
2	120 mR/hr	091
3	1600 "	105
4	150 "	113
5	120 "	120
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☐ AIRBORNE SURVEY

AIR SAMPLER SERIAL NO'S. _____ ; _____

AIR SAMPLE RESULTS: _____ ; _____ ; _____ ; _____ ; _____ ; _____ ; _____

(% EFFECTIVE MPC)

SURVEYOR(S) _____ EXPOSURE _____ (mR) REVIEW _____

(SIGNATURE)

_____ _____ (mR)

(SIGNATURE)(S)

REV 0

(SEE OTHER SIDE FOR INSTRUMENT DATA & SURVEY LOG KEY

RES SURVEY LOG SHEET

FORM NO SSL-1

☐ RWP NO. _____
☐ RPOP-5 _____
☐ _____

SURVEY NO. _____

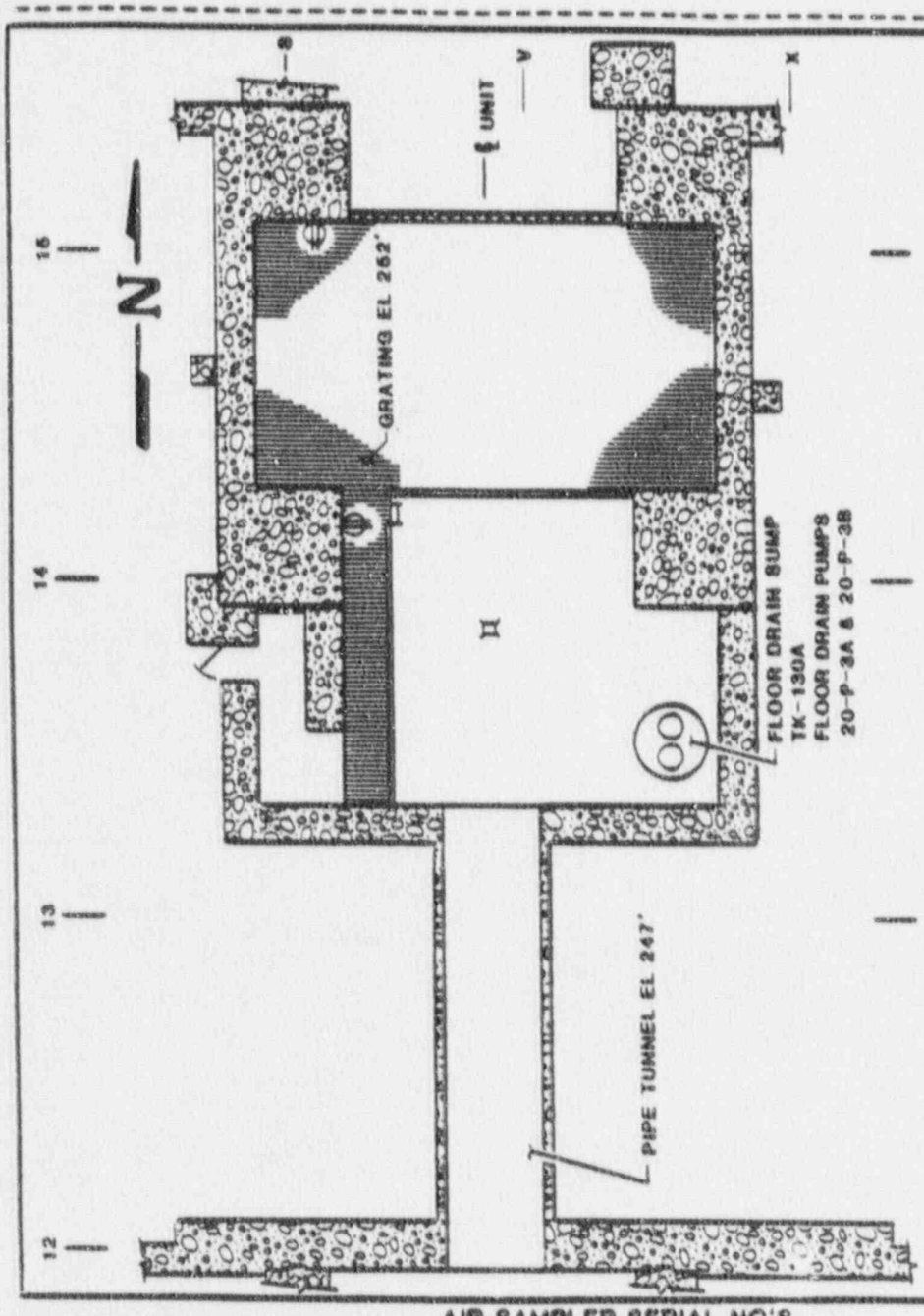
DATE _____

TIME _____

BUILDING / ELEVATION TB 252-1

SYSTEM & COMPONENT I D / ELEVATION _____

DESCRIPTION / COMMENT _____



RADIATION		Time
1	Normal	0700
2	120 mR/hr	0910
3	1600 "	1000
4	150 "	1130
5	120 "	1200
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☐ AIRBORNE SURVEY

AIR SAMPLER SERIAL NO'S. _____; _____

AIR SAMPLE RESULTS: _____; _____; _____; _____; _____; _____; _____

(% EFFECTIVE MPC)

SURVEYOR(S) _____ EXPOSURE _____ (mR) REVIEW _____ (SIGNATURE)

_____ _____ (mR)

(SIGNATURE)(S)

REV 0 (SEE OTHER SIDE FOR INSTRUMENT DATA & SURVEY KEY)

RES SURVEY LOG SHEET

FORM NO SSL-1

☐ RWP NO. _____
☐ RPOP-5 _____
☐ _____

SURVEY NO. _____

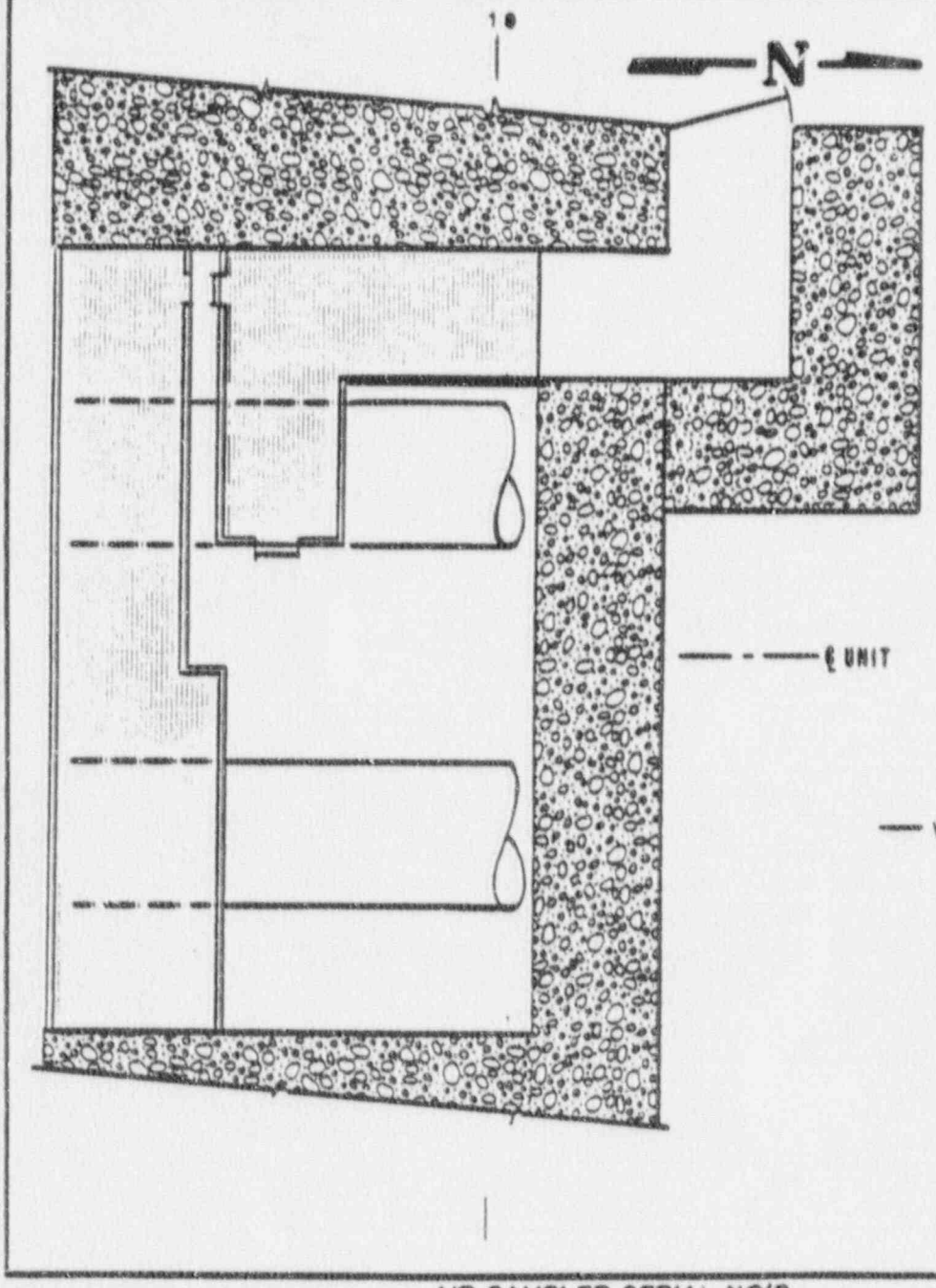
DATE _____

TIME _____

BUILDING / ELEVATION TB 252-2

SYSTEM & COMPONENT I D / ELEVATION _____

DESCRIPTION / COMMENT _____



RADIATION		
1	Normal	0.7
2	120 MR/hr	0.9
3	1600 "	12
4	150 "	11.3
5	120 "	12
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☐ AIRBORNE SURVEY AIR SAMPLER SERIAL NO'S. _____; _____

AIR SAMPLE RESULTS: _____; _____; _____; _____; _____; _____; _____
(% EFFECTIVE MPC)

SURVEYOR(S) _____ EXPOSURE _____ (mR) REVIEW _____
(SIGNATURE)

_____ _____ (mR)
(SIGNATURE)(S)

REV 0

(SEE OTHER SIDE FOR INSTRUMENT DATA & SURVEY LOG KEY

RES SURVEY LOG SHEET

FORM NO SSL-1

☐ RWP NO. _____
☐ RPOP-5 _____
☐ _____

SURVEY NO. _____

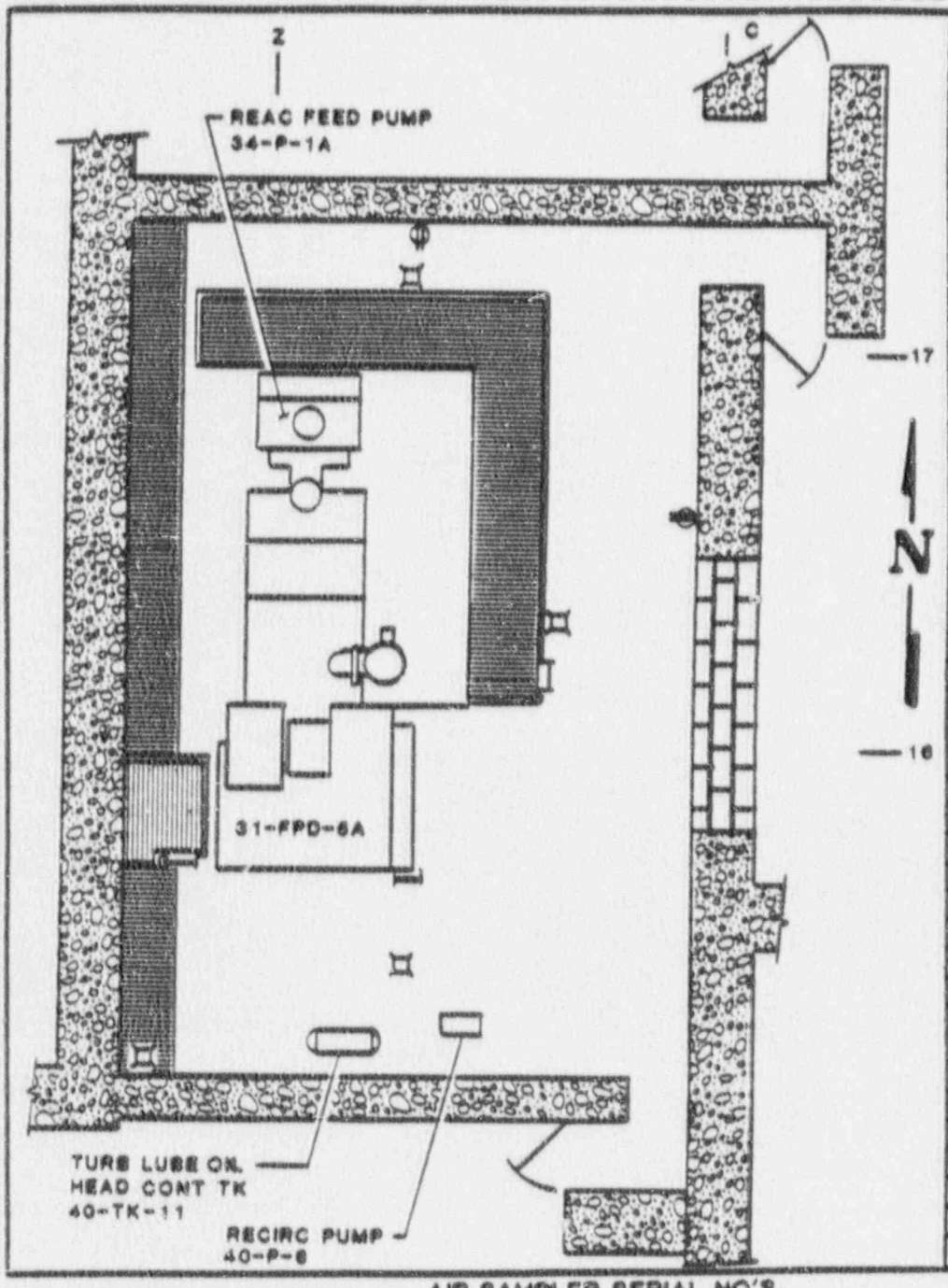
DATE _____

TIME _____

BUILDING / ELEVATION TB 252-3

SYSTEM & COMPONENT I D / ELEVATION _____

DESCRIPTION / COMMENT _____



	RADIATION	T.
1	NBBMA/	07
2	120 MR/HR	09
3	1600 MR/HR	10
4	150 "	11
5	120 "	12
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☐ AIRBORNE SURVEY

AIR SAMPLER SERIAL NO'S. _____; _____

AIR SAMPLE RESULTS: _____; _____; _____; _____; _____; _____; _____

(% EFFECTIVE MPC)

SURVEYOR(S) _____ EXPOSURE _____ (mR) REVIEW _____ (SIGNATURE)

_____ _____ (mR)

(SIGNATURE)(S)

REV 0

(SEE OTHER SIDE FOR INSTRUMENT DATA & SURVEY LOG KE

RES SURVEY LOG SHEET

FORM NO SSL-1

☐ RWP NO. _____
☐ RPOP-5 _____
☐ _____

SURVEY NO. _____

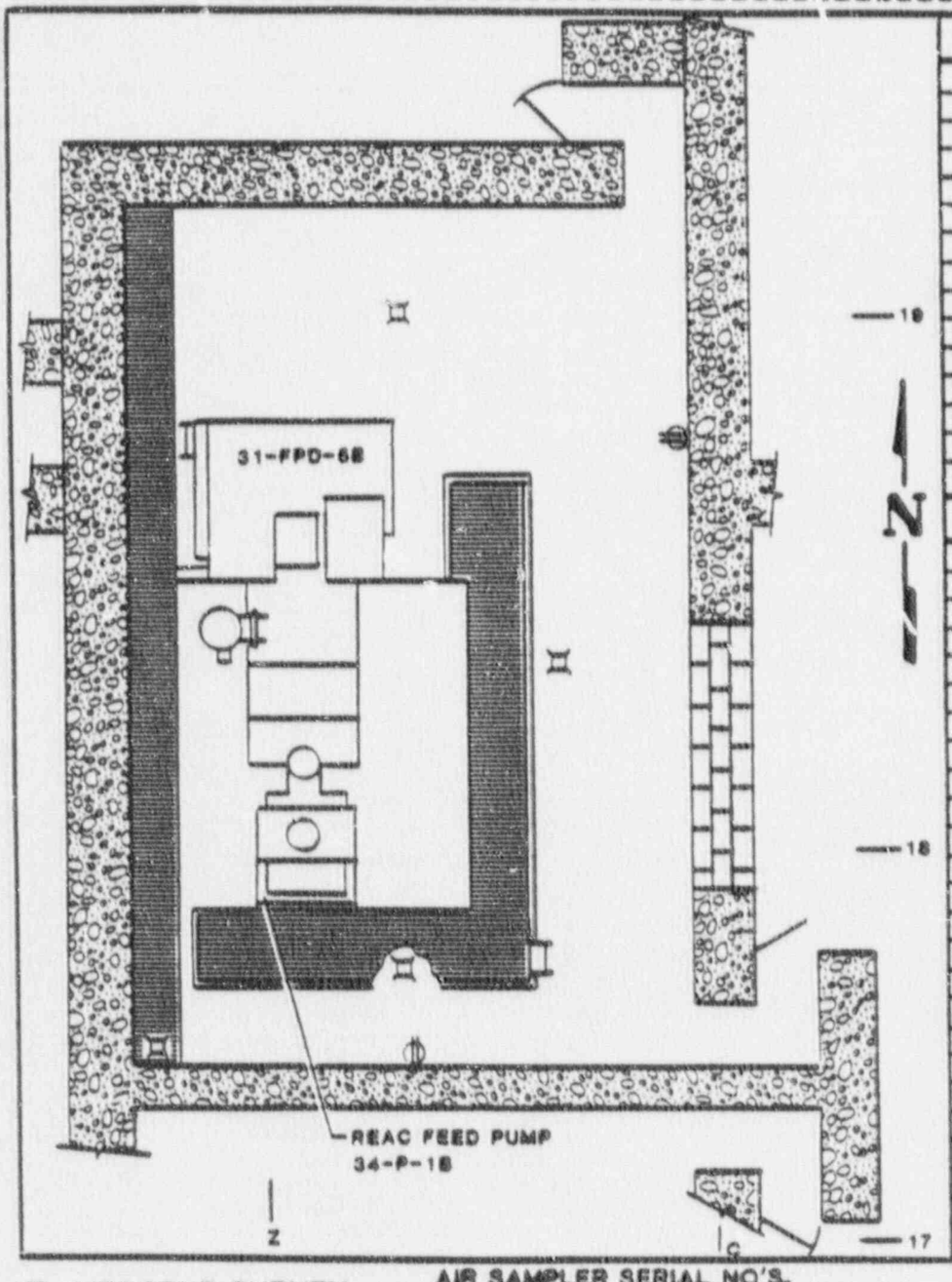
DATE _____

TIME _____

BUILDING / ELEVATION TB 252-4

SYSTEM & COMPONENT I D / ELEVATION _____

DESCRIPTION / COMMENT _____



	RADIATION	mR/hr
1	Normal	0.2
2	120 MR/HR	0.6
3	1600 "	100
4	150 "	1.5
5	120 "	1.2
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☐ AIRBORNE SURVEY AIR SAMPLER SERIAL NO'S. _____ ; _____

AIR SAMPLE RESULTS: _____ ; _____ ; _____ ; _____ ; _____ ; _____ ; _____
(% EFFECTIVE MPC)

SURVEYOR(S) _____ EXPOSURE _____ (mR) REVIEW _____
(SIGNATURE)

_____ _____ (mR)
(SIGNATURE)(S)

REV 0

(SEE OTHER SIDE FOR INSTRUMENT DATA & SURVEY LOG KEY

RES SURVEY LOG SHEET

FORM NO SSL-1

☐ RWP NO. _____
☐ RPOP-5 _____
☐ _____

SURVEY NO.

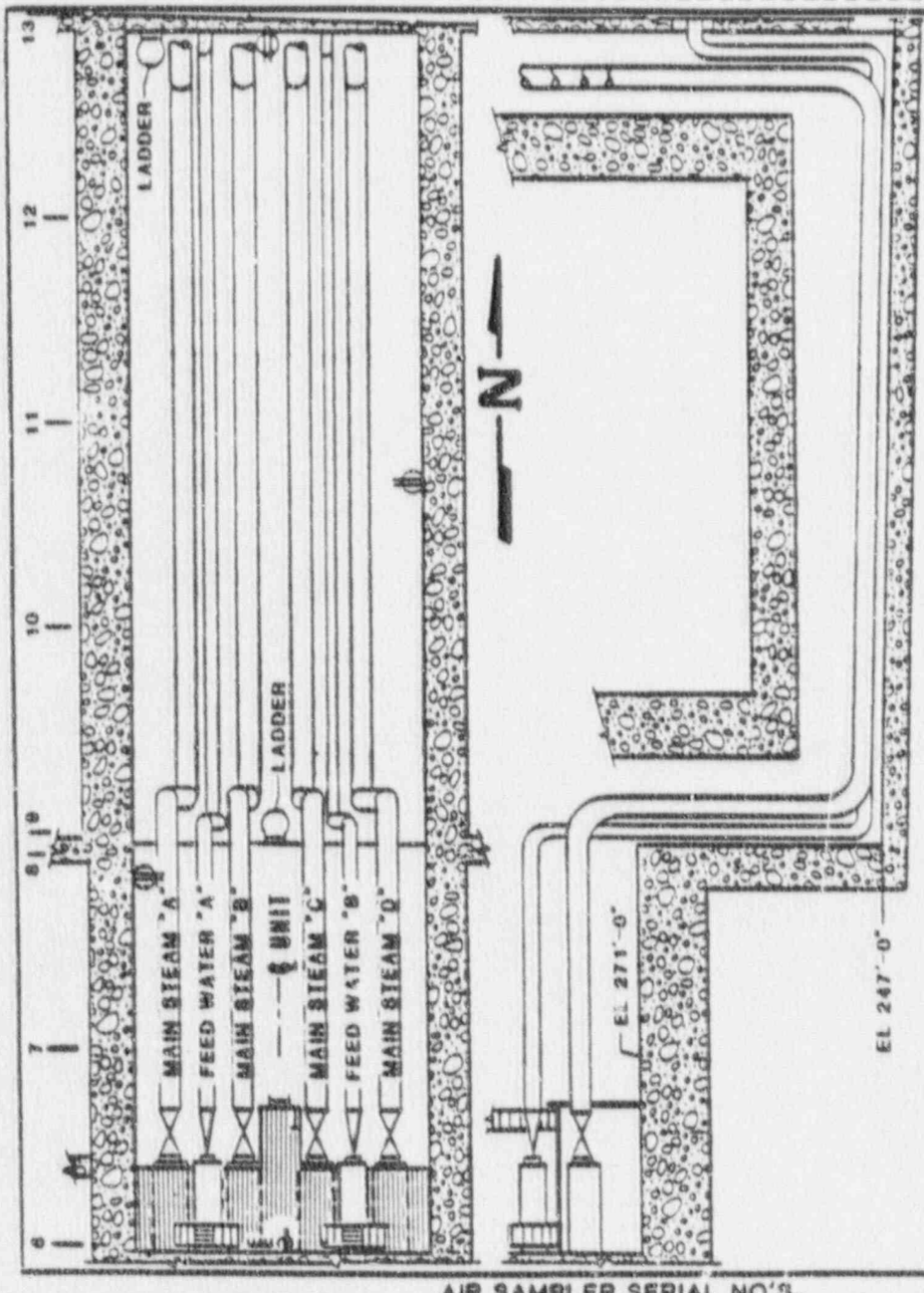
DATE _____

TIME _____

BUILDING / ELEVATION TB 252-5

SYSTEM & COMPONENT I D / ELEVATION _____

DESCRIPTION / COMMENT _____



RADIATION		TIME
1	Normal	0700
2	2000 MR/hr	0901
3	4000 "	0916
4	5000 "	1006
5	3000 "	1126
6	1000 "	1200
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☐ AIRBORNE SURVEY AIR SAMPLER SERIAL NO'S. _____ ; _____

AIR SAMPLE RESULTS: _____ ; _____ ; _____ ; _____ ; _____ ; _____ ; _____

(% EFFECTIVE MPC)

SURVEYOR(S) _____ EXPOSURE _____ (mR) REVIEW _____

(SIGNATURE)

_____ _____ (mR)

(SIGNATURE)(S)

REV 0

(SEE OTHER SIDE FOR INSTRUMENT DATA & SURVEY LOG KEY)

RES SURVEY LOG SHEET

FORM NO SSL-1

☐ RWP NO. _____
☐ RPOP-5 _____
☐ _____

SURVEY NO. _____

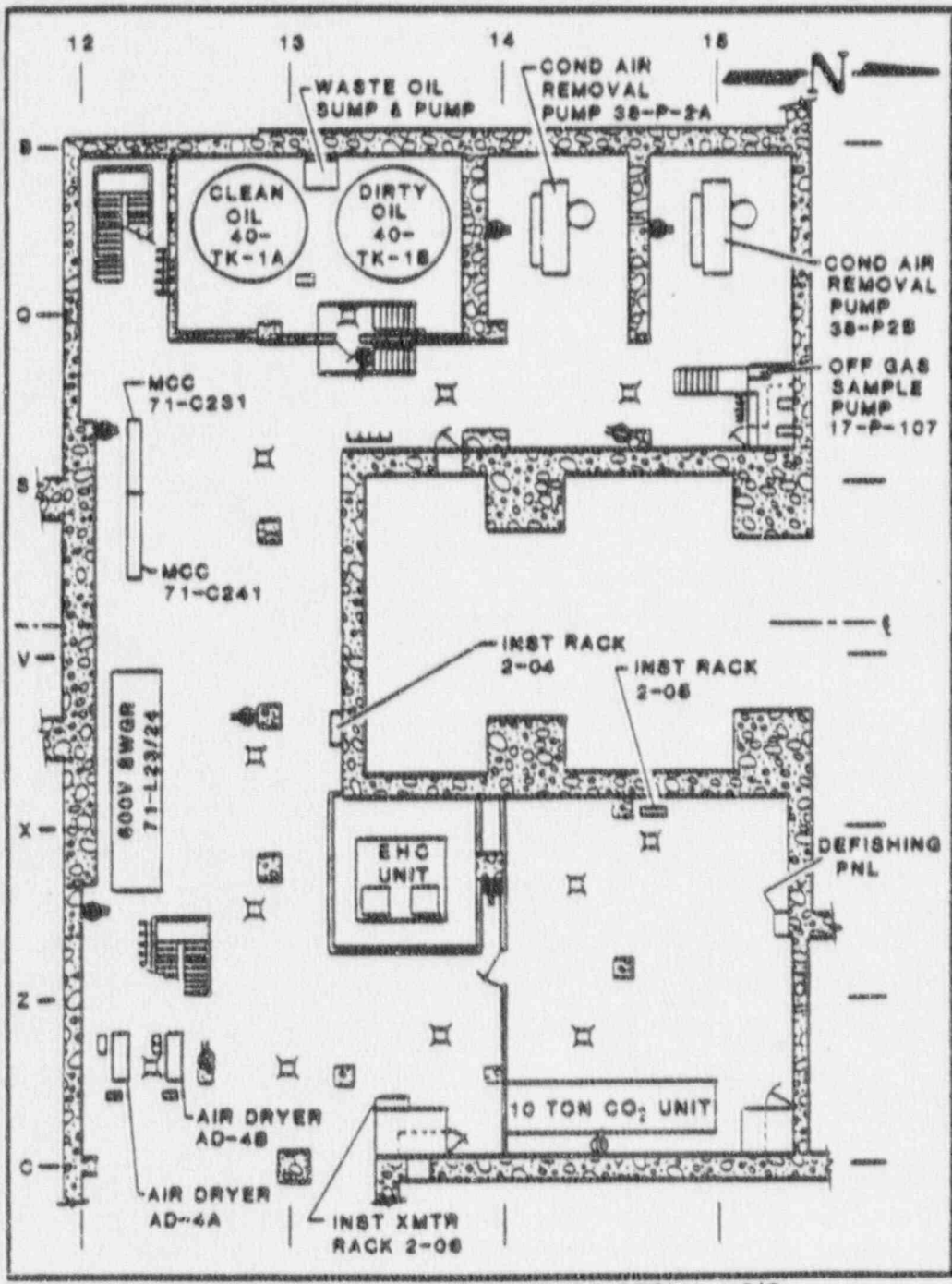
DATE _____

TIME _____

BUILDING / ELEVATION TB 252-6

SYSTEM & COMPONENT I D / ELEVATION _____

DESCRIPTION / COMMENT _____



RADIATION		Time
1	Normal	0700
2	120 mR/hr	0915
3	160R "	1000
4	150 "	1130
5	120 "	1200
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CONTAMINATION		
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☐ AIRBORNE SURVEY

AIR SAMPLER SERIAL NO'S. _____ ; _____

AIR SAMPLE RESULTS: _____ ; _____ ; _____ ; _____ ; _____ ; _____ ; _____

(% EFFECTIVE MPC)

SURVEYOR(S) _____ EXPOSURE _____ (mR) REVIEW _____ (SIGNATURE)

_____ _____ (mR)

(SIGNATURE)(S)

REV 0

(SEE OTHER SIDE FOR INSTRUMENT DATA & SURVEY LOG KEY

RES SURVEY LOG SHEET

FORM NO SSL-1

☐ RWP NO. _____
☐ RPOP-5 _____
☐ _____

SURVEY NO.

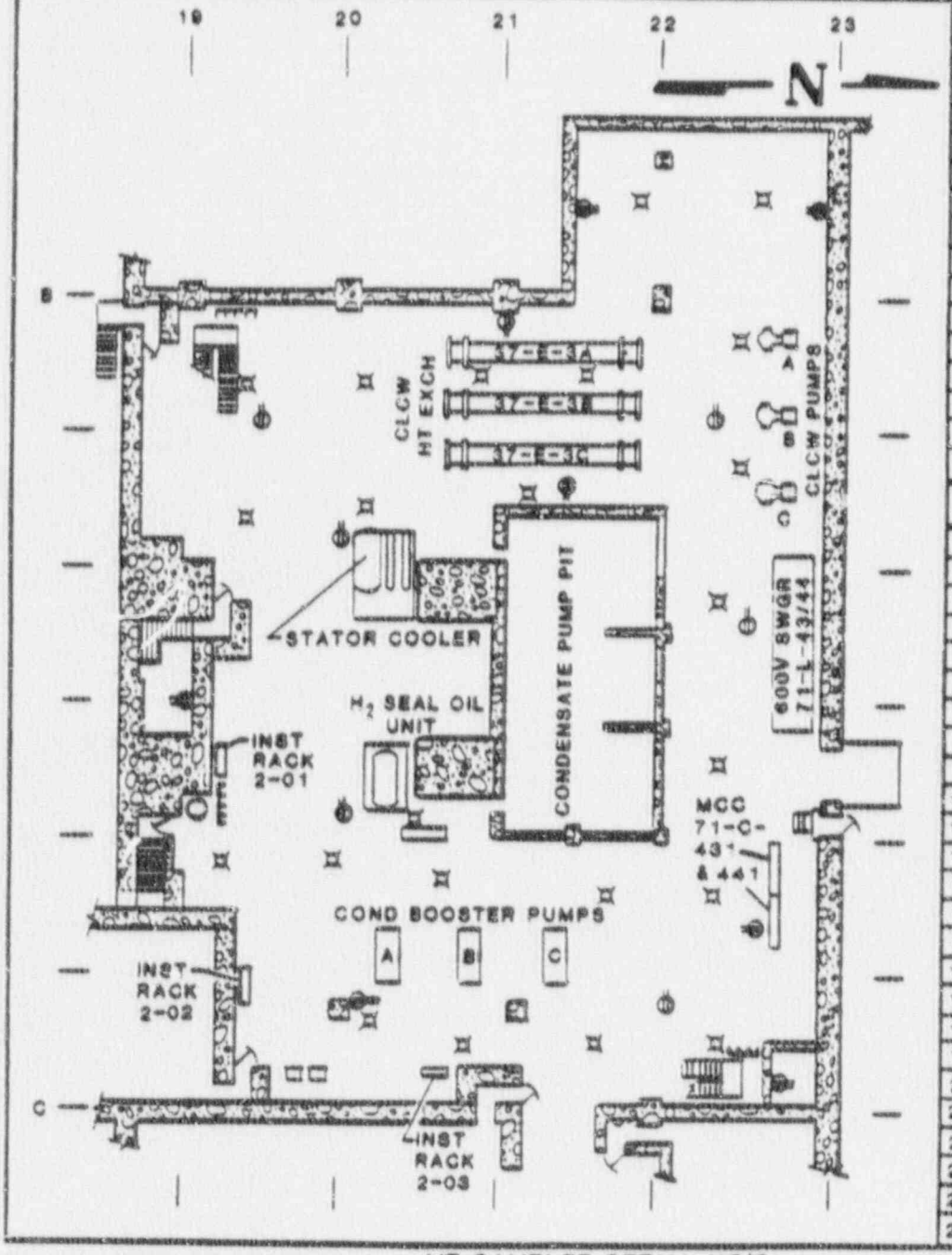
DATE _____

TIME _____

BUILDING / ELEVATION TB 252-7

SYSTEM & COMPONENT I D / ELEVATION _____

DESCRIPTION / COMMENT _____



RADIATION		
1	Normal	07
2	120 MR/HR	09
3	1600 "	101
4	150 "	11
5	120 "	12
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CONTAMINATION	
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☐ AIRBORNE SURVEY

AIR SAMPLER SERIAL NO'S. _____ ; _____

AIR SAMPLE RESULTS: _____ ; _____ ; _____ ; _____ ; _____ ; _____ ; _____

(% EFFECTIVE MPC)

SURVEYOR(S) _____ EXPOSURE _____ (mR) REVIEW _____

(SIGNATURE)

_____ _____ (mR)

(SIGNATURE)(S)

REV 0

(SEE OTHER SIDE FOR INSTRUMENT DATA & SURVEY LOG KEY

RES SURVEY LOG SHEET

FORM NO SSL-1

☐ RWP NO. _____
☐ RPOP-5 _____
☐ _____

SURVEY NO.

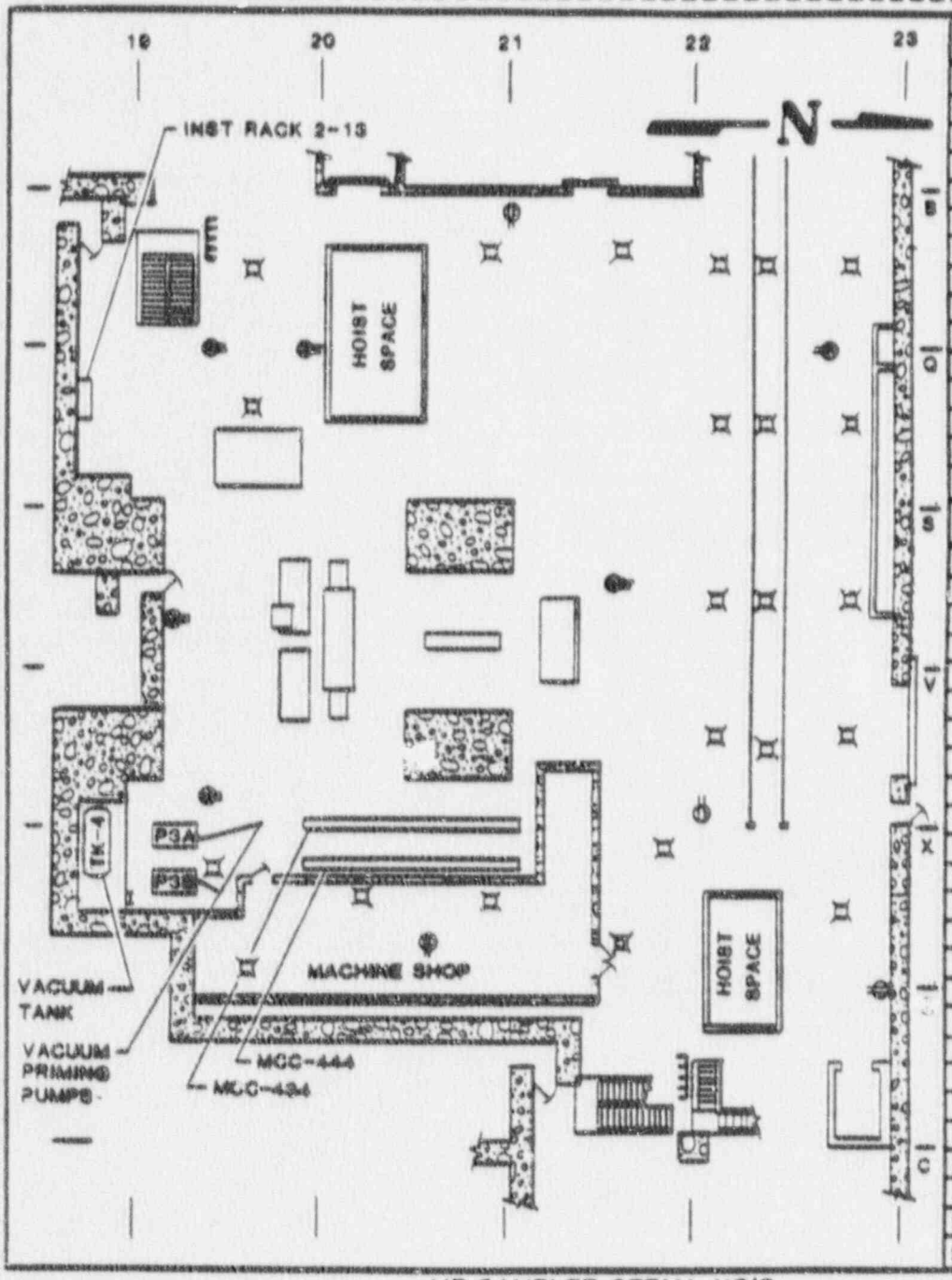
DATE _____

TIME _____

BUILDING / ELEVATION TB 272-2

SYSTEM & COMPONENT I D / ELEVATION _____

DESCRIPTION / COMMENT _____



RADIATION		
1	Normal	0700
2	140 mR/hr	0910
3	1200 "	1000
4	160 "	1130
5	120 "	1200
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☐ AIRBORNE SURVEY AIR SAMPLER SERIAL NO'S. _____ ; _____

AIR SAMPLE RESULTS: _____ ; _____ ; _____ ; _____ ; _____ ; _____ ; _____
(% EFFECTIVE MPC)

SURVEYOR(S) _____ EXPOSURE _____ (mR) REVIEW _____
(SIGNATURE)

_____ _____ (mR)
(SIGNATURE)(S)

REV 0

(SEE OTHER SIDE FOR INSTRUMENT DATA & SURVEY LOG KEY)

RES SURVEY LOG SHEET

FORM NO SSL-1

☐ RWP NO. _____
☐ RPOP-5 _____
☐ _____

SURVEY NO. _____

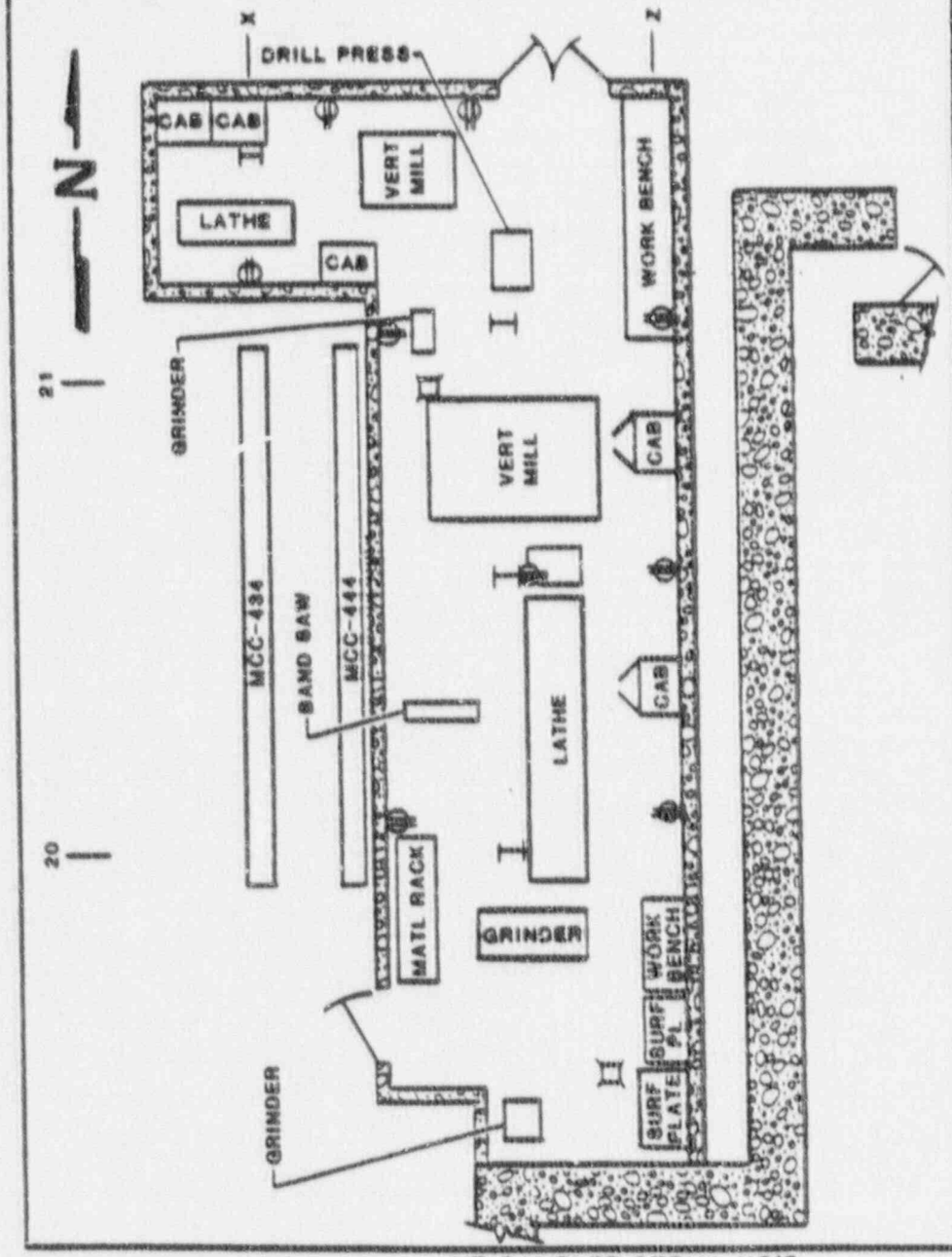
DATE _____

TIME _____

BUILDING / ELEVATION TB 272-3

SYSTEM & COMPONENT I D / ELEVATION _____

DESCRIPTION / COMMENT _____



RADIATION		
1	NORMAL	0741
2	140 MR/HR	0915
3	1200 "	1205
4	160 "	1135
5	120 "	1201
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CONTAMINATION		
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☐ AIRBORNE SURVEY

AIR SAMPLER SERIAL NO'S. _____ ; _____

AIR SAMPLE RESULTS: _____ ; _____ ; _____ ; _____ ; _____ ; _____ ; _____
(% EFFECTIVE MPC)

SURVEYOR(S) _____ EXPOSURE _____ (mR) REVIEW _____
(SIGNATURE)

_____ _____ (mR)
(SIGNATURE)(S)

REV 0

(SEE OTHER SIDE FOR INSTRUMENT DATA & SURVEY LOG KEY

RES SURVEY LOG SHEET

FORM NO SSL-1

☐ RWP NO. _____
☐ RPOP-5 _____
☐ _____

SURVEY NO. _____

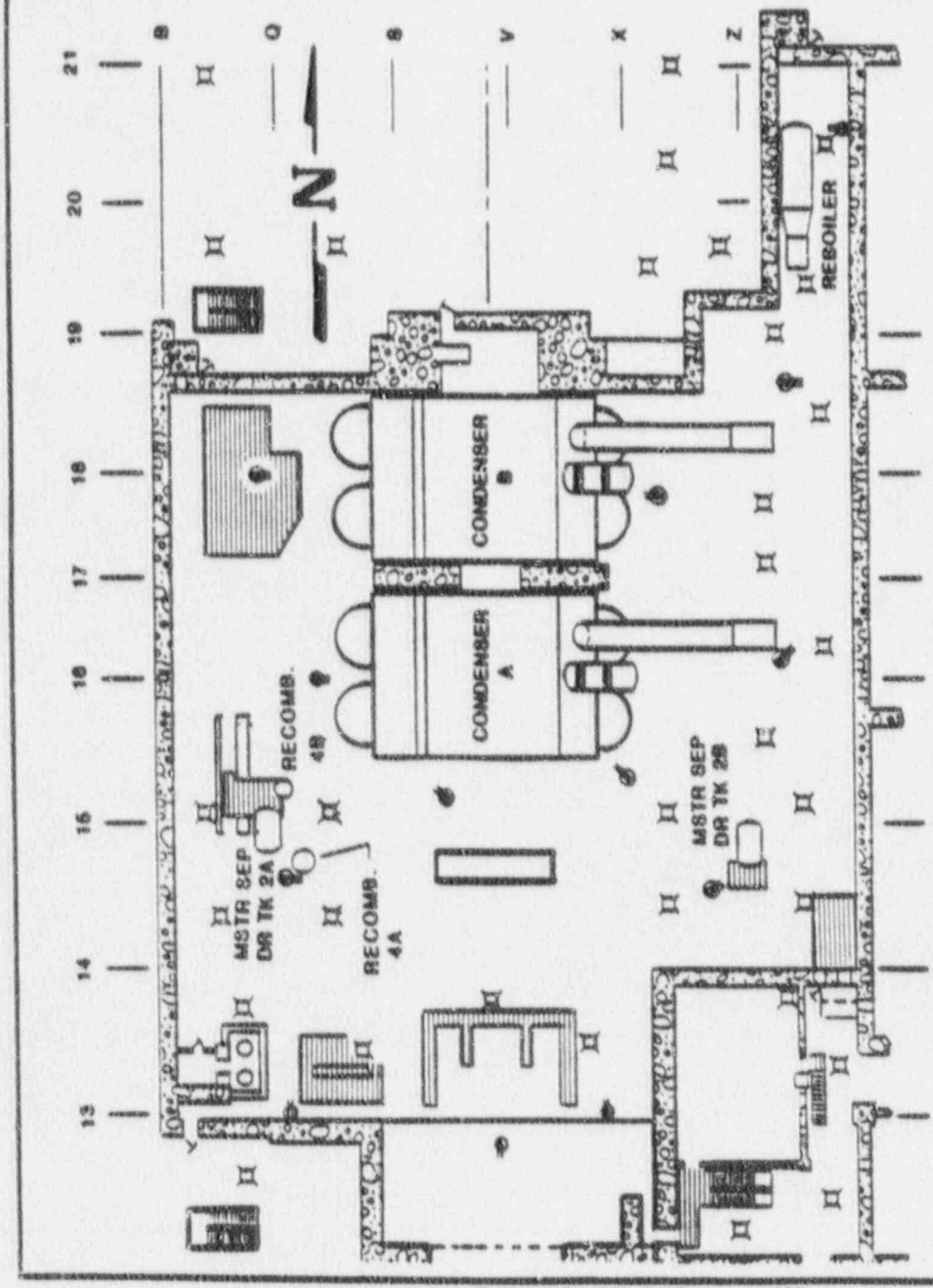
DATE _____

TIME _____

BUILDING / ELEVATION TB 272-4

SYSTEM & COMPONENT I D / ELEVATION _____

DESCRIPTION / COMMENT _____



RADIATION		
1	Normal	0700
2	140 MR/hr	0915
3	1200 "	1000
4	160 "	1134
5	120 "	1206
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CONTAMINATION		
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☐ AIRBORNE SURVEY AIR SAMPLER SERIAL NO'S. _____ ; _____

AIR SAMPLE RESULTS: _____ ; _____ ; _____ ; _____ ; _____ ; _____ ; _____

(% EFFECTIVE MPC)

SURVEYOR(S) _____ EXPOSURE _____ (mR) REVIEW _____

(SIGNATURE)

_____ _____ (mR)

(SIGNATURE)(S)

REV 0

(SEE OTHER SIDE FOR INSTRUMENT DATA & SURVEY LOG KEY)

RES SURVEY LOG SHEET

FORM NO SSL-1

☐ RWP NO. _____
☐ RPOP-5 _____
☐ _____

SURVEY NO. _____

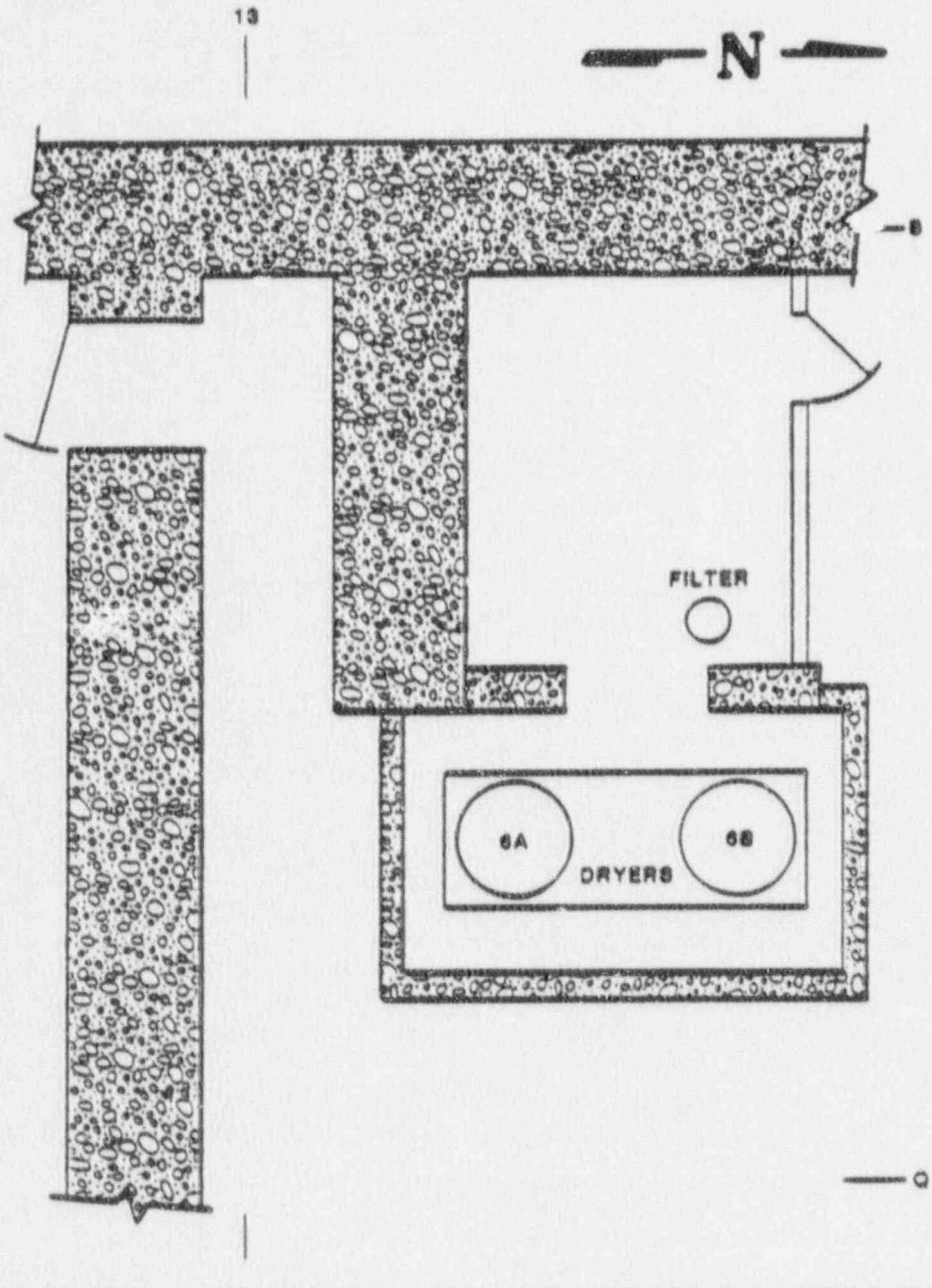
DATE _____

TIME _____

BUILDING / ELEVATION TB 272-5

SYSTEM & COMPONENT I D / ELEVATION _____

DESCRIPTION / COMMENT _____



RADIATION		
1	NORMAL	070
2	140 MR/HR	091
3	1700 "	100
4	160 "	113
5	120 "	120
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CONTAMINATION		
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☐ AIRBORNE SURVEY

AIR SAMPLER SERIAL NO'S. _____ ; _____

AIR SAMPLE RESULTS: _____ ; _____ ; _____ ; _____ ; _____ ; _____ ; _____
(% EFFECTIVE MPC)

SURVEYOR(S) _____ EXPOSURE _____ (mR) REVIEW _____
(SIGNATURE)

_____ _____ (mR)
(SIGNATURE)(S)

REV 0

(SEE OTHER SIDE FOR INSTRUMENT DATA & SURVEY LOG KEY

RES SURVEY LOG SHEET

FORM NO SSL-1

☐ RWP NO. _____
☐ RPOP-5 _____
☐ _____

SURVEY NO. _____

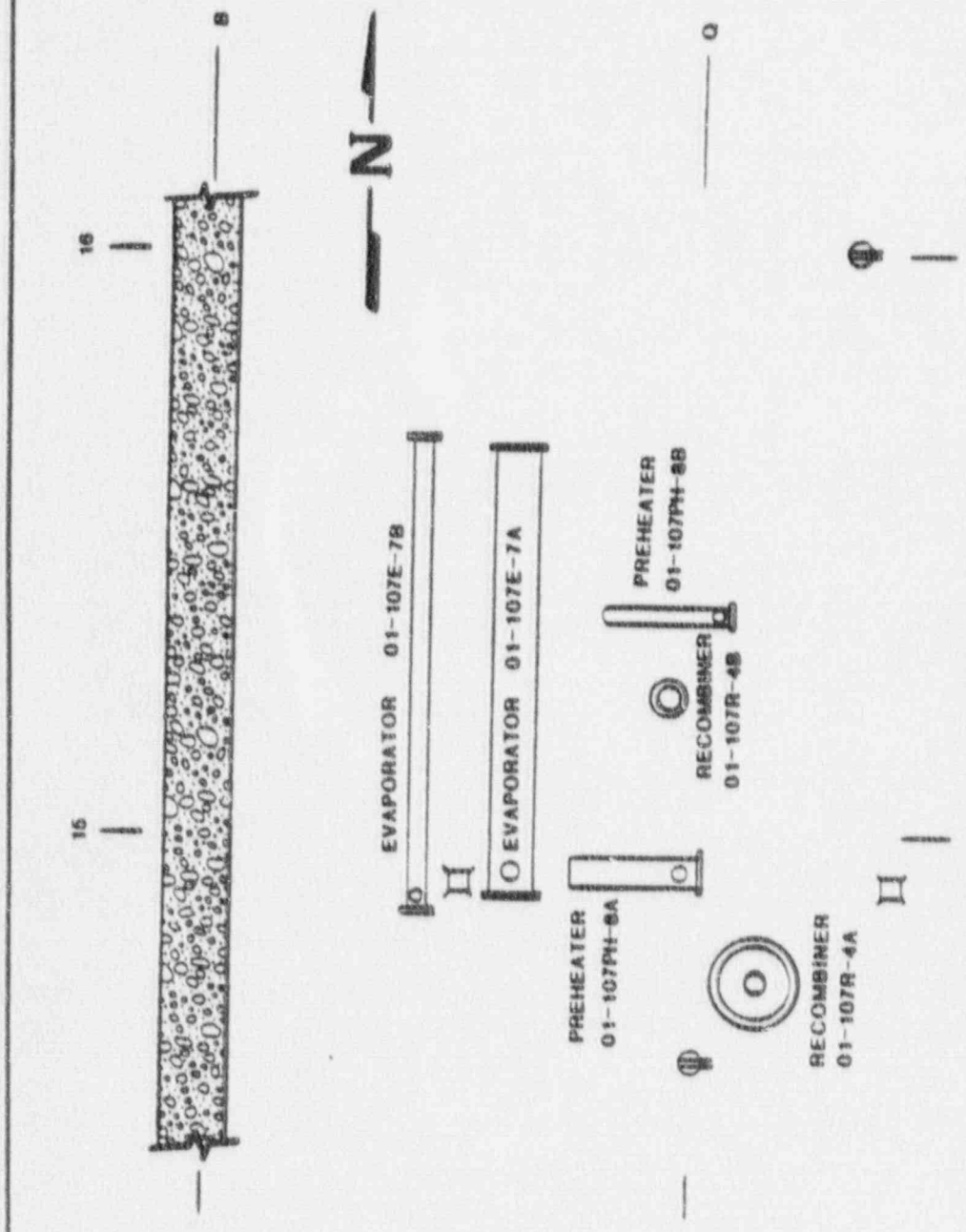
DATE _____

TIME _____

BUILDING / ELEVATION TB 272-6

SYSTEM & COMPONENT I D / ELEVATION _____

DESCRIPTION / COMMENT _____



RADIATION		
1	Normal	0700
2	140 mR/hr	0915
3	1700 "	1000
4	160 "	1130
5	120 "	1200
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☐ AIRBORNE SURVEY

AIR SAMPLER SERIAL NO'S. _____ ; _____

AIR SAMPLE RESULTS: _____ ; _____ ; _____ ; _____ ; _____ ; _____ ; _____

(% EFFECTIVE MPC)

SURVEYOR(S) _____ EXPOSURE _____ (mR) REVIEW _____ (SIGNATURE)

_____ _____ (mR)

(SIGNATURE)(S)

REV 0

(SEE OTHER SIDE FOR INSTRUMENT DATA & SURVEY LOG KEY)

RES SURVEY LOG SHEET

FORM NO SSL-1

☐ RWP NO. _____
☐ RPOP-5 _____
☐ _____

SURVEY NO. _____

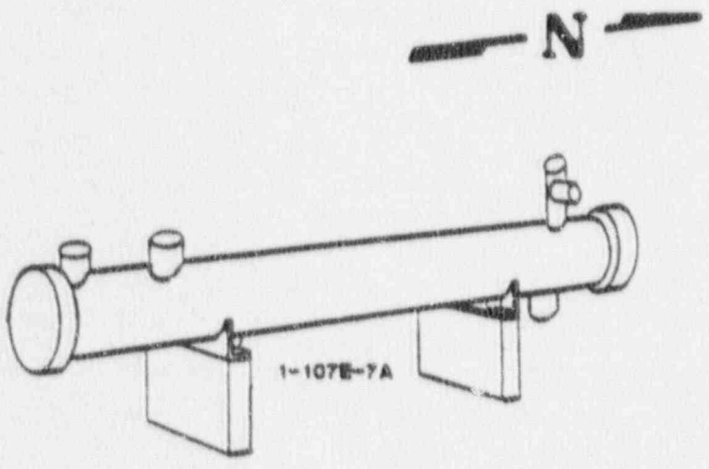
DATE _____

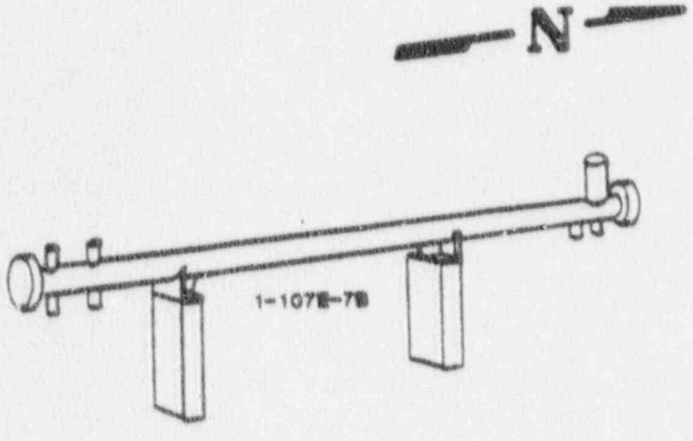
TIME _____

BUILDING / ELEVATION TB 272-7

SYSTEM & COMPONENT I D / ELEVATION _____

DESCRIPTION / COMMENT _____





RADIATION		
1	NBAMA	874
2	140 mR/hr	091
3	1700 "	100
4	160 "	113
5	120 "	124
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CONTAMINATION		
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☐ AIRBORNE SURVEY AIR SAMPLER SERIAL NO'S. _____ ; _____

AIR SAMPLE RESULTS: _____ ; _____ ; _____ ; _____ ; _____ ; _____ ; _____
(% EFFECTIVE MPC)

SURVEYOR(S) _____ EXPOSURE _____ (mR) REVIEW _____
(SIGNATURE)

_____ _____ (mR)
(SIGNATURE)(S)

REV 0

(SEE OTHER SIDE FOR INSTRUMENT DATA & SURVEY LOG KEY

RES SURVEY LOG SHEET

FORM NO SSL-1

☐ RWP NO. _____
☐ RPOP-5 _____
☐ _____

SURVEY NO. _____

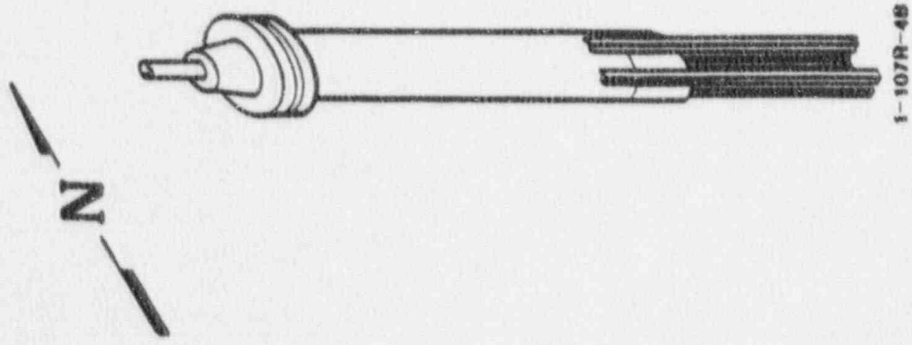
DATE _____

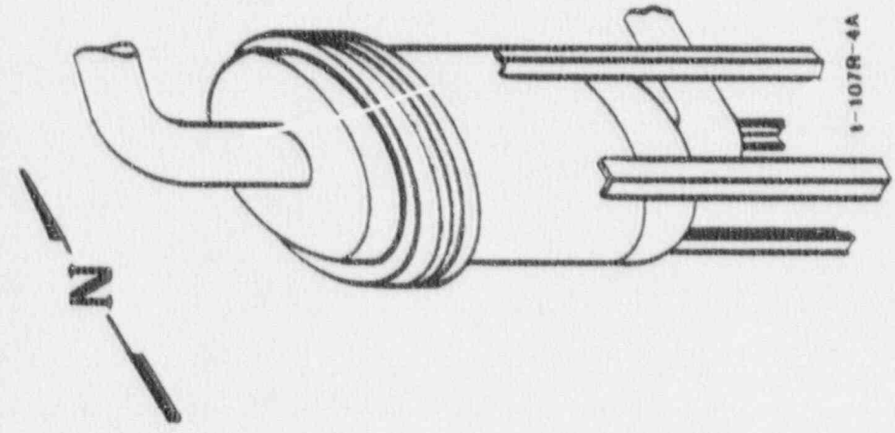
TIME _____

BUILDING / ELEVATION TB 272-8

SYSTEM & COMPONENT I D / ELEVATION _____

DESCRIPTION / COMMENT _____





RADIATION		
1	Normal	07
2	140 mR/hr	091
3	1700 "	100
4	160 "	113
5	120 "	120
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CONTAMINATION		
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☐ AIRBORNE SURVEY AIR SAMPLER SERIAL NO'S. _____; _____

AIR SAMPLE RESULTS: _____; _____; _____; _____; _____; _____; _____
(% EFFECTIVE MPC)

SURVEYOR(S) _____ EXPOSURE _____ (mR) REVIEW _____
(SIGNATURE)

_____ _____ (mR)
(SIGNATURE)(S)

REV 0 (SEE OTHER SIDE FOR INSTRUMENT DATA & SURVEY LOG KE

RES SURVEY LOG SHEET

FORM NO SSL-1

☐ RWP NO. _____
☐ RPOP-5 _____
☐ _____

SURVEY NO. _____

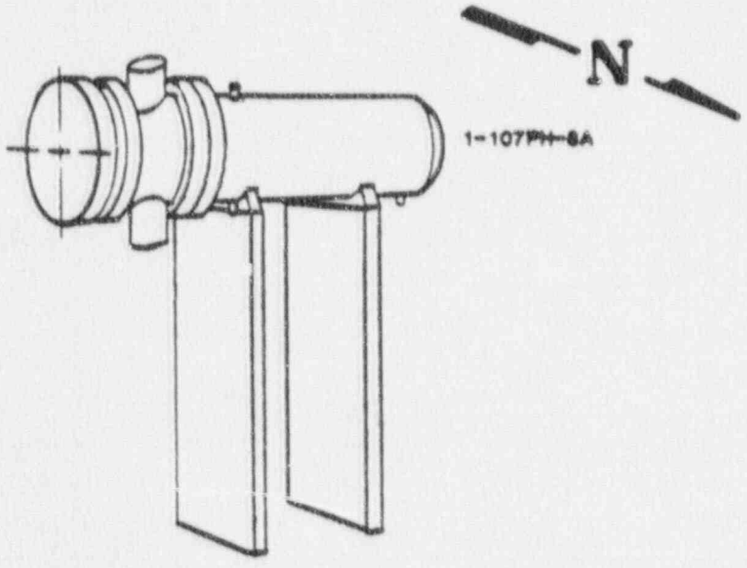
DATE _____

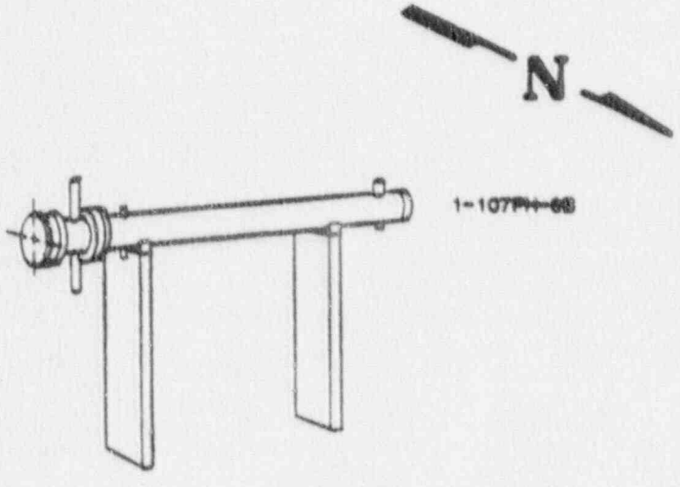
TIME _____

BUILDING / ELEVATION TB 272-9

SYSTEM & COMPONENT I D / ELEVATION _____

DESCRIPTION / COMMENT _____





RADIATION		
1	Normal	67
2	140 MR/HR	691
3	1700 "	100
4	160 "	113
5	120 "	120
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CONTAMINATION		
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☐ AIRBORNE SURVEY AIR SAMPLER SERIAL NO'S. _____ ; _____

AIR SAMPLE RESULTS: _____ ; _____ ; _____ ; _____ ; _____ ; _____ ; _____
(% EFFECTIVE MPC)

SURVEYOR(S) _____ EXPOSURE _____ (mR) REVIEW _____
(SIGNATURE)

_____ _____ (mR)
(SIGNATURE)(S)

REV 0 (SEE OTHER SIDE FOR INSTRUMENT DATA & SURVEY LOG KEY

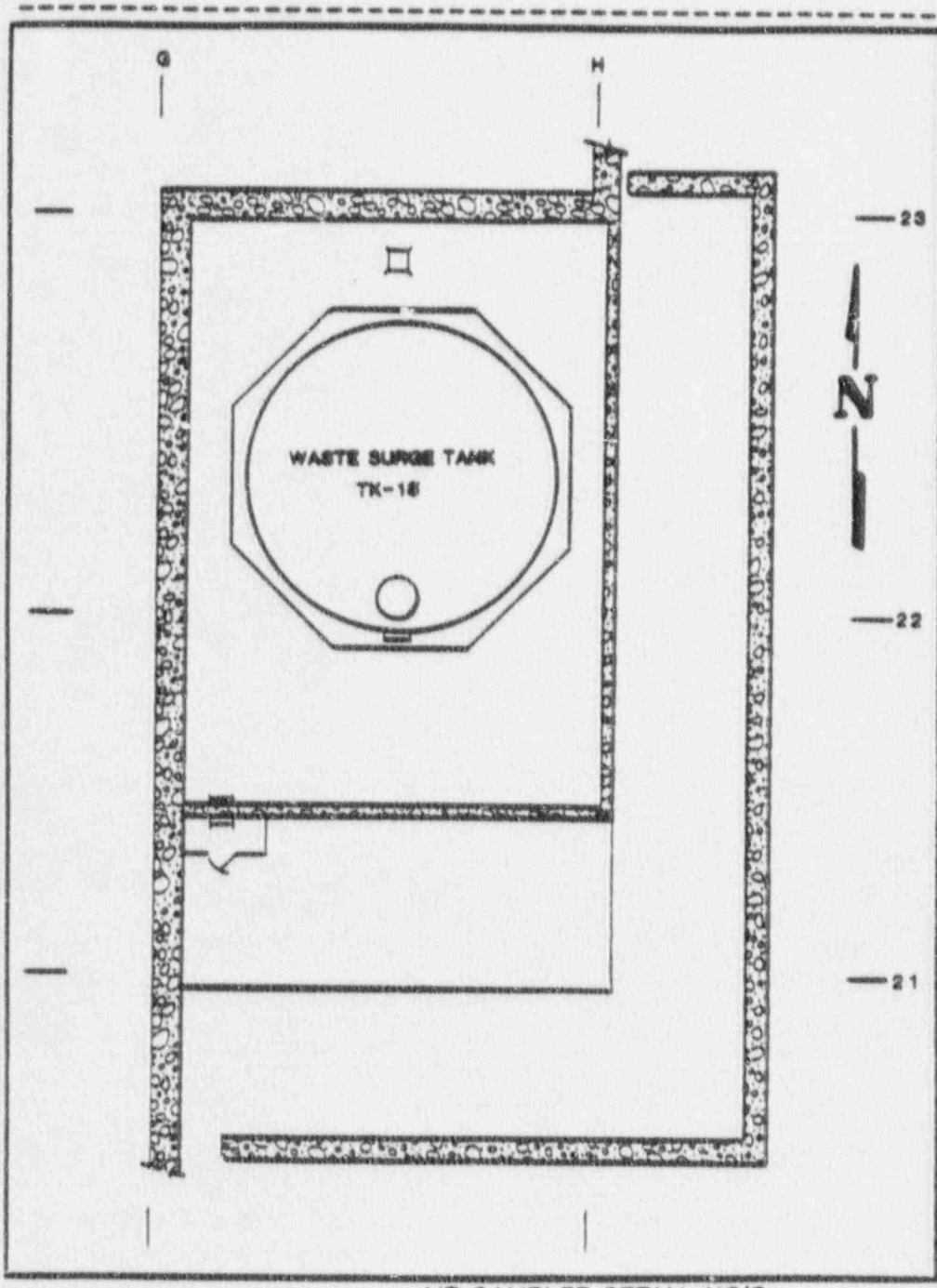
RES SURVEY LOG SHEET

FORM NO SSL-1

☐ RWP NO. _____
☐ RPOP-5 _____
☐ _____

BUILDING / ELEVATION TB 272-10
SYSTEM & COMPONENT I D / ELEVATION _____
DESCRIPTION / COMMENT _____

SURVEY NO. _____
DATE _____
TIME _____



	RADIATION/Tic
1	Normal 070
2	140 mR/hr 0916
3	1200 " 1800
4	160 " 1136
5	120 " 1200
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☐ AIRBORNE SURVEY AIR SAMPLER SERIAL NO'S. _____ ; _____

AIR SAMPLE RESULTS: _____ ; _____ ; _____ ; _____ ; _____ ; _____ ; _____
(% EFFECTIVE MPC)

SURVEYOR(S) _____ EXPOSURE _____ (mR) REVIEW _____
(SIGNATURE)

_____ _____ (mR)
(SIGNATURE)(S)

REV 0 (SEE OTHER SIDE FOR INSTRUMENT DATA & SURVEY LOG KEY

RES SURVEY LOG SHEET

FORM NO SSL-1

☐ RWP NO. ____________
☐ RPOP-5 ____________
☐ ____________

SURVEY NO.

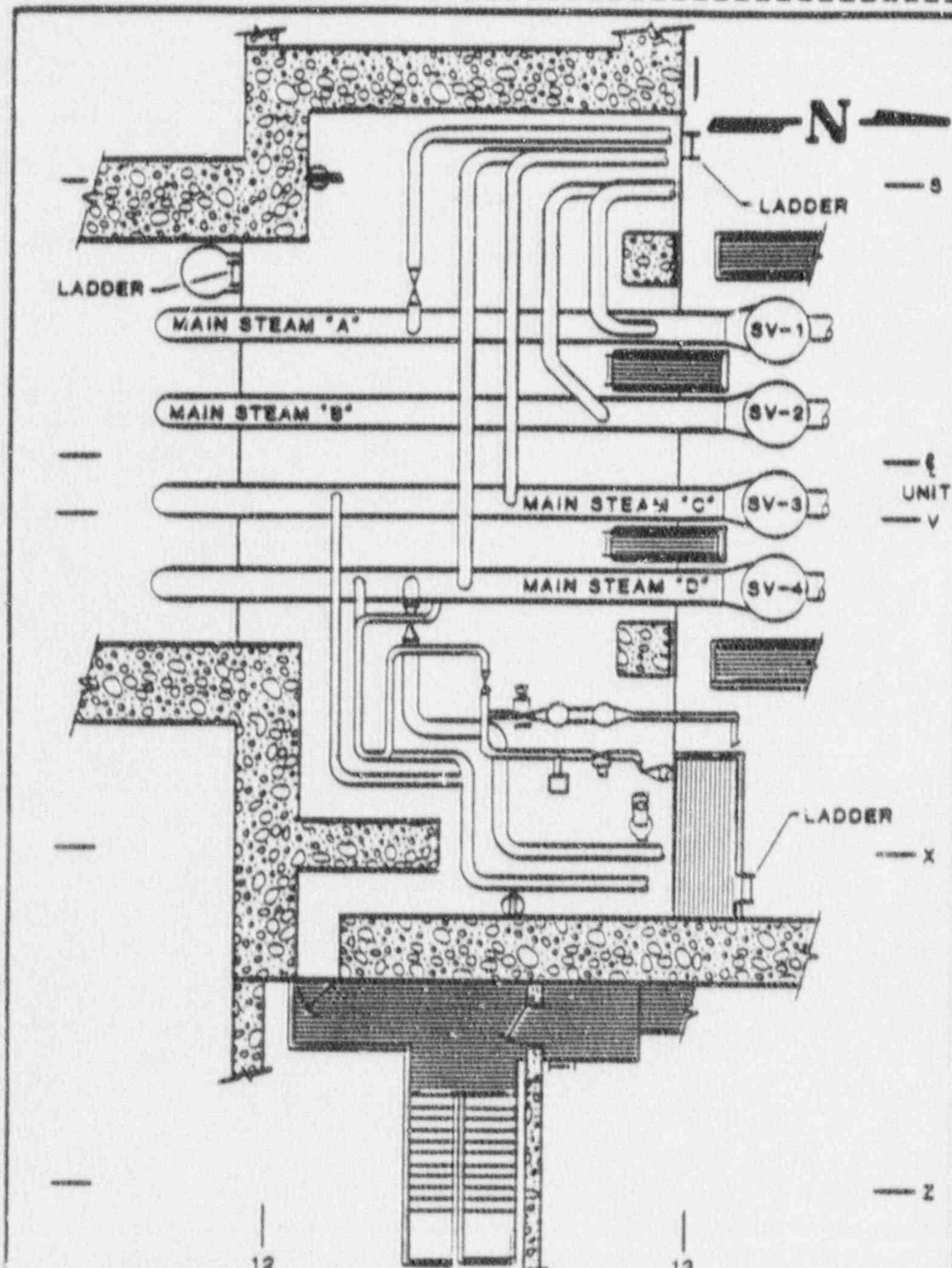
DATE ________

TIME ________

BUILDING / ELEVATION TB 286-1

SYSTEM & COMPONENT I D / ELEVATION ____________

DESCRIPTION / COMMENT ____________



	RADIATION	Time
1	Normal	0700
2	140 mR/hr	0910
3	1700 "	1000
4	160 "	1130
5	120 "	1200
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☐ AIRBORNE SURVEY

AIR SAMPLER SERIAL NO'S. ________ ; ________

AIR SAMPLE RESULTS: _____ ; _____ ; _____ ; _____ ; _____ ; _____ ; _____

(% EFFECTIVE MPC)

SURVEYOR(S) ____________ EXPOSURE ______ (mR) REVIEW ____________

____________ ______ (mR) (SIGNATURE)

(SIGNATURE)(S)

REV 0 (SEE OTHER SIDE FOR INSTRUMENT DATA & SURVEY LOG KEY)

RES SURVEY LOG SHEET

FORM NO SSL-1

☐ RWP NO. _____
☐ RPOP-5 _____
☐ _____

SURVEY NO. _____

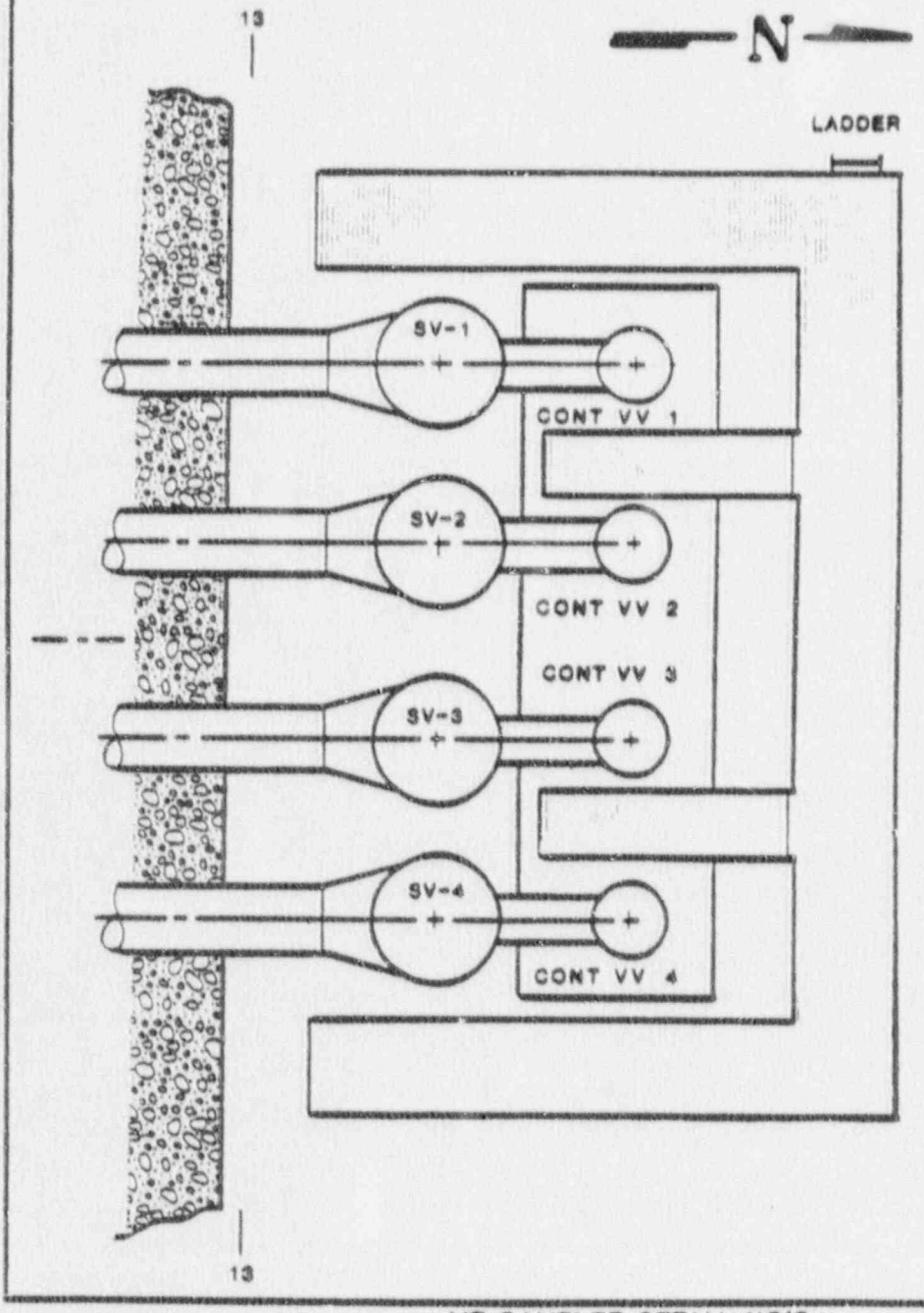
DATE _____

TIME _____

BUILDING / ELEVATION TB 288-1

SYSTEM & COMPONENT I D / ELEVATION _____

DESCRIPTION / COMMENT _____



RADIATION/T.		
1	Normal	82
2	140 mR/hr	091
3	1200 "	106
4	160 "	113
5	120 "	126
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CONTAMINATION	
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☐ AIRBORNE SURVEY

AIR SAMPLER SERIAL NO'S. _____ ; _____

AIR SAMPLE RESULTS: _____ ; _____ ; _____ ; _____ ; _____ ; _____ ; _____

(% EFFECTIVE MPC)

SURVEYOR(S) _____ EXPOSURE _____ (mR) REVIEW _____

(SIGNATURE)

_____ _____ (mR)

(SIGNATURE)(S)

REV 0

(SEE OTHER SIDE FOR INSTRUMENT DATA & SURVEY LOG KEY

RES SURVEY LOG SHEET

FORM NO SSL-1

☐ RWP NO. _____
☐ RPOP-5 _____
☐ _____

SURVEY NO. _____

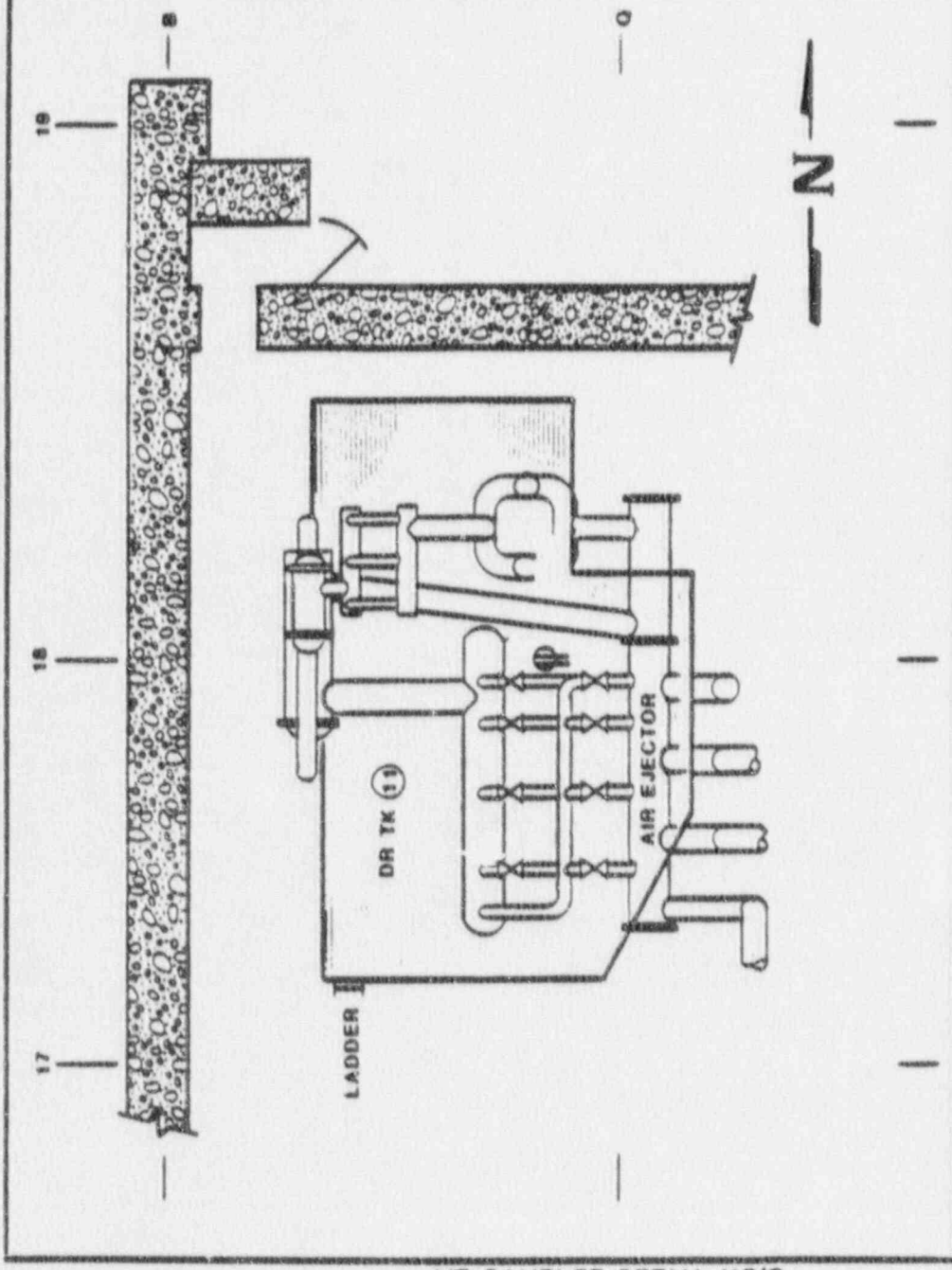
DATE _____

TIME _____

BUILDING / ELEVATION TB 288-2

SYSTEM & COMPONENT I D / ELEVATION _____

DESCRIPTION / COMMENT _____



RADIATION		
1	Normal	0700
2	140 mR/hr	0910
3	1200 "	1045
4	160 "	1134
5	130 "	1200
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☐ AIRBORNE SURVEY

AIR SAMPLER SERIAL NO'S. _____ ; _____

AIR SAMPLE RESULTS: _____ ; _____ ; _____ ; _____ ; _____ ; _____ ; _____

(% EFFECTIVE MPC)

SURVEYOR(S) _____ EXPOSURE _____ (mR) REVIEW _____ (SIGNATURE)

_____ _____ (mR)

(SIGNATURE)(S)

REV 0

(SEE OTHER SIDE FOR INSTRUMENT DATA & SURVEY LOG KEY

RES SURVEY LOG SHEET

FORM NO SSL-1

☐ RWP NO. _____
☐ RPOP-5 _____
☐ _____

BUILDING / ELEVATION TB 288-3

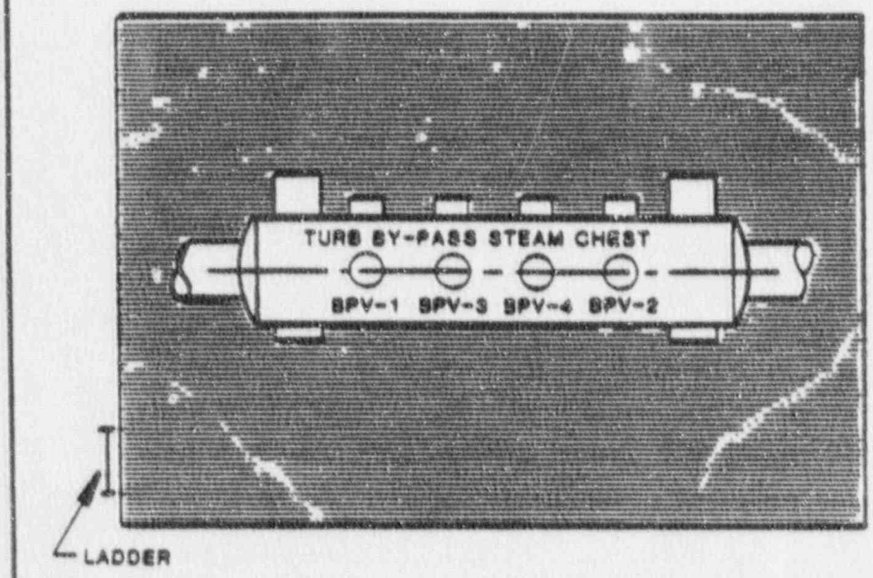
SYSTEM & COMPONENT I D / ELEVATION _____

DESCRIPTION / COMMENT _____

SURVEY NO. _____

DATE _____

TIME _____



RADIATION		Time
1	NORMAL	0700
2	140 mR/hr	0910
3	170 "	1000
4	160 "	1130
5	120 "	1200
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CONTAMINATION	
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☐ AIRBORNE SURVEY AIR SAMPLER SERIAL NO'S. _____; _____

AIR SAMPLE RESULTS: _____; _____; _____; _____; _____; _____; _____

(% EFFECTIVE MPC)

SURVEYOR(S) _____ EXPOSURE _____ (mR) REVIEW _____ (SIGNATURE)

_____ _____ (mR)

(SIGNATURE)(S)

REV 0

(SEE OTHER SIDE FOR INSTRUMENT DATA & SURVEY LOG KEY)

RES SURVEY LOG SHEET

FORM NO 3SL-1

☐ RWP NO. _____
☐ RPOP-5 _____
☐ _____

SURVEY NO. _____

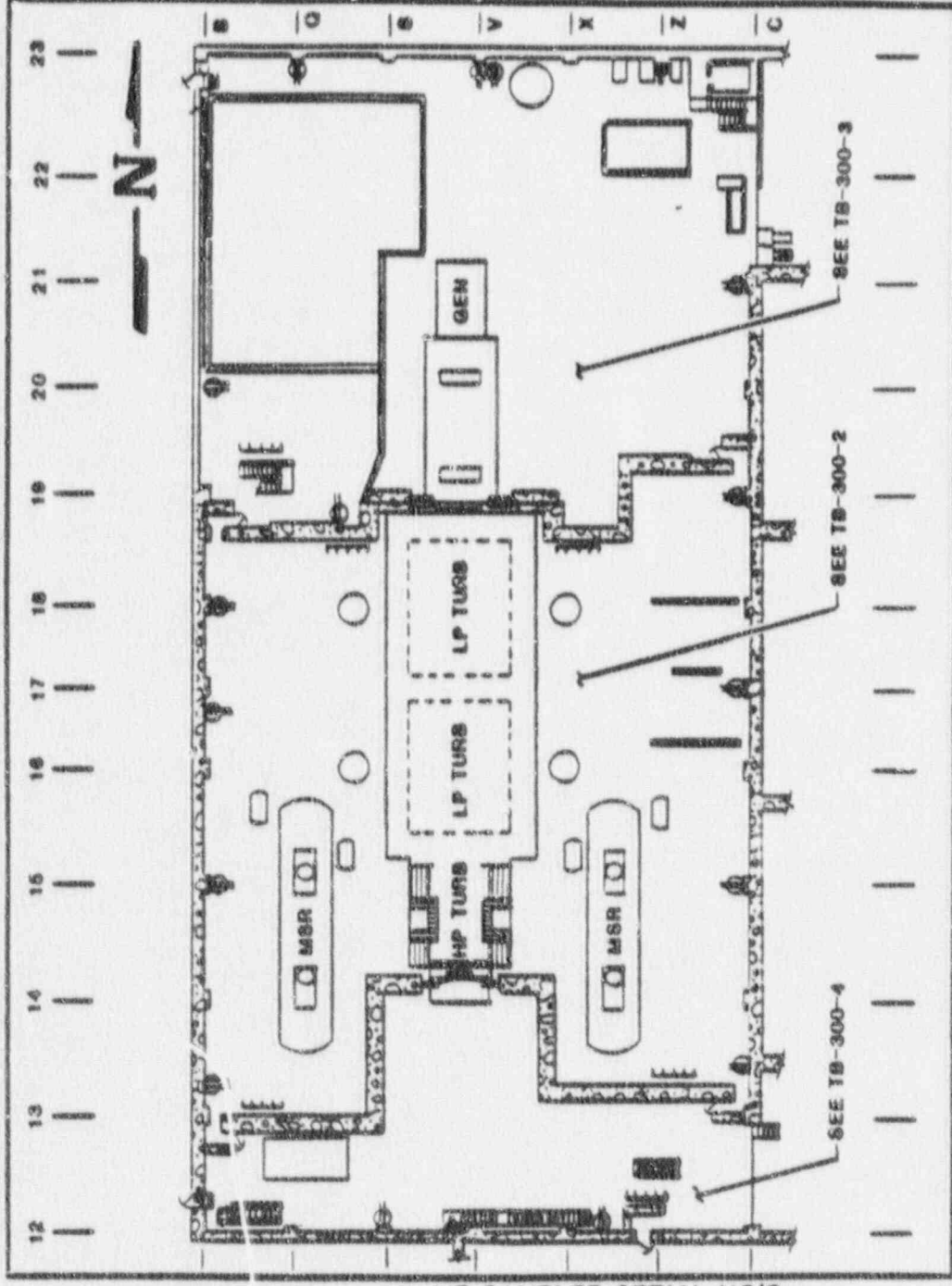
DATE _____

TIME _____

BUILDING / ELEVATION TB 300-1

SYSTEM & COMPONENT I D / ELEVATION _____

DESCRIPTION / COMMENT _____



RADIATION		
1	NORMAL	0700
2	145 mR/hr	0916
3	1670 "	1000
4	195 "	1130
5	150 "	1200
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CONTAMINATION		
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☐ AIRBORNE SURVEY

AIR SAMPLER SERIAL NO'S. _____ ; _____

AIR SAMPLE RESULTS: _____ ; _____ ; _____ ; _____ ; _____ ; _____ ; _____

(% EFFECTIVE MPC)

SURVEYOR(S) _____ EXPOSURE _____ (mR) REVIEW _____

(SIGNATURE)

_____ _____ (mR)

(SIGNATURE)(S)

REV 0

(SEE OTHER SIDE FOR INSTRUMENT DATA & SURVEY LOG KEY)

RES SURVEY LOG SHEET

FORM NO SSL-1

☐ RWP NO. _____
☐ RPOP-5 _____
☐ _____

SURVEY NO. _____

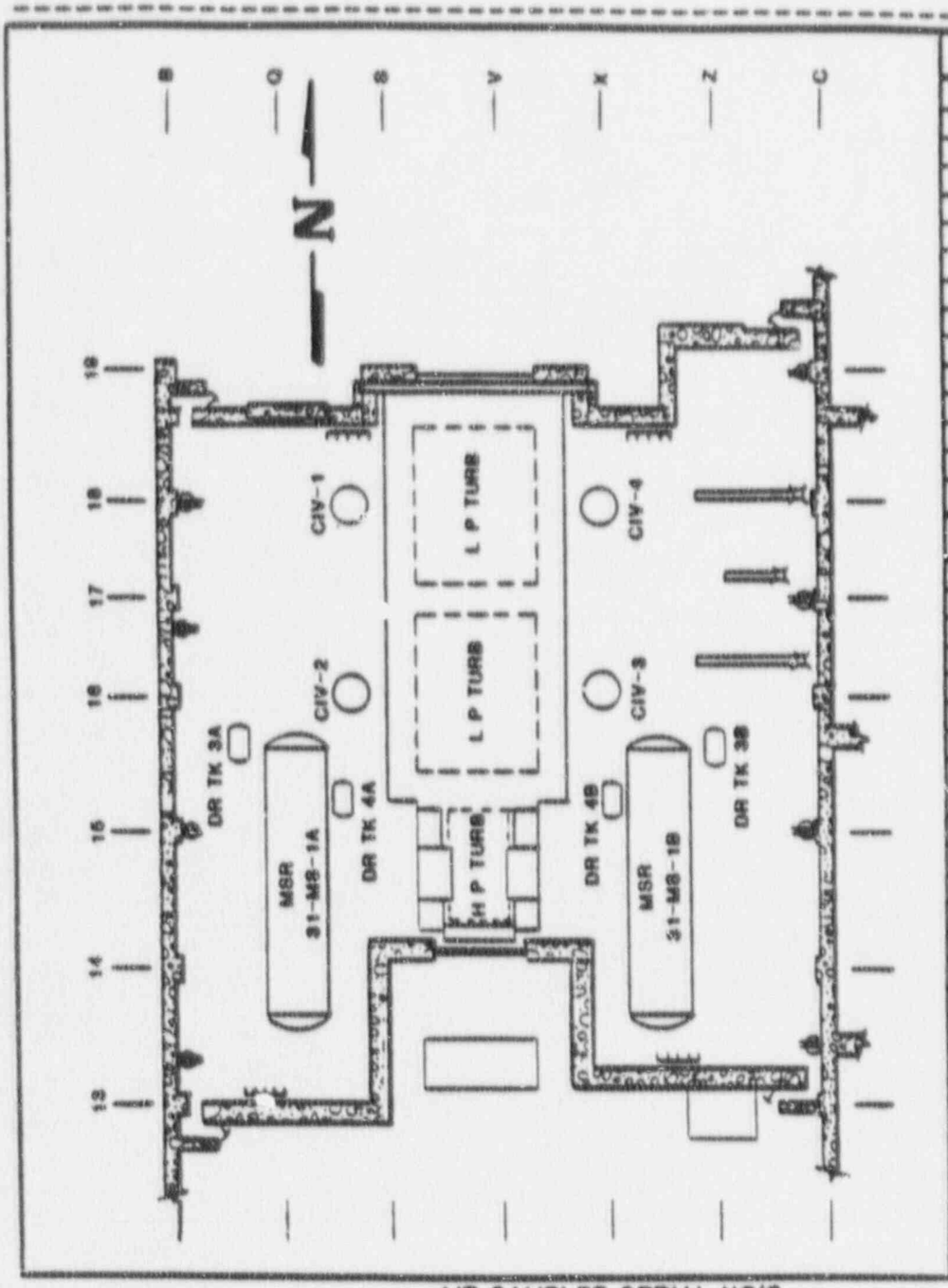
DATE _____

TIME _____

BUILDING / ELEVATION TB 300-2

SYSTEM & COMPONENT I D / ELEVATION _____

DESCRIPTION / COMMENT _____



RADIATION/T.		
1	Alpha/	0.70
2	110 mR/hr	0.91
3	1400 "	1.04
4	160 "	1.30
5	110 "	1.20
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☐ AIRBORNE SURVEY AIR SAMPLER SERIAL NO'S. _____ ; _____

AIR SAMPLE RESULTS: _____ ; _____ ; _____ ; _____ ; _____ ; _____ ; _____

(% EFFECTIVE MPC)

SURVEYOR(S) _____ EXPOSURE _____ (mR) REVIEW _____ (SIGNATURE)

_____ _____ (mR)

(SIGNATURE)(S)

REV 0

(SEE OTHER SIDE FOR INSTRUMENT DATA & SURVEY LOG KEY)

RES SURVEY LOG SHEET

FORM NO SSL-1

☐ RWP NO. _____
☐ RPOP-5 _____
☐ _____

SURVEY NO. _____

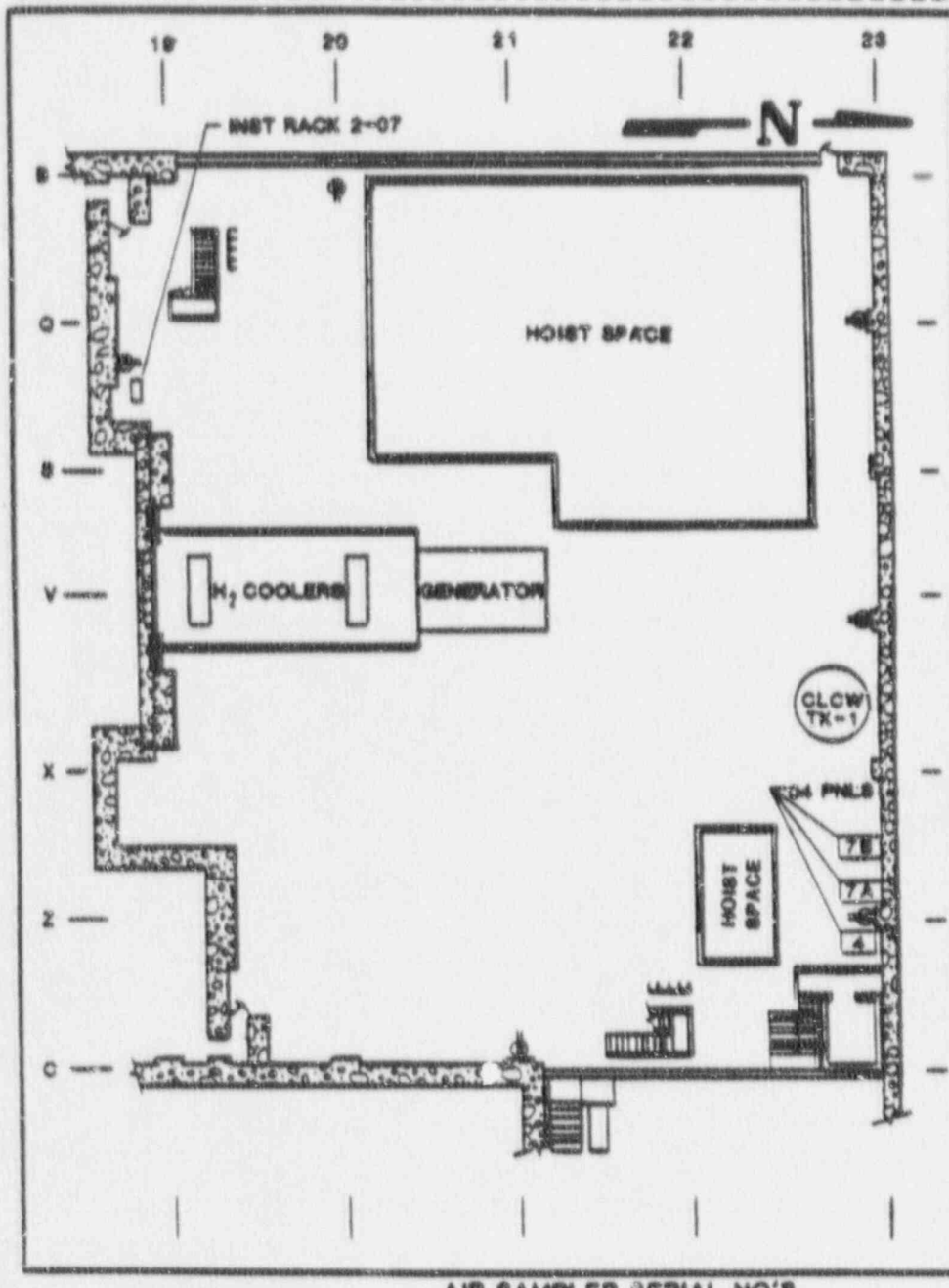
DATE _____

TIME _____

BUILDING / ELEVATION TB 300-3

SYSTEM & COMPONENT I D / ELEVATION _____

DESCRIPTION / COMMENT _____



RADIATION		
1	Normal	076
2	145 mR/hr	091
3	1670 "	100
4	195 "	113
5	150 "	121
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CONTAMINATION	
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☐ AIRBORNE SURVEY AIR SAMPLER SERIAL NO'S. _____ ; _____

AIR SAMPLE RESULTS: _____ ; _____ ; _____ ; _____ ; _____ ; _____ ; _____
(% EFFECTIVE MPC)

SURVEYOR(S) _____ EXPOSURE _____ (mR) REVIEW _____
(SIGNATURE)

_____ _____ (mR)
(SIGNATURE)(S)

REV 0

(SEE OTHER SIDE FOR INSTRUMENT DATA & SURVEY LOG KE)

RES SURVEY LOG SHEET

FORM NO SSL-1

☐ RWP NO. _____
☐ RPOP-5 _____
☐ _____

SURVEY NO. _____

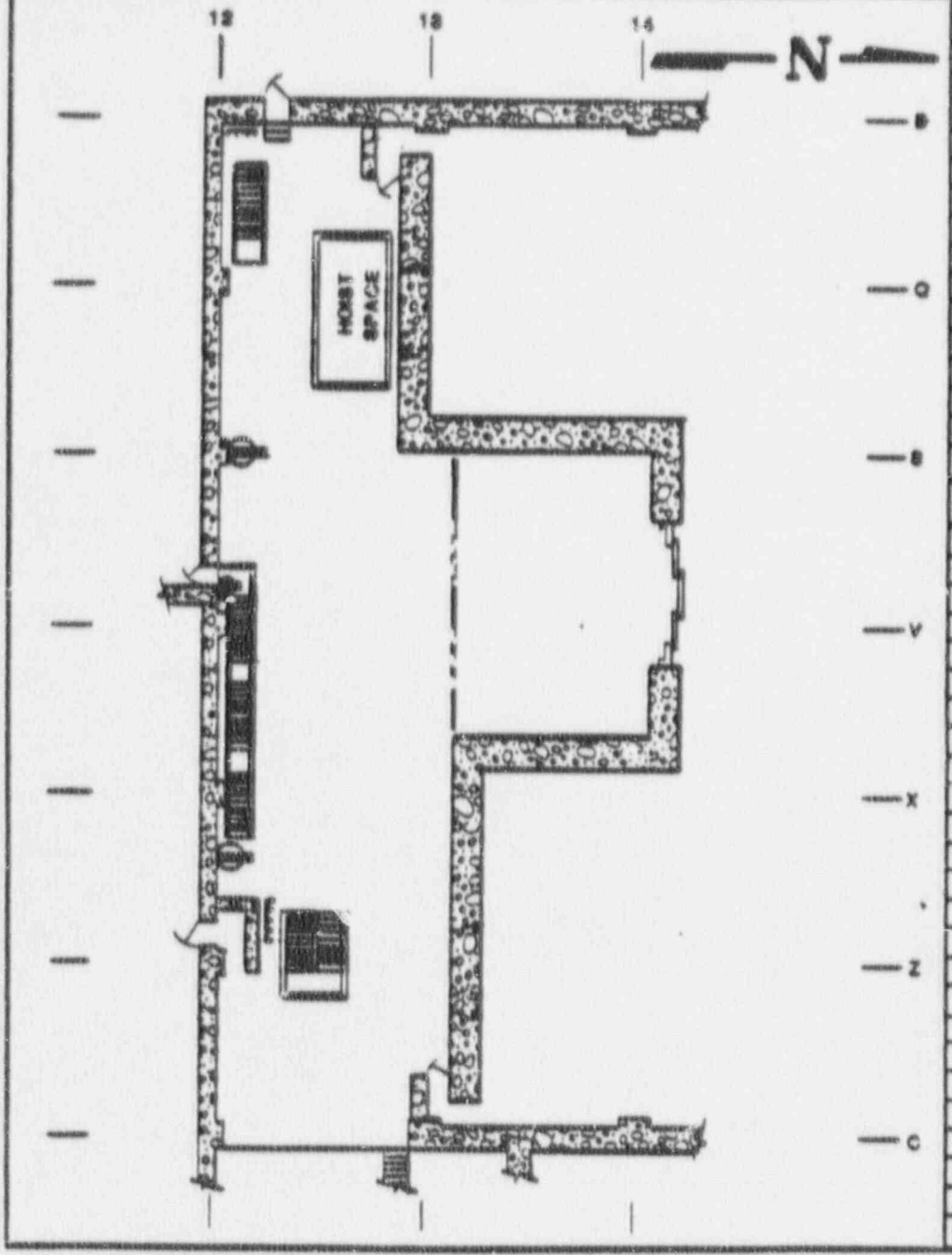
DATE _____

TIME _____

BUILDING / ELEVATION TB 300-4

SYSTEM & COMPONENT I D / ELEVATION _____

DESCRIPTION / COMMENT _____



RADIATION		
1	Normal	0
2	145 mR/HR	09
3	1670 "	10
4	125 "	11
5	150 "	12
6		
7		
8		
9		
10		
11		
12		
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15		

CONTAMINATION	
1	
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☐ AIRBORNE SURVEY

AIR SAMPLER SERIAL NO'S. _____ ; _____

AIR SAMPLE RESULTS: _____ ; _____ ; _____ ; _____ ; _____ ; _____ ; _____
(% EFFECTIVE MPC)

SURVEYOR(S) _____ EXPOSURE _____ (mR) REVIEW _____
(SIGNATURE)

_____ _____ (mR)
(SIGNATURE)(S)

REV 0

(SEE OTHER SIDE FOR INSTRUMENT DATA & SURVEY LOG KE

SECTION 7

James A. FitzPatrick Nuclear Power Plant
1990 Partial Scale Emergency Exercise

December 19, 1990

Chemistry Data

Table 1

SMALL VOLUME
REACTOR WATER RADIATION LEVELS (1)

Time		Dose Rates (mR/hr)
		Normal Sink
Entire Drill	Contact:	30
	@ 1 ft.:	5
	@ 2 ft.:	2
	@ 3 ft.:	1

(1) Data to be provided to Sample Team Member if the sink is operated.

Table 2

REACTOR COOLANT ACTIVITY DATA (μCi/g)

(Entire Exercise)

I-131	4.8E-2
I-132	8.5E-3
I-133	7.1E-3
I-134	1.1E-2
I-135	1.2E-2
Kr-83m	8.2E-1
Kr-85	1.7E-1
Kr-85m	3.1E+0
Kr-87	1.2E+0
Kr-88	8.4E+0
Xe-131	1.1E-3
Xe-133	3.1E-1
Xe-133m	1.1E-2
Xe-135	6.1E-2
Xe-135m	4.2E-2
Xe-138	1.2E-3
Cs-137	3.1E-2

SECTION 7

James A. FitzPatrick Nuclear Power Plant
1990 Partial Scale Emergency Exercise

December 19, 1990

Offsite Radiological Data

The following Turbine Building Vent Radiation Monitor readings and dose rate maps are based on a simulated atmospheric release of approximately 4 Ci/sec out the Turbine Building vent.

On the day of the exercise, actual meteorology will be used. An example of a similar release under real time meteorology can be seen in Section 3 of this scenario.

SECTION 7

TURBINE BUILDING
VENT RADIATION MONITORS

Time	Vent Flow	Monitor Reading Low Range	High Range
0700	70,000 cfm	70 cpm	0 mR/hr
0715	70,000 cfm	70 cpm	0 mR/hr
0730	70,000 cfm	70 cpm	0 mR/hr
0745	70,000 cfm	70 cpm	0 mR/hr
0800	70,000 cfm	70 cpm	0 mR/hr
0815	0	70 cpm	0 mR/hr
0830	0	70 cpm	0 mR/hr
0845	0	70 cpm	0.1 mR/hr
0900	0	70 cpm	0.2 mR/hr
0915	0	70 cpm	0.25 mR/hr
0930	70,000 cfm*	Offscale	0.27 mR/hr
0945	70,000 cfm	Offscale	0.26 mR/hr
1000	70,000 cfm	Offscale	0.27 mR/hr
1015	70,000 cfm	Offscale	0.27 mR/hr
1030	70,000 cfm	Offscale	0.25 mR/hr
1045	70,000 cfm	Offscale	0.26 mR/hr
1100	70,000 cfm	Offscale	0.26 mR/hr
1115	70,000 cfm	Offscale	0.23 mR/hr
1130	70,000 cfm	Offscale	0.20 mR/hr
1145	70,000 cfm	600,000 cpm	0.1 mR/hr
1200	70,000 cfm	50,000 cpm	0 mR/hr

*Assuming building ventilation unisolated and exhaust fans started.

SECTION 7

OFFSITE
AIR SAMPLE DATA

Conc. (μCi/cc)
for Various Dose Rate Readings (mR/hr)

NUCLIDE	1-10	11-20	21-50	51-100	101-200
Kr-85	8.4E-09	8.4E-08	2.5E-07	4.0E-07	1.2E-06
Kr-85m	8.4E-08	8.4E-07	2.5E-06	4.0E-06	1.2E-05
Kr-87	1.5E-07	1.5E-06	4.3E-06	7.0E-06	2.1E-05
Kr-88	2.1E-07	2.1E-06	6.2E-06	1.0E-05	3.0E-05
Xe-131m	2.5E-07	2.5E-06	7.4E-06	1.2E-05	3.0E-05
Xe-133	7.8E-07	7.8E-06	2.3E-08	3.7E-05	3.6E-05
Xe-133m	1.1E-07	1.0E-06	3.1E-09	5.0E-06	1.1E-04
Xe-135	8.4E-08	8.4E-07	2.5E-09	4.0E-09	1.5E-05
Xe-135m	1.3E-07	1.3E-06	3.7E-09	6.0E-09	1.3E-05
I-131	8.4E-11	8.4E-10	2.5E-09	4.0E-09	1.8E-08
I-133	1.2E-10	1.2E-09	3.5E-09	5.7E-09	1.7E-08
I-135	7.1E-11	7.1E-10	2.1E-09	3.4E-09	1.0E-08
TOT. CONC.	1.8E-06	1.8E-05	5.3E-05	8.5E-5	2.6E-04

Part. Count Rate (cpm)	0	5	15	20	75
I Count Rate (cpm)	5	45	135	210	750

NOTE: Information on this sheet is for controller use only. This page will not be handed to drill players.

Deposition increase due to precipitation.
1 - Light rain, drizzle or snow flurries = no change in data
2 - Moderate rain or snow = multiply data by 1.1
3 - Heavy rain or snow = multiply data by 1.3

Reference: Slade, D.H. "Meteorology and Atomic Energy," 1968.

SECTION 7

RELEASE DOSE RATES

Time	Red(A)	Green(B)	Blue(C)
0930 to 1130	28	2.8	1

Doses are OPEN WINDOW, edge of Isopleth

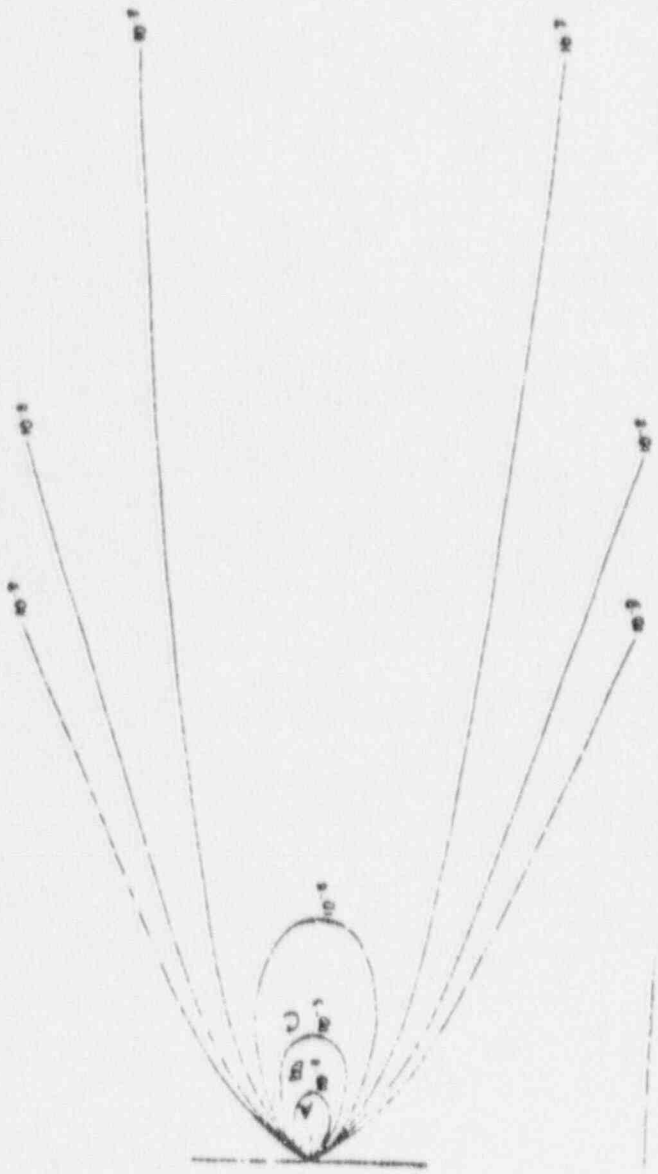
Linear interpretation to be used for areas enclosed in Isopleth.

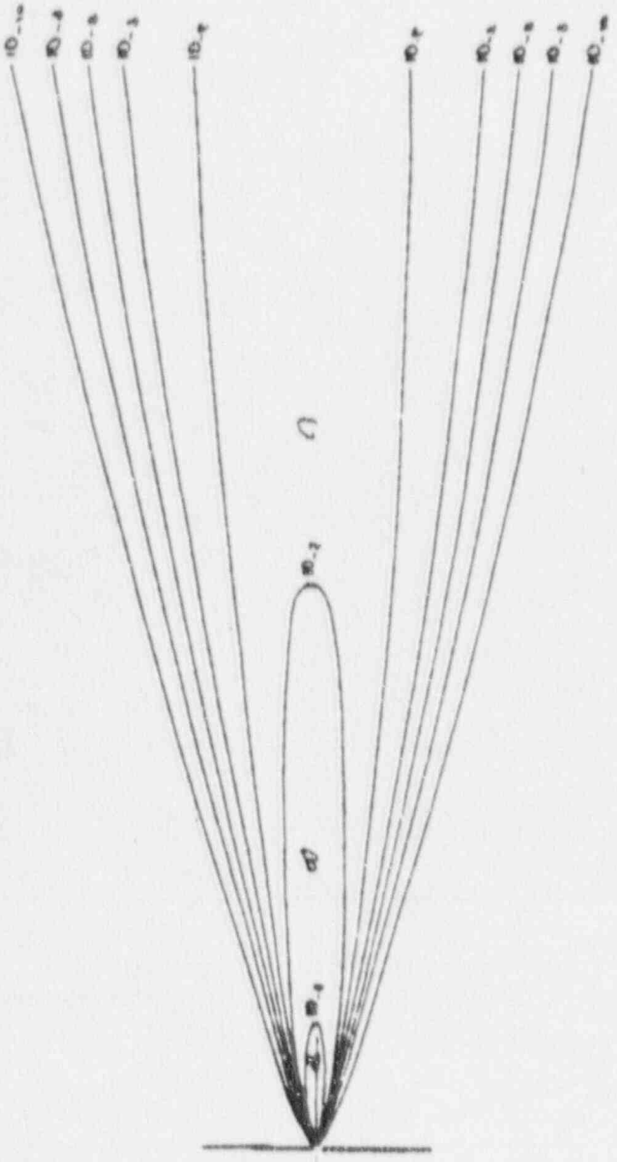
CLOSED WINDOW = 1/2 OPEN WINDOW

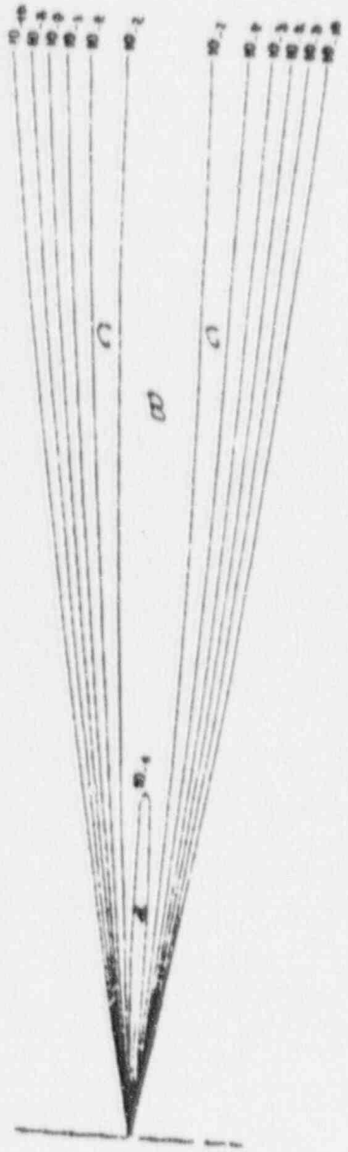
PLUME ARRIVAL TIMES

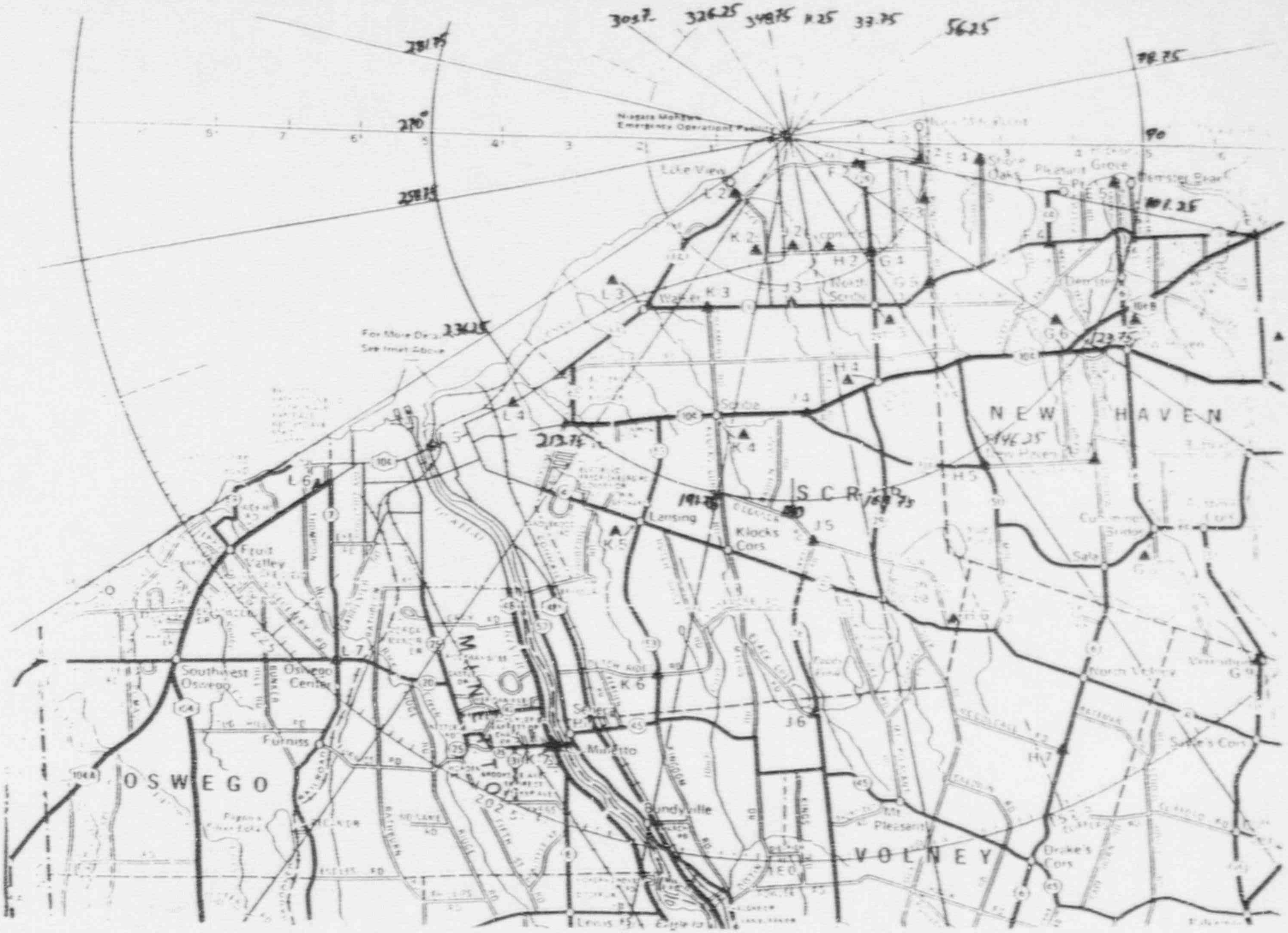
WIND SPEED* M/S	WIND SPEED* MPH	TIME 15	30	45	60	75	90	105	120	135	150	165	180	195	210	225	240	Minutes
0.5	1.0	0.25	0.5	0.75	1.0	1.25	1.5	1.75	2.0	2.25	2.5	2.75	3.0	3.25	3.5	3.75	4	
1.1	2.5	0.6	1.3	1.9	2.5	3.1	3.8	4.4	5.0	5.6	6.3	6.9	7.5	8.1	8.8	9.4	10	
2.3	5.0	1.3	2.5	3.8	5.0	6.3	7.5	8.8	10	>10								Distance Downwind (miles)
3.4	7.5	1.9	3.8	5.6	7.5	9.4	>10											
4.5	10.0	2.5	5.0	7.5	10	>10												
5.7	12.5	3.1	6.3	9.4	>10													
6.8	15.0	3.8	7.5	>10														
8.0	17.5	4.4	8.8	>10														
9.1	20.0	5.0	10	>10														
10.2	22.5	5.6	>10															
11.4	25.0	6.3	>10															
12.5	27.5	6.4	>10															
13.6	30.0	7.5	>10															

*Intermediate wind speeds to be interpolated

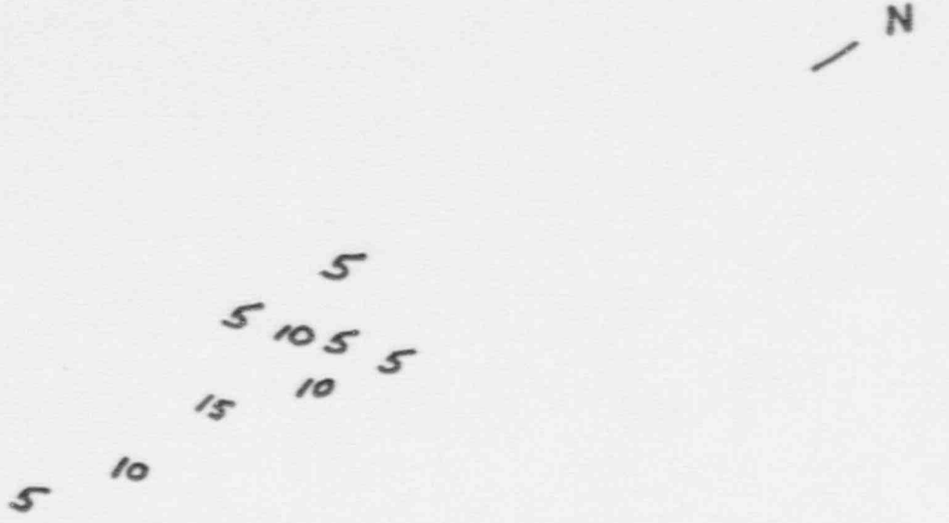





C
C
B

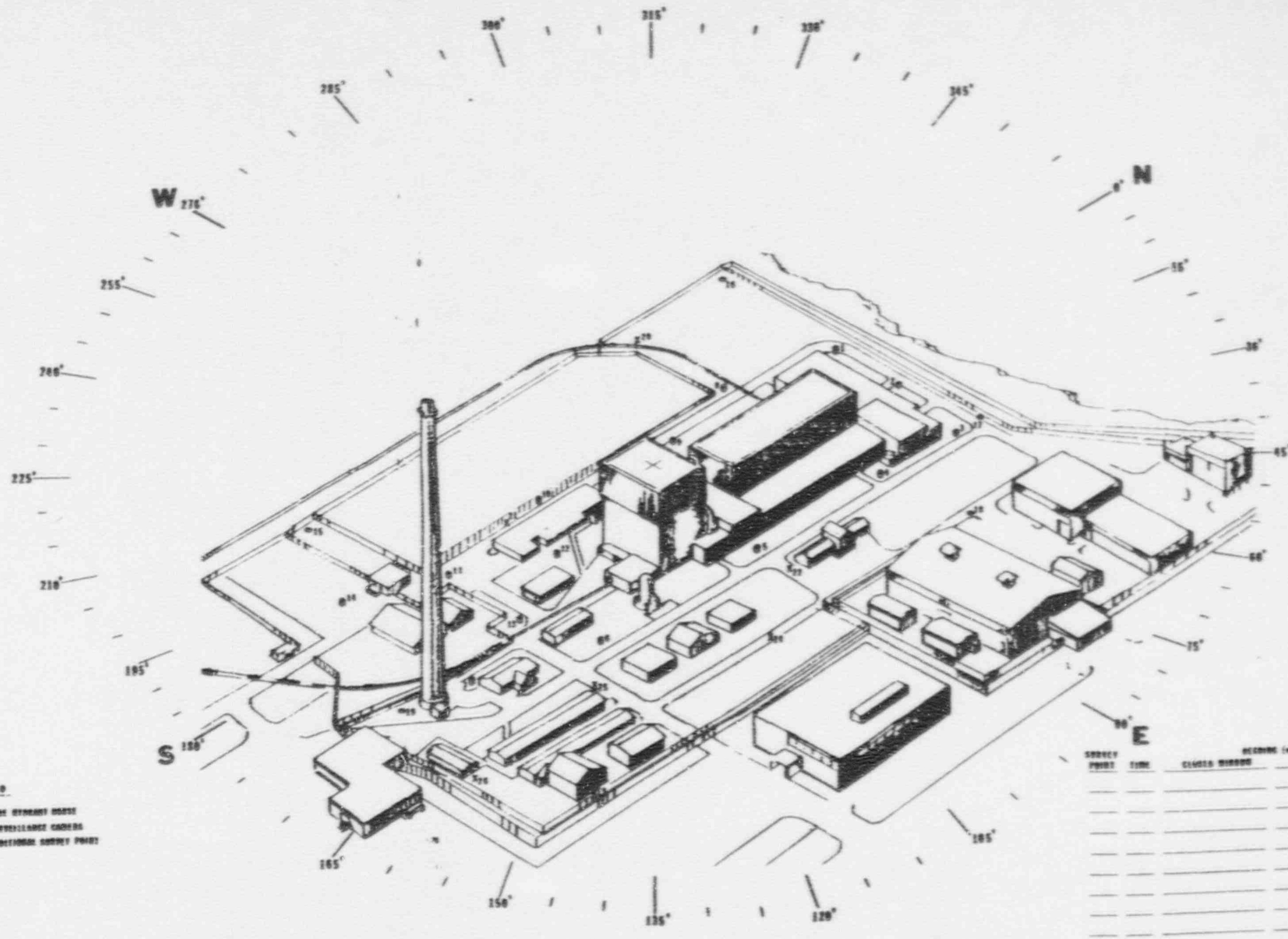


303.75
326.25
348.75
11.25
33.75
56.25
78.75
281.75
270°
258.75
236.25
213.75
191.25
168.75
146.25
123.75
101.25
90
Niagara Mohawk Emergency Operations Facility
For More Detail See Inset Above
NEW HAVEN
SCRIBA
MINETTO
OSWEGO
VOLNEY
Lake View
Scriba
Lansing
Klocks Cors.
Fruit Valley
Southwest Oswego
Oswego Center
Furniss
Minetto
Bundyville
Mt. Pleasant
Drake's Cors
North Volney
New Haven
Sala
Pleasant Grove
Demster Beach
Stone Oaks
North Scriba



After 0930, all readings
are mR/hr and open window.

closed window = ½ open window



LEGEND

ⓕ FIRE HYDRANT HOUSE

→ SURVEILLANCE CAMERA

† ADDITIONAL SURVEY POINT

SURVEY POINT	TIME	READING (mR/hr) CLOSED WINDOW	OPEN WINDOW

EMERGENCY PLANNING MAP NO. 1

SECTION 8

James A. FitzPatrick Nuclear Power Plant
1990 Partial Scale Emergency Exercise

December 19, 1990

ANTICIPATED ACTIONS

I. Control Room

II. Technical Support Center

III. Operational Support Center

IV. Emergency Operations Facility

Event	Time	Control Room	Tech. Support Ctr.	Oper. Support Ctr.	Emerg. Ops. Fac.
1. Stuck control rod	0700 to 0730	1. Actions in accordance with AOP-24. 2. Recognize 24-hour LCO requirement. 3. Classify UNUSUAL EVENT.	None	None	None
2. Loss of 115KV reserve power	0730 to 0930	1. Contact Niagara Mohawk and request emergency restoration of power. 2. Actions in accordance with AOP's.	None	None	None

Event	Time	Control Room	Tech. Support Ctr.	Oper. Support Ctr.	Emerg. Ops. Fac.
3. Scram with break in Main Steam Line Drain Pipe	0800	1. Actions in accordance with EOP's and AOP's to stabilize plant. 2. Classify SITE AREA EMERGENCY based on steam line break. 3. Notify offsite agencies.	1. Determine source of break and rate of leakage. 2. Determine appropriate mitigating actions.	1. Work with engineers on implementing corrective actions.	1. Perform dose projections. 2. Recommend protective actions. 3. Relieve TSC of offsite communications.
4. Hydrogen Explosion	0930	1. Actions in accordance with EOP's and AOP's to stabilize plant.	1. Assess potential damage due to explosion. 2. Continue efforts to mitigate accident consequences.	1. Work with engineers on implementing corrective actions.	1. Perform dose projections. 2. Recommend protective actions. 3. Relieve TSC of offsite communications.

SECTION 9

James A. FitzPatrick Nuclear Power Plant
1990 Partial Scale Emergency Exercise

December 19, 1990

METHOD OF EVALUATION

I. SAP-6 "Drill/Exercise Conduct"

NEW YORK POWER AUTHORITY
JAMES A. FITZPATRICK NUCLEAR POWER PLANT
EMERGENCY PLAN IMPLEMENTING PROCEDURE

EMERGENCY PLAN VOLUME 3

PROCEDURE NO.: SAP-6

TITLE: DRILL/EXERCISE CONDUCT*

PORC REVIEW NO.: Meeting No. 89-48 Date 8-16-89

APPROVED BY: _____
Resident Manager

APPROVED BY: _____
Emergency Planning Coordinator

PAGE NO.	1	2	3	4	5	6	7	8	9	10
REV. NO.	3	0	4	3	4	0	3	0	0	4
PAGE NO.	11	12	13	14	15	16	17	18	19	20
REV. NO.	4	0	4	4	4	4	4	4	4	4
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PAGE NO.	61	62	63	64	65	66	67	68	69	70
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PAGE NO.	71	72	73	74	75	76				
REV. NO.	4	4	4	4	4	4				

Rev. No. 4

Date: 08/89

EMERGENCY PLAN IMPLEMENTING PROCEDURE

SAP-6

DRILL/EXERCISE CONDUCT*

1.0 PURPOSE

To establish a procedure for the conduct and evaluation of all Emergency Plan Drills and Exercises at JAFNPP. This procedure also outlines the management controls used to ensure that corrective actions are implemented.

2.0 REFERENCES

2.1 NUREG-0654 - "Criteria for the Preparation and Evaluation of Radiological Emergency Response Plans and Preparedness in Support of Nuclear Power Plants".

2.2 JAFNPP Emergency Plan and Implementing Procedures, Volumes 1, 2 and 3.

2.3 SAP-1 MAINTAINING EMERGENCY PREPAREDNESS*

2.4 Administrative Procedure 6.2 PROCEDURE FOR CONTROL & DISTRIBUTION OF THE EMERGENCY PLAN & IMPLEMENTING PROCEDURES*

3.0 INITIATING EVENTS

Not Applicable.

4.0 PROCEDURE

4.1 Drill Conduct is discussed in Section 5 of this procedure. This section delineates the minimum acceptable activity for a drill at JAFNPP.

4.2 Exercise Conduct is discussed in Section 6 of this procedure. This section delineates the minimum acceptable activity for an exercise at JAFNPP.

4.3 Observer Conduct is discussed in Section 7 of this procedure. This section specifies the minimum acceptable, preparation, training and response required for an observer of a JAFNPP drill or exercise.

4.4 Critiques and Corrective Actions are discussed in Section 8 of this procedure. This section specifies the method in which problems with Emergency Preparedness at JAFNPP are handled.

5.0 DRILL CONDUCT

5.1 Drills shall be directed with the frequency established by SAP-1 - MAINTAINING EMERGENCY PREPAREDNESS.

5.2 Drills shall be directed by a Lead Controller who shall be responsible for conducting the drill in accordance with the drill scenario and the drill report.

5.2.1 The Lead Controller may conduct a briefing with drill participants. The intent of such a briefing would be to insure that drill participants understand their function and purpose in the drill.

5.2.2 The Lead Controller may delegate controller responsibilities to other individuals. Controllers and observers can be used for this purpose. A controller shall be called such when that individuals sole responsibility is to assist in the conduct of a drill. An observer can function as a controller when assigned the task of providing information or instruction during a certain aspect of a drill.

5.2.3 The Lead Controller shall insure that plant safety is not compromised by a drill, and may stop a drill at any time if in his opinion plant safety may be affected.

5.2.4 The Lead Controller shall attempt to collect the signatures of as many participants as possible for training documentation. This responsibility can be delegated to other controllers, or observers.

5.2.5 The Lead Controller shall commence and end the Drill, upon approval from the JAFNPP Resident Manager.

5.2.6 The Lead Controller shall ensure that drill observers are stationed to properly observe the drill.

5.3 During a drill, when (public address system) announcements are made, those announcements shall be prefaced or followed by the words "This is a Drill".

5.4 During a drill when contacting any offsite or non-NYPA institution, the individual shall insure that the organization fully realizes that no emergency exists onsite and that it is a test of the JAFNPP Emergency Plan.

5.5 Drills shall be conducted using the guidance established by Form SAP-6.1 "Drill Conduct Checklist".

5.6 The Emergency Planning Coordinator shall conduct an observer meeting prior to a drill. The meeting shall be to inform the observers of their specific tasks.

6.0 EXERCISE CONDUCT

An Exercise shall include all items specified for a drill with the following differences:

6.1 Exercises shall be conducted with the frequency established by SAP-1 - MAINTAINING EMERGENCY PREPAREDNESS.

6.2 A Lead Controller stationed in the Control Room shall be responsible for conducting the exercise in accordance with the written scenario. As a minimum, controllers shall also be present in the Technical Support Center, Operational Support Center, and the Emergency Operations Facility.

6.3 Every attempt should be made to include Federal, State and local input into the development of the exercise scenario.

6.3.1 The Exercise scenario shall be developed by a committee headed and organized by the Emergency Planning Coordinator at JAFNPP.

7.0 OBSERVER CONDUCT

7.1 Observers shall be used to record all significant events and the time at which they occur during a drill or exercise using Form SAP-6.2 "Evaluation Form". The drill or exercise scenario shall state the objectives of the drill or exercise which will determine the major areas for the observers to concentrate their observation. Actions to be observed include: the ability to control the emergency, timely and proper notification, availability and use of equipment and personnel for control and recovery, assessment of consequences of the emergency actions taken by emergency personnel, and the necessity for off-shift notifications.

7.2 Observers shall be selected with the concurrence of the Superintendent of Power.

7.3 There shall be enough observers to match all points specified in the drill/exercise scenario or drill form (SAP-1.2). The degree of observation shall be made based on the extent of the drill or scenario. As a general rule, however, observers shall be stationed to observe all expected major actions of the drill expected and as listed in objectives statement of the drill or exercise scenario. As a general rule of thumb, an observer shall be available at a ratio of one observer to each five participants. (With the exception of Plant accountability drills.) At least two observers must be available for drills and at least eight observers for an exercise.

7.4 In plant observers shall be badged following normal plant badging procedures, and are required to contact security during any accountability drills or exercises to identify themselves and their location.

7.5 Observers shall be vizibly identified as observers, and they should take no part in the action of the drill or exercise except to:

7.5.1 Indicate simulated conditions to the exercise or drill participants, (e.g., survey meter readings, contamination levels, etc.), but only after instructions by the lead controller or individual acting on behalf of lead controller.

7.5.2 Observe poor communication techniques and procedures and note/correct such occurrences when they occur.

7.5.3 Prevent the communication of simulated emergency conditions as actual conditions outside of the exercise or drill area and to ensure that radio or telephone messages are periodically preceded and ended by the statement "This is a Drill".

7.5.4 Prevent actions which might create a hazard to personnel or equipment. In such cases, observers shall require personnel participating in the exercise or drill to indicate the action verbally.

7.6 Observers shall be briefed as to their duties prior to the commencement of the drill or exercise. Drill observers should be briefed within 24-hours of the commencement of a drill. Exercise observers should be briefed within 24 hours of the commencement of an exercise and written aids and procedures shall be provided for use by the observers. This 24-hour time frame may be adjusted to compensate for unannounced exercises.

7.7 Training shall be provided to observers by the JAFNPP Training Department. The training provided for observers will entail the briefing listed in Section 7.6. The briefing shall include a review of the drill or exercise scenario, the observer duties with regard to the assigned areas of observation, and the key points to be noted. The Emergency Planning Coordinator shall provide the JAFNPP Training Department with a list of observers to be trained seven (7) days in advance of drill or exercise. Exceptions to the qualified observer list may be made by the Emergency Planning Coordinator. The exception shall be noted in a memorandum detailing why the exception was being made.

7.8 At the conclusion of the drill or exercise, the Emergency Planning Coordinator or the Fire Protection Supervisor shall collect the completed "Evaluation Forms" (SAP-6.2), compile a list of participants and conduct a critique with the observers.

7.9 Observers shall familiarize themselves with the duties and action requirements of the personnel they are monitoring. The Drill Report SAP-6.2 shall list Observers' Name, Organization, Area of Responsibility, and Reference Procedures. Observers shall review referenced procedures. Observers shall use the following as guidelines.

7.9.1 Control Room

The observer shall observe the action of personnel assigned to the Control Room and personnel who report to the Control Room for assignment. In addition, special attention will be given to the following.

- Use of map and overlays.
- Notifications to onsite personnel and offsite agencies.
- Request for the call-in of off duty personnel.
- Operations handling of accident conditions.
- Instructions given to Search and Rescue, Repair and Corrective Action Teams and H.P. Tech's by the Shift Supervisor (SS), as applicable.
- Does the SS handle the emergency by directing people or by trying to do the work himself?

- Are the time frames of actions by the SS reasonable enough?

- Actions of personnel in the Control Room.

- Communications with the EOF.

- Communications with the TSC.

- Actions of the STA.

7.9.2 Control Point

It is to be noted that all normal practices such as sign out and use of frisker and the portal monitor are to be accomplished unless the H.P. Technician gives other directions because of radiological conditions. The observer will pay special attention to the above along with the following.

- No one is wearing radiological protection clothing when leaving.

- All alarms from monitoring equipment are acknowledged.

7.9.3 Assembly Area

Observe the following for assembly area personnel:

- They seek out their assembly area, generally stay together as a group and remain orderly.

- Time of assembly and completed accountability.

7.9.4 Emergency Operation Facility

This is the command post for the Emergency and it should seem so to the observer. Look for the following things:

- The Emergency Director is in command of the EOF.

- Any extra personnel, spectators and those awaiting orders, are quietly standing out of the way.

- The Emergency Director has assigned personnel to various functions, i.e., Communications, Radiological Assessment.

- Has the Emergency Director contacted the Operations Coordinator?

- The Radiation Protection or Support Personnel are performing duties in an efficient manner and reporting results to the Emergency Director.

- Instrumentation deployed in the EOF is placed in an non-interfering position.

- How problems with the radio and telephone are handled.

- Using time as criteria, release rates and thyroid and whole body exposures to the offsite population are calculated quickly after the receipt of data from the Control Room or the Offsite Monitoring Team(s).

- The time frame of updates to offsite agencies and the reporting of exposure data and changes to site meteorological conditions, to those same agencies.

- The Emergency Director assigns, where possible, the duty of making routine calls to someone else thereby leaving himself free to command the action.

- Assessment Teams methods to make protective actions to offsite populations.

7.9.5 Off-Site Monitoring Teams

The observers shall observe the following items:

- Received KI dose, if necessary.

- Operational check performed on survey instruments, sample counter and sample pump before leaving the site.

- Equipment check off performed.

- Assignment of TLD's and dosimeters before leaving the site.

- Silver Zeolite Cartridges made available before leaving the site.

- Survey instrument operationally checked out and turned on prior to leaving to take field readings.

- Radio check out by communicating to EOF or TSC before leaving.

- Beta and gamma field surveys performed on the way to sample point.

- Sampling and field surveys performed at sample location.

- Instrument calibration performed and samples counted.

- Work performed in a professional manner.

7.9.6 On-Site Monitoring Team

On-site monitoring teams may be assigned field survey work along the perimeter of the site. Check on the following items:

- Where do they receive their instructions?

- Dosimeter and film badge are being worn.

- What type of survey instruments used.

- Do they pick up a portable radio with survey kit?

- Radio check performed.

- Field readings taken along the route to the designated area.

- Work performed in a professional manner.

7.9.7 Security Force

- Are all security personnel accounted for?

- Does security direct people to the nearest assembly area for accountability?

- Are access and egress roads blocked off?

7.9.8 Technical Support Center

- The area maintained as a controlled area.

- Are communications initiated?

- Are H.P. Surveys performed and by whom?

7.9.9 Operations Support Center

- How is it staffed?
- What and how many teams are brought to the OSC?
- Are phones continuously manned?
- Are H.P. Surveys performed and by whom?

7.9.10 Radiation Protection Office

- Do they report to the Radiation Protection Office when an Evacuation Alarm Sounds?
- Do they receive instructions from whom?
- Are accurate protective measures taken if an entry into the controlled area is required?
- Who are survey results reported to? (CR and/or EOF)

7.9.11 Fire Brigade

- Do they receive instructions and from whom?
- Are protective measures taken if an entry into a controlled are is required?
- Are Fire preplans consulted?
- Is assistance requested from local support fire departments?

8.0 CRITIQUES AND CORRECTIVE ACTIONS

8.1 Critique

A post exercise/drill critique should be held for observers and plant supervision by the Emergency Planning Coordinator. The critique should be held within 24 hours of the drill/exercise, at a time and place specified by the Emergency Planning Coordinator. This meeting shall be held to help resolve questions raised by various observers and plant supervisors and to develop a list of corrective actions as necessary. The observations should include those actions noted by the observers which were not in accordance with approved procedures. In addition, the exercise drill observers should identify any areas which require clarification, development or revision of procedures.

8.2 Emergency Plan Corrective Action Report

Following the critique, the Emergency Planning Coordinator shall develop a list of required Corrective Actions as identified at this critique. The required Corrective Actions shall be documented on SAP-6.3 "Emergency Plan Corrective Action Report". In addition to the items requiring corrective action identified at the critique, any additional items as identified on the SAP-6.2 "Evaluation Form" or through routine surveillances shall be detailed on SAP-6.3.

8.3 Review

Following the preparation of the Emergency Plan Corrective Action Reports, the Superintendent of Power shall review the contents of the reports.

8.3.1 The Superintendent of Power shall review and approve the final recommended corrective action to be taken, the individual assigned responsibility to complete the recommended corrective action, and the recommended completion dates.

8.3.2 Upon approval of the recommendations included in the Emergency Plan Corrective Action Report, the approved report shall be returned to the Emergency Planning Coordinator. The EPC shall then prepare SAP-6.4 "Emergency Plan Corrective Action Log". This log shall be used for work tracking associated with the Emergency Plan Corrective Actions.

8.4 Close Out

8.4.1 Each individual assigned responsibility to complete the recommended corrective action shall be sent a copy of each form SAP-6.3 that has been assigned to that individual. The action addresses shall resolve the problem, indicate on the form the final corrective action taken, sign and date it, and forward it to the Emergency Planning Coordinator.

8.4.2 In the event that an action addressee cannot complete the final corrective actions by the date specified on form SAP-6.3, a formal request for an extension (by memorandum) shall be presented to the Superintendent of Power. The request shall include the circumstances or reasons that necessitate the extension and an estimated completion date. The Superintendent of Power upon receipt of an extension request, shall review its justifications and, either approve or reject the request.

8.4.3 For approved extensions, the Superintendent of Power shall assign a revised completion data, and inform the action addressee of the extension. Furthermore, the Emergency Planning Coordinator shall be directed to update the Emergency Plan Corrective Action Log, and file the request for extension. For rejected extensions, the action addressee shall be informed and the request forwarded to the Emergency Planning Coordinator.

8.4.4 The Emergency Planning Coordinator shall, after the preparation and review of an Emergency Plan Corrective Action Report, present the report(s) to the Plant Operating Review Committee (PORC). The PORC shall review the report(s). This review shall be incorporated into the PORC meeting minutes. The Emergency Planning Coordinator shall maintain the active file on Emergency Plan Corrective Action Reports. The JAFNPP Training Department shall maintain the file on training reports.

9.0 FIGURES, FORMS AND ATTACHMENTS

9.1 FORM SAP-6.1 Drill or Exercise Conduct Checklist

9.2 FORM SAP-6.2 Evaluation Form

9.3 FORM SAP-6.3 Emergency Plan Corrective Action Report

9.5 FORM SAP-6.4 Emerge.. Plan Corrective Action Log

FORM SAP 6.1

DRILL OR EXERCISE CONDUCT CHECKLIST

_____	1.	Prepare a drill or exercise scenario.
_____	2.	Prepare a drill or exercise report.
_____	3.	Present the drill or exercise to the Plant Operating Review Committee Representative for approval.
_____	4.	Brief observers on the entire drill or exercise.
_____	5.	Brief the individual observers on specified tasks.
_____	6.	Issue Observer Aids and Drill/Exercise Observation Sheet.
_____	7.	Initiate the drill or exercise.
_____	8.	Ensure the "flow" of activity throughout the drill or exercise.
_____	9.	Terminate the drill or exercise when it's purpose is accomplished.
_____	10.	Conduct a critique with participants or observers.
_____	11.	Collect Drill/Exercise Observation Sheets.
_____	12.	Complete Emergency Plan Corrective Action Report, list all deficiencies and recommendations.
_____	13.	Present the Emergency Plan Corrective Action Report to PORC.
_____	14.	Complete action required on deficiencies.

EVALUATION FORM

OBSERVER	CONTROLLER	LOCATION	DATE	REFERENCE PROCEDURES

OBJECTIVE 1: EMERGENCY CLASSIFICATION LEVELS

Demonstrate the ability to monitor, understand and use emergency classification levels (ECL) through the appropriate implementation of emergency functions and activities corresponding to ECL's as required by the scenario. The four ECL's are: Notification of Unusual Event, Alert, Site Area Emergency and General Emergency.

NUREG REF	POINTS OF REVIEW	YES	NO	N/A	N/O
D.3	1. Did the organization declare appropriate emergency classification levels?	___	___	___	___
D.3	2. Were staff notified of ECL's?	___	___	___	___
D.3	3. When was your location notified of the following ECL's? [Specify the time staff were notified and not the time that the ECL was declared by the utility. Also, specify if these notifications were verified (if required).]				
	ECL — TIME NOTIFIED — WERE NOTIFICATIONS VERIFIED? (IF REQUIRED)				
	Notification of Unusual Event _____	___	___	___	___
	Alert _____ ___	___	___	___	
	Site Area Emergency _____	___	___	___	___
	General Emergency _____	___	___	___	___
D.3 D.4	4. Were the ECL's prominently displayed?	___	___	___	___
D.3	5. Were the staff aware of the current ECL?	___	___	___	___

EVALUATION FORM

OBSERVER	CONTROLLER	LOCATION	DATE	REFERENCE PROCEDURES

OBJECTIVE 1: EMERGENCY CLASSIFICATION LEVELS

NUREG REF	POINTS OF REVIEW	YES	NO	N/A	N/O
D.3 D.4	6. Were the relevant functions and activities implemented in a manner that is consistent with the organization's emergency plan and procedures? If not, explain why the organization did not follow its emergency plan and procedures.	___	___	___	___

EVALUATION FORM

OBSERVER	CONTROLLER	LOCATION	DATE	REFERENCE PROCEDURES

EVALUATION: Objective 1 was: _____ met _____ not met

NARRATIVE SUMMARY: (Describe play of objective, including points of review)

EVALUATION FORM

OBSERVER	CONTROLLER	LOCATION	DATE	REFERENCE PROCEDURES

OBJECTIVE 2: MOBILIZATION OF EMERGENCY PERSONNEL

Demonstrate the ability to fully alert, mobilize and activate personnel for both facility and field-based emergency functions.

NUREG REF	POINTS OF REVIEW	YES	NO	N/A	N/O
E.1	1. During what time period were calls/contacts made to notify offsite response organizations as required by the plan? Start: _____ Completed: _____ Who made the calls/contacts? _____				
E.1	2. What method(s) was used to notify organizations? _____ _____ If a written call-list was used, were staff names and telephone numbers current and correct?				
E.2	3. Were required staff alerted in a timely manner?	___	___	___	___
E.2	4. During what time period were facility staff activated/deployed? Start: _____ Completed: _____	___	___	___	___
E.2	5. List all staff who were present at your location:				

_____ _____ _____
_____ _____ _____
_____ _____ _____
_____ _____ _____
_____ _____ _____
_____ _____ _____
_____ _____ _____
_____ _____ _____

EVALUATION FORM

OBSERVER	CONTROLLER	LOCATION	DATE	REFERENCE PROCEDURES

OBJECTIVE 2: MOBILIZATION OF EMERGENCY PERSONNEL

NUREG REF	POINTS OF REVIEW	YES	NO	N/A	N/O
E.1 E.2	6. Were staff dispatched to other facilities or locations, if required?	___	___	___	___

If so, list these staff members and their time of dispatch.

STAFF	TIME OF DISPATCH

E.1 E.2	7. Were the relevant functions and activities implemented in a manner that is consistent with the organization's emergency plan and procedures?	___	___	___	___

If not, explain why the organization did not follow its emergency plan and procedures:

EVALUATION FORM

OBSERVER	CONTROLLER	LOCATION	DATE	REFERENCE PROCEDURES

EVALUATION: Objective 2 was: _____ met _____ not met

NARRATIVE SUMMARY: (Describe play of objective, including points of review)

EVALUATION FORM

OBSERVER	CONTROLLER	LOCATION	DATE	REFERENCE PROCEDURES

OBJECTIVE 3: DIRECTION AND CONTROL

Demonstrate the ability to direct, coordinate and control emergency activities.

NUREG REF	POINTS OF REVIEW	YES	NO	N/A	N/O
A.1.d	1. Was an individual effectively in charge of the emergency response?	___	___	___	___
A.1.d	2. If so, name this individual: _____				
A.1.b A.2.a	3. Were periodic briefings held, to update staff on situation?	___	___	___	___
A.1.b A.2.a	4. Were staff (as appropriate) involved in decision making?	___	___	___	___
A.1.b A.2.a	5. Was a copy of the plan available for reference?	___	___	___	___
A.2.a	6. Were message logs kept for all incoming and outgoing messages and transmissions?	___	___	___	___
	Were message reproduced and distributed, if appropriate?	___	___	___	___
A.2.a	7. Did the facility use an internal message-handling system?	___	___	___	___
	If so, was information provided to staff in a prompt manner?	___	___	___	___
	If so, was a record kept of internal messages through logs or file copies?	___	___	___	___
A.1.d	8. Were protective action decisions and implementation of these decisions coordinated effectively with all appropriate organizations?	___	___	___	___

EVALUATION FORM

OBSERVER	CONTROLLER	LOCATION	DATE	REFERENCE PROCEDURES

OBJECTIVE 3: DIRECTION AND CONTROL

NUREG REF	POINTS OF REVIEW	YES	NO	N/A	N/O
A.1.a A.1.b A.1.d A.2.a	9. Were the relevant functions and activities implemented in a manner that is consistent with the organization's emergency plan and procedures? If not, explain why the organization did not follow its emergency plan and procedures: _____ _____ _____ _____ _____ _____ _____ _____	__	__	__	__

EVALUATION FORM

OBSERVER	CONTROLLER	LOCATION	DATE	REFERENCE PROCEDURES

EVALUATION: Objective 3 was: _____ met _____ not met

NARRATIVE SUMMARY: (Describe play of objective, including points of review)

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SAP-6.2

EVALUATION FORM

OBSERVER	CONTROLLER	LOCATION	DATE	REFERENCE PROCEDURES

OBJECTIVE 4: COMMUNICATIONS

Demonstrate ability to communicate with all appropriate locations, organizations, and field personnel.

NUREG REF	POINTS OF REVIEW	YES	NO	N/A	N/O
F.1.b F.1.d F.2	1. Identify communications systems demonstrated during the exercise: _____ Commercial telephone. Specify number of lines available: _____ _____ Conferencing capability _____ Cellular telephone in vehicle _____ Radio system. Specify type of systems: _____ _____ Computer link. Describe system: _____ _____ Facsimile machine _____ Other (specify) _____				
F.1.b F.1.d F.2	2. Identify organizations that this facility had communication links with: _____				
F.1.b	3. Were primary communications systems able to handle communication flow without undue delays?				
F.1.b	4. What was the average delay for the primary communications system?				

System	Average Delay (Minutes)

EVALUATION FORM

OBSERVER	CONTROLLER	LOCATION	DATE	REFERENCE PROCEDURES

OBJECTIVE 4: COMMUNICATIONS

NUREG REF	POINTS OF REVIEW	YES	NO	N/A	N/O
F.1.b F.1.d	5. Were backup systems demonstrated?		___	___	___ ___
	If so, did they function properly?		___	___	___ ___
F.1.b F.1.d	6. Did any of the communications systems break down?	___	___	___	___
	If so, describe the break downs with respect to what occurred; the primary or backup system; the length of time; impact on organizational functioning; steps taken to correct problem, and any use of a backup system:				
F.1.b F.1.d F.2	7. Were the relevant functions and activities implemented in a manner that is consistent with the organization's emergency plan and procedures?	___	___	___	___
	If not, explain why the organization did not follow its emergency plan and procedures:				

EVALUATION FORM

OBSERVER	CONTROLLER	LOCATION	DATE	REFERENCE PROCEDURES

EVALUATION: Objective 4 was: _____ met _____ not met

NARRATIVE SUMMARY: (Describe play of objective, including points of review)

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SAP-6.2

EVALUATION FORM

OBSERVER	CONTROLLER	LOCATION	DATE	REFERENCE PROCEDURES

OBJECTIVE 5: FACILITIES, EQUIPMENT AND DISPLAYS

Demonstrate the adequacy of facilities, equipment, displays and other materials to support emergency operations.

NUREG REF	POINTS OF REVIEW	YES	NO	N/A	N/O
H.3	1. Were the following facility components sufficient to support emergency operations?		Sufficient		
	a. Space	___	___	___	___
	b. Furnishings	___	___	___	___
	c. Lighting	___	___	___	___
	d. Rest rooms	___	___	___	___
	e. Ventilation	___	___	___	___
	f. Backup power (if required per plan)	___	___	___	___
	g. Other (specify) _____	___	___	___	___
	_____	___	___	___	___
H.3	2. Was the following equipment available (check if it was) and sufficient to support emergency operations?				
	Available Equipment		Sufficient		
	_____ Typewriter	___	___	___	___
	_____ Computer/word processor	___	___	___	___
	_____ Copier	___	___	___	___
	_____ Kitchen supplies	___	___	___	___
	_____ Cots	___	___	___	___
	_____ Other (specify) _____	___	___	___	___
	_____	___	___	___	___
	If not, identify needs:				
	_____	___	___	___	___
	_____	___	___	___	___
	_____	___	___	___	___

EVALUATION FORM

OBSERVER	CONTROLLER	LOCATION	DATE	REFERENCE PROCEDURES

OBJECTIVE 5: FACILITIES, EQUIPMENT AND DISPLAYS

NUREG REF	POINTS OF REVIEW	YES	NO	N/A	N/O
H.3	3. Was access to the facility controlled?	___	___	___	___
H.3	4. Identify maps which were used:				
	Maps		Provided		
	Plume EPZ with appropriate planning areas labeled	___	___	___	___
	Evacuation Routes	___	___	___	___
	Plume EPZ population by planning areas	___	___	___	___
	Relocation Centers	___	___	___	___
	Radiological monitoring points	___	___	___	___
	Ingestion EPZ for agricultural information	___	___	___	___
	Other (specify) _____	___	___	___	___
	_____	___	___	___	___
H.3	5. Were status boards used and positioned for viewing by staff?	___	___	___	___
	If so, identify information placed on status boards:		Provided on Status Boards		
	Emergency classification levels	___	___	___	___
	Protective action decisions	___	___	___	___
	Weather data (actual and forecast)		___	___	___ ___
	Other (specify) _____	___	___	___	___
	_____	___	___	___	___
H.3	6. Were status boards and displays updated in a timely manner (i.e., were they updated within 10 minutes of notification of status changes)?		___	___	___ ___

EVALUATION FORM

OBSERVER	CONTROLLER	LOCATION	DATE	REFERENCE PROCEDURES

OBJECTIVE 5: FACILITIES, EQUIPMENT AND DISPLAYS

NUREG REF	POINTS OF REVIEW	YES	NO	N/A	N/O
H.3 J.10.a J.10.b J.11	7. Were the relevant functions and activities addressed above implemented in a manner that is consistent with the organization's emergency plan and procedures? If not, explain why the organization did not follow its emergency plan and procedures: _____ _____ _____ _____ _____	__	__	__	__

EVALUATION FORM

OBSERVER	CONTROLLER	LOCATION	DATE	REFERENCE PROCEDURES

EVALUATION: Objective 5 was: _____ met _____ not met

NARRATIVE SUMMARY: (Describe play of objective, including points of review)

OBSERVER	CONTROLLER	LOCATION	DATE	REFERENCE PROCEDURES

OBJECTIVE 6: EMERGENCY WORKER EXPOSURE CONTROL

Demonstrate the ability to continuously monitor and control emergency worker exposure.

NUREG REF	POINTS OF REVIEW	YES	NO	N/A	N/O
K.3.a	1. Did each emergency worker who entered the plume EPZ have a non self-reading permanent record dosimeter?	___	___	___	___
	If not, identify the reason why a permanent record dosimeter was not available to each emergency worker: _____ _____ _____ _____				
K.3.a	2. Did each emergency worker who entered the plume EPZ have:				
	_____(a) Two direct reading dosimeters with different ranges?				
	List the ranges of the two dosimeters: #1 _____ #2 _____				
	OR:				
	_____(b) One direct reading dosimeter?				
	List the range of the dosimeter: _____				
K.3.a	3. Did each team or group have access to a charger for the direct reading dosimeter?	___	___	___	___
K.3.b	4. Did each worker have an exposure record?	___	___	___	___

EVALUATION FORM

OBSERVER	CONTROLLER	LOCATION	DATE	REFERENCE PROCEDURES

OBJECTIVE 6: EMERGENCY WORKER EXPOSURE CONTROL

NUREG REF	POINTS OF REVIEW	YES	NO	N/A	N/O
K.3.b	5. Were appropriate instructions issued?	___	___	___	___
K.3.b	6. Were the dosimeters zeroed or the initial readings recorded?	___	___	___	___
K.3.b	7. Were the dosimeters read and the exposure recorded for each emergency worker?	___	___	___	___
K.4	8. What exposure was authorized for the mission? _____				
K.4	9. Did the emergency workers know who to contact for authorization to incur exposures in excess of the authorized mission exposure limit?	___	___	___	___
K.3.b	10. Did each emergency worker know what to do if they received an exposure higher than authorized?	___	___	___	___
K.3.a	11. Were the relevant functions and activities implemented in a manner that is consistent with the organization's emergency plan and procedures?	___	___	___	___
	If not, explain why the organization did not follow its emergency plan and procedures: _____ _____ _____				

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EVALUATION FORM

OBSERVER	CONTROLLER	LOCATION	DATE	REFERENCE PROCEDURES

EVALUATION: Objective 6 was: _____ met _____ not met

NARRATIVE SUMMARY: (Describe play of objective, including points of review)

EVALUATION FORM

OBSERVER	CONTROLLER	LOCATION	DATE	REFERENCE PROCEDURES

OBJECTIVE 7: FIELD RADIOLOGICAL MONITORING - AMBIENT RADIATION MONITORING

Demonstrate the appropriate equipment and procedures for determining field radiation measurements.

NUREG REF	POINTS OF REVIEW	YES	NO	N/A	N/O
I.8 I.11	1. Did the team have a survey instrument?	___	___	___	___
	Was it the instrument specified in the plan?	___	___	___	___
	If not, was it equivalent?	___	___	___	___
I.8 I.11	2. Did the team have a high-range (greater than 0.05 rem/hour, survey instrument?	___	___	___	___
	Was it the instrument specified in the plan?	___	___	___	___
	If not, was it equivalent?	___	___	___	___
I.8 I.11	3. Did the team have access to equipment spares?	___	___	___	___
I.8 I.11	4. Did they perform battery checks and source checks?	___	___	___	___
	Was there evidence that the instruments were within their calibration period?	___	___	___	___
I.8 I.11	5. Were Gamma-only and Beta+Gamma readings made at both about 1 meter (waist level) and about 2 cm (near ground level)?	___	___	___	___
I.8 I.11	6. Did the team avoid contaminating instruments?	___	___	___	___
I.8 I.11	7. Were readings logged along with location, time and date?	___	___	___	___

EVALUATION FORM

OBSERVER	CONTROLLER	LOCATION	DATE	REFERENCE PROCEDURES

OBJECTIVE 7: FIELD RADIOLOGICAL MONITORING - AMBIENT RADIATION MONITORING

NUREG REF	POINTS OF REVIEW	YES	NO	N/A	N/O
I.8 I.11	8. Was the team able to find and arrive at monitoring location promptly?	___	___	___	___
I.8 I.11	9. Did the staff, in general, display adequate training and knowledge?	___	___	___	___
I.8 I.11	10. Did the team transmit field readings to the appropriate location promptly?	___	___	___	___
I.8 I.11 H.12	11. Were the relevant functions and activities implemented in a manner that is consistent with the organization's emergency plan and procedures?	___	___	___	___

If not, explain why the organization did not follow its emergency plan and procedures:

_____

_____

_____

_____

_____

EVALUATION FORM

OBSERVER	CONTROLLER	LOCATION	DATE	REFERENCE PROCEDURES

EVALUATION: Objective 7 was: _____ met _____ not met

NARRATIVE SUMMARY: (Describe play of objective, including points of review)

EVALUATION FORM

OBSERVER	CONTROLLER	LOCATION	DATE	REFERENCE PROCEDURES

OBJECTIVE 8: FIELD RADIOLOGICAL MONITORING - AIRBORNE IODINE MONITORING

Demonstrate the appropriate equipment and procedures for the measurement of airborne radioiodine concentrations as low as 10^{-7} (.0000001) microcuries per cc in the presence of noble gases.

NUREG REF	POINTS OF REVIEW	YES	NO	N/A	N/O
I.8 I.9	1. Did the field team use monitoring equipment identified in the organization's plan?	___	___	___	___
	If not, was it equivalent?	___	___	___	___
I.8 I.9	2. Did the field team have power to operate the air sampler (pump), either a 12 volt vehicle source or portable generator for 120 volt systems?	___	___	___	___
I.8 I.9	3. Was a silver-based adsorber available? (A charcoal adsorber is acceptable for exercise demonstrations if silver media is available.)	___	___	___	___
I.8 I.9	4. Were air samples taken using the proper flow rate and sample duration, as specified in plan and procedures?	___	___	___	___
I.8 I.9	5. Were samples/components properly bagged and labeled with time, date, location and identification of person who took them?	___	___	___	___
I.8 I.9	6. Did the field team leave the plume and travel to a low-background area before attempting to count the air sample media?	___	___	___	___
I.8 I.9	7. Was the instrument used to count the air sample media within calibration date?	___	___	___	___
I.8 I.9	8. Was a fixed (reproducible) geometry used to count the air sample media?	___	___	___	___

EVALUATION FORM

OBSERVER	CONTROLLER	LOCATION	DATE	REFERENCE PROCEDURES

OBJECTIVE 8: FIELD RADIOLOGICAL MONITORING - AIRBORNE IODINE MONITORING

NUREG REF	POINTS OF REVIEW	YES	NO	N/A	N/O
I.8 I.9	9. Were the count rates of air sample media transmitted to the proper location promptly?	___	___	___	___
I.8 I.9	10. Were the relevant functions and activities implemented in a manner that is consistent with the organization's emergency plan and procedures?	___	___	___	___

If not, explain why the organization did not follow its emergency plan and procedures:

_____

_____

_____

_____

_____

EVALUATION FORM

OBSERVER	CONTROLLER	LOCATION	DATE	REFERENCE PROCEDURES

EVALUATION: Objective 8 was: _____ met _____ not met

NARRATIVE SUMMARY: (Describe play of objective, including points of review)

EVALUATION FORM

OBSERVER	CONTROLLER	LOCATION	DATE	REFERENCE PROCEDURES

OBJECTIVE 9: FIELD RADIOLOGICAL MONITORING - PARTICULATE ACTIVITY

Demonstrate the ability to obtain samples of particulate activity in the airborne plume and promptly perform laboratory analyses.

NUREG REF	POINTS OF REVIEW	YES	NO	N/A	N/O
I.10	1. Did a field monitoring team send an iodine particulate filter to a laboratory for analyses? Answer questions below if answer is yes.	___	___	___	___
I.10	2. Identify field team official who arranged for filter to be sent to laboratory. _____				
I.10	3. When was the filter sent to the laboratory? What was the time interval between the taking of iodine samples and sending of the filter to the laboratory? _____ _____				
I.10	4. Who took the filter to the laboratory? _____ Was this person a "runner" who is not part of the field monitoring team?	___	___	___	___
I.10	5. Identify the name and location of this laboratory: _____				
I.10	6. When did the laboratory receive the filter or simulated filter? _____				
I.10	7. Were established laboratory procedures for analysis of the filter followed?	___	___	___	___

EVALUATION FORM

OBSERVER	CONTROLLER	LOCATION	DATE	REFERENCE PROCEDURES

OBJECTIVE 9: FIELD RADIOLOGICAL MONITORING - PARTICULATE ACTIVITY

NUREG REF	POINTS OF REVIEW	YES	NO	N/A	N/O
I.10	8. Describe the results of the laboratory analysis based on a controller inject message provided to laboratory staff: _____ _____ _____ _____ _____				
I.10	9. When did the laboratory complete its analysis? (Or, when did the laboratory receive the controller inject message? _____				
I.10 H.12	10. Did the laboratory provide the results of its analysis to other organizations?	___	___	___	___
	If so, identify these organizations and the time they were provided the data:				
	Organization / Time Provided Data: _____ _____ _____ / _____ _____ _____				
I.10 H.12	11. Were the relevant functions and activities implemented in a manner that is consistent with the organization's emergency plan and procedures?	___	___	___	___
	If not, explain why the organization did not follow its emergency plan and procedures: _____ _____ _____				

EVALUATION FORM

OBSERVER	CONTROLLER	LOCATION	DATE	REFERENCE PROCEDURES

EVALUATION: Objective 9 was: _____ met _____ not met

NARRATIVE SUMMARY: (Describe play of objective, including points of review)

EVALUATION FORM

OBSERVER	CONTROLLER	LOCATION	DATE	REFERENCE PROCEDURES

OBJECTIVE 10: PLUME DOSE PROJECTION

Demonstrate the ability, within the plume exposure pathway, to project dosage to the public via plume exposure, based on plant and field data.

NUREG REF	POINTS OF REVIEW	YES	NO	N/A	N/O
I.10	1. Were primary and backup systems available for dose projection?	___	___	___	___
I.10	2. Was plant status information promptly provided so that offsite dose projections could be made?	___	___	___	___
I.10	3. Were new dose projections made, if appropriate, based upon field monitoring data?	___	___	___	___
	Were new dose projections made as plant status changed?	___	___	___	___
I.10	4. Was the projected plume location plotted and displayed?				
	_____Plotted _____Displayed				
I.10	5. Were field monitoring teams properly directed so that the plume could be defined?	___	___	___	___
	Was the plume properly defined by field teams?	___	___	___	___
I.10	6. Was field monitoring team data compared with projected dose rates?	___	___	___	___
I.10	7. Were projected wind shifts considered in the determination of the area of concern?	___	___	___	___

EVALUATION FORM

OBSERVER	CONTROLLER	LOCATION	DATE	REFERENCE PROCEDURES

OBJECTIVE 10: PLUME DOSE PROJECTION

NUREG REF	POINTS OF REVIEW	YES	NO	N/A	N/O
I.10	8. Were the relevant functions and activities implemented in a manner that is consistent with the organization's emergency plan and procedures? If not, explain why the organization did not follow its emergency plan and procedures: _____ _____ _____ _____ _____				

EVALUATION FORM

OBSERVER	CONTROLLER	LOCATION	DATE	REFERENCE PROCEDURES

EVALUATION: Objective 10 was: _____ met _____ not met

NARRATIVE SUMMARY: (Describe play of objective, including points of review)

EVALUATION FORM

OBSERVER	CONTROLLER	LOCATION	DATE	REFERENCE PROCEDURES

OBJECTIVE 11: PLUME PROTECTIVE ACTION DECISION MAKING

Demonstrate the ability to make appropriate protective action decisions, based on projected or actual dosage, EPA PAG's, availability of adequate shelter, evacuation time estimates and other relevant factors.

NUREG REF	POINTS OF REVIEW	YES	NO	N/A	N/O
J.9	1. Were the correct PAG's used in consideration of the technical basis for the protective action recommendation?	___	___	___	___
J.10.m	2. Were any preplanned PAR's implemented for special populations?	___	___	___	___
	If so, were they in accord with the plan?	___	___	___	___
J.10.m	3. Were factors other than projected dose, i.e., quality of shelter, ETE's, etc., considered, if appropriate, in the protective action decision making process?	___	___	___	___
	If so, specify these factors: _____ _____ _____				
J.9	4. Were new protective action recommendations made as the plant status changed?	___	___	___	___
J.9	5. Were protective action recommendations made without undue delay, recognizing that some discussion is appropriate in all but the fastest-breaking accidents?	___	___	___	___

EVALUATION FORM

OBSERVER	CONTROLLER	LOCATION	DATE	REFERENCE PROCEDURES

OBJECTIVE 11: PLUME PROTECTIVE ACTION DECISION MAKING

NUREG REF	POINTS OF REVIEW	YES	NO	N/A	N/O
J.9	7. List all protective action recommendations and the time that the protective action recommendation was made. PROTECTIVE ACTION RECOMMENDATION TIME DECISION MADE _____ _____ _____ _____ _____ _____ _____ _____ _____ _____ _____ _____ _____ _____				
J.9 J.10.d J.10.m	8. Were the relevant functions and activities implemented in a manner that is consistent with the organization's emergency plan and procedures? If not, explain why the organization did not follow its emergency plan and procedures: _____ _____ _____ _____ _____	___	___	___	___

EVALUATION FORM

OBSERVER	CONTROLLER	LOCATION	DATE	REFERENCE PROCEDURES

EVALUATION: Objective 11 was: _____ met _____ not met

NARRATIVE SUMMARY: (Describe play of objective, including points of review)

EVALUATION FORM

OBSERVER	CONTROLLER	LOCATION	DATE	REFERENCE PROCEDURES

OBJECTIVE 12: ALERT, NOTIFICATION AND EMERGENCY INFORMATION - INITIAL ALERT AND NOTIFICATION

Demonstrate the ability to initially alert the public within the 10-mile EPZ and begin dissemination of an instructional message within 15 minutes of a decision by appropriate State and/or local official(s).

NUREG REF	POINTS OF REVIEW	YES	NO	N/A	N/O
E.5 E.6	1. Did involved organizations demonstrate the capability to disseminate an alert signal and initiate and instructional message to the public within 15 minutes for the plume Emergency Planning Zone?	___	___	___	___
	(This demonstration is required for a pre-agreed to alert and notification sequence. Also, the timing of the 15-minute sequence starts with designated offsite officials making the decision to implement a protective action or notify the public and ends with the instructional message being put on the air.)				
	Was this capability actually demonstrated?	___	___	___	___

If not, describe how the capability was simulated: _____

_____

_____

Identify alerting and notification methods used, if actually demonstrated:

Alert		Notification	
____	Fixed siren system	____	Siren system with PA system
____	Mobile siren system	____	EBS
____	Tone alert radios	____	NOAA Weather Radio
____	Route alerting (primary)	____	Tone alert radios
____	Mobile PA alerting units (e.g., helicopter, boat or airplane)	____	Mobile PA system notification units (e.g., helicopter, boat or airplane)
____	Telephone calls to individuals or institutions	____	Telephone calls to individuals or institutions
____	Other (specify): _____	____	Other (specify): _____

EVALUATION FORM

OBSERVER	CONTROLLER	LOCATION	DATE	REFERENCE PROCEDURES

OBJECTIVE 12: ALERT, NOTIFICATION AND EMERGENCY INFORMATION - INITIAL ALERT AND NOTIFICATION

NUREG REF	POINTS OF REVIEW	YES	NO	N/A	N/O
	Identify involved organizations: _____ _____ _____ _____				
E.5 E.6	2. If <u>primary</u> route alerting was demonstrated, answer the following questions:				
	How and when was the route alerting team(s) notified to initiate its functions?				
	How: _____ _____				
	When: _____				
	What route alerting vehicle(s) (e.g., police car) did you observe? _____				
	For each route observed:				
	When did route alerting being and when was it completed?				
	Route 1 Time Initiated _____ Time Completed _____				
	Route 2 _____ _____				
	Route 3 _____ _____				
	Was the route alerting completed within 15 minutes (or 45 minutes if for isolated area 5 to 10 miles from the plant)?				
	Route 1	___	___	___	___
	Route 2	___	___	___	___
	Route 3	___	___	___	___

EVALUATION FORM

OBSERVER	CONTROLLER	LOCATION	DATE	REFERENCE PROCEDURES

OBJECTIVE 12: ALERT, NOTIFICATION AND EMERGENCY INFORMATION - INITIAL ALERT AND NOTIFICATION

NUREG REF	POINTS OF REVIEW	YES	NO	N/A	N/O
	Did route alerting team(s) have route maps?	___	___	___	___
	Did route alerting team(s) have a written message to read over the PA system? (If so, secure a copy of the message.)	___	___	___	___
	Were the contents of the message accurate according to recommended protective actions?	___	___	___	___
	Was the PA system actually demonstrated?	___	___	___	___
E.5 E.6	3. Document the times associated with the following steps in the pre-agreed to 15-minute alerting and notification sequence.				

Step	Time
a. Message from utility with PAR	_____
b. Decision made by designated offsite officials	_____
c. Alert signal activated	_____
d. Instructional message begun	_____

E.5 E.6	4. Did involved organizations demonstrate the capability to disseminate an alert signal and initiate instructional messages to special populations (e.g., extremely rural low-population areas; recreational areas and open water) from 5 to 10 miles from the plant within 45 minutes?	___	___	___	___
	(This demonstration is required for the pre-agreed to alert and notification sequence. Also, the timing of the 45-minute sequence starts with the designated offsite officials making the decision to implement a protective action or notify the public and ends with the initiation of an instructional message to the public.)				

OBSERVER	CONTROLLER	LOCATION	DATE	REFERENCE PROCEDURES

OBJECTIVE 12: ALERT, NOTIFICATION AND EMERGENCY INFORMATION - INITIAL ALERT AND NOTIFICATION

NUREG REF	POINTS OF REVIEW	YES	NO	N/A	N/O
	Was this capability actually demonstrated by the involved organizations or simulated? (provide explanation below)	___	___	___	___

_____

_____

_____

Alert		Notification	
_____	Mobile siren system	_____	EBS
_____	Tone alert radios	_____	NOAA Weather Radio
_____	Route alerting (backup)	_____	Tone alert radios
_____	Mobile PA alerting units (e.g., helicopter, boat or airplane)	_____	Route alerting (backup)
_____	Telephone calls to individuals or institutions	_____	Mobile PA system notification units (e.g., helicopter, boat or airplane)
_____	Other (specify): _____ _____	_____	Telephone calls to individuals or institutions
		_____	Other (specify):

Identify involved organizations:

_____

_____

_____

_____

E.5 E.6 2. If backup route alerting was demonstrated, answer the following questions:

How and when was the route alerting team(s) notified to initiate its functions?

How: _____

_____

When: _____

EVALUATION FORM

OBSERVER	CONTROLLER	LOCATION	DATE	REFERENCE PROCEDURES

OBJECTIVE 12: ALERT, NOTIFICATION AND EMERGENCY INFORMATION - INITIAL ALERT AND NOTIFICATION

NUREG REF	POINTS OF REVIEW	YES	NO	N/A	N/O
	What route alerting vehicle(s) (e.g., police car) did you observe?				
	For each route observed:				
	When did route alerting being and when was it completed?				
	Time Initiated / Time Completed				
	Route 1				
	Route 2				
	Route 3				
	Did route alerting team(s) have route maps?				
	Did route alerting team(s) have a written message to read over the PA system? (If so, secure a copy of the message.)				
	Were the contents of the message accurate according to recommended protective actions?				
	Was the PA system actually demonstrated?				
E.5 E.6	6. Document the times of the following steps for the pre-agreed to, 45-minute special alerting and notification sequence.				

Step	Time
a. Message from utility with PAR	
b. Decision made by designated offsite officials	
[illegible] signal activated	
[illegible] message begun	

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12.5

EVALUATION FORM

OBSERVER	CONTROLLER	LOCATION	DATE	REFERENCE PROCEDURES

OBJECTIVE 12: ALERT, NOTIFICATION AND EMERGENCY INFORMATION - INITIAL ALERT AND NOTIFICATION

NUREG REF	POINTS OF REVIEW	YES	NO	N/A	N/O
E.5 E.6	7. Who authorized the 15 and 45-minute alerting and notification? _____ _____ _____				
E.5 E.6	8. Was the public notification process by the involved organizations effectively coordinated among involved organizations? If not, describe coordination problems: _____ _____ _____	___	___	___	___
E.7	9. Obtain copies of all instructional messages and attach to this form (Objective 12).				
E.7	10. Was the content appropriate to protective action decisions? Identify the scope of the instructional message: _____ Plume exposure pathway _____ Ingestion exposure pathway _____ Re-entry, resettlement and relocation _____ Other (specify) _____ _____	___	___	___	___
E.5 E.6	11. Were the timing of the alert signal and the instructional message coordinated to ensure that the alert signal came first, followed by the initiation of the instructional message within a few minutes?	___	___	___	___
E.5	12. Were arrangements made to periodically rebroadcast the instructional message?	___	___	___	___

EVALUATION FORM

OBSERVER	CONTROLLER	LOCATION	DATE	REFERENCE PROCEDURES

OBJECTIVE 12: ALERT, NOTIFICATION AND EMERGENCY INFORMATION - INITIAL ALERT AND NOTIFICATION

NUREG REF	POINTS OF REVIEW	YES	NO	N/A	N/O
	What route alerting vehicle(s) (e.g., police car) did you observe? _____				
	For each route observed:				
	When did route alerting being and when was it completed?				
	Route 1 Time Initiated _____ Time Completed _____				
	Route 2 _____ _____				
	Route 3 _____ _____				
	Did route alerting team(s) have route maps?	___	___	___	___
	Did route alerting team(s) have a written message to read over the PA system? (If so, secure a copy of the message.)	___	___	___	___
	Were the contents of the message accurate according to recommended protective actions?	___	___	___	___
	Was the PA system actually demonstrated?	___	___	___	___
E.5 E.6	6. Document the times of the following steps for the pre-agreed to, 45-minute special alerting and notification sequence.				

Step	Time
a. Message from utility with PAR	_____
b. Decision made by designated offsite officials	_____
c. Alert signal activated	_____
d. Instructional message begun	_____

OBSERVER	CONTROLLER	LOCATION	DATE	REFERENCE PROCEDURES

OBJECTIVE 12: ALERT, NOTIFICATION AND EMERGENCY INFORMATION - INITIAL ALERT AND NOTIFICATION

NUREG REF	POINTS OF REVIEW	YES	NO	N/A	N/O
E.5 E.6 E.7	13. Were the relevant functions and activities implemented in a manner that is consistent with the organization's emergency plan and procedures? If not, explain why the organization did not follow its emergency plan and procedures: _____ _____ _____ _____ _____	___	___	___	___

EVALUATION FORM

OBSERVER	CONTROLLER	LOCATION	DATE	REFERENCE PROCEDURES

EVALUATION: Objective 12 was: _____ met _____ not met

NARRATIVE SUMMARY: (Describe play of objective, including points of review)

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EVALUATION FORM

OBSERVER	CONTROLLER	LOCATION	DATE	REFERENCE PROCEDURES

OBJECTIVE 13: ALERT, NOTIFICATION AND EMERGENCY INFORMATION - PUBLIC INSTRUCTIONS

Demonstrate the ability to coordinate the formulation and dissemination of accurate information to the public in a timely fashion.

NUREG REF	POINTS OF REVIEW	YES	NO	N/A	N/O
E.5	1. Did staff disseminate information to the public after the first alert and notification sequence? If so, specify the times when these disseminations were initiated: #1 _____ #2 _____ #3 _____ #4 _____ #5 _____	___	___	___	___
E.5	2. What organization(s) was responsible for this function? _____ _____ _____				
E.5	3. Did the staff have access to current, accurate and timely information?	___	___	___	___
E.5	4. Describe how staff secured current information on protective action recommendations: _____ _____ _____ _____ _____				

EVALUATION FORM

OBSERVER	CONTROLLER	LOCATION	DATE	REFERENCE PROCEDURES

OBJECTIVE 13: ALERT, NOTIFICATION AND EMERGENCY INFORMATION - PUBLIC INSTRUCTIONS

NUREG REF	POINTS OF REVIEW	YES	NO	N/A	N/O
E.7	5. Were pre-scripted messages used?	___	___	___	___
	If not, describe how messages were developed: _____ _____ _____				
G.4.a	6. Was the formulation and dissemination of the information to the public effectively coordinated by the involved organizations?	___	___	___	___
	If not, describe the nature and extent of the coordination problems: _____ _____ _____				
G.4.a	7. Did staff provide accurate and timely information to the public?	___	___	___	___
G.4	8. Who authorized releases of information to the public? _____				
E.6	9. Was a log maintained of all releases disseminated to the public?	___	___	___	___
	Were copies of all releases kept and accessible to all staff?	___	___	___	___
E.6	10. Did staff monitor radio and television broadcasts to keep track of information provided to the public?	___	___	___	___
G.4	11. Was there a return flow of information to the EOF and EOC's, so that staff at these locations were aware of what information had been given to the media?	___	___	___	___

EVALUATION FORM

OBSERVER	CONTROLLER	LOCATION	DATE	REFERENCE PROCEDURES

OBJECTIVE 13: ALERT, NOTIFICATION AND EMERGENCY INFORMATION - PUBLIC INSTRUCTIONS

NUREG REF	POINTS OF REVIEW	YES	NO	N/A	N/O
G.4	12. Were press releases and results of press interviews exchanged with other spokespersons?	___	___	___	___
	How was information exchanged? _____				
	Did the information management system provide for the effective and timely coordination of the formulation and dissemination of emergency information and instructions?	___	___	___	___
	If not, describe problems:				
E.5 E.6 E.7 G.4 G.4.a G.4.c	13. Were the relevant functions and activities implemented in a manner that is consistent with the organization's emergency plan and procedures?	___	___	___	___
	If not, explain why the organization did not follow its emergency plan and procedures:				

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EVALUATION FORM

OBSERVER	CONTROLLER	LOCATION	DATE	REFERENCE PROCEDURES

EVALUATION: Objective 13 was: _____ met _____ not met

NARRATIVE SUMMARY: (Describe play of objective, including points of review)

EVALUATION FORM

OBSERVER	CONTROLLER	LOCATION	DATE	REFERENCE PROCEDURES

OBJECTIVE 14: ALERT, NOTIFICATION AND EMERGENCY INFORMATION - MEDIA

Demonstrate the ability to brief the media in an accurate, coordinated and timely manner.

NUREG REF	POINTS OF REVIEW	YES	NO	N/A	N/O
G.3.a	1. Did staff provide briefings to the "media"?	___	___	___	___

If so, specify the times when briefings were initiated:

Briefing #1 _____ Briefing #6 _____
Briefing #2 _____ Briefing #7 _____
Briefing #3 _____ Briefing #8 _____
Briefing #4 _____ Briefing #9 _____
Briefing #5 _____ Briefing #10 _____

G.3.a 2. What organization(s) was responsible for this function?

_____
_____
_____

G.3.a 3. How many staff were utilized for this function? _____

G.3.a 4. Did the staff have access to current, accurate and timely information? ___ ___ ___ ___

G.3.a 5. Describe how (source and method) staff secured current information on emergency response:

_____
_____
_____
_____
_____

EVALUATION FORM

OBSERVER	CONTROLLER	LOCATION	DATE	REFERENCE PROCEDURES

OBJECTIVE 14: ALERT, NOTIFICATION AND EMERGENCY INFORMATION - MEDIA

NUREG REF	POINTS OF REVIEW	YES	NO	N/A	N/O
G.3.a	6. Was pre-scripted information used for the media briefings?	___	___	___	___
G.3.a	7. Did staff provide accurate, responsive information to the media in a prompt manner?	___	___	___	___
G.3.a	8. Who authorized release of information to the media? _____				
G.3.a	9. Did staff provide media with accurate information on the protective action recommendations for the public?	___	___	___	___
G.3.a	10. Were all protective action decisions described in terms of familiar landmarks and boundaries for affected planning areas?	___	___	___	___
G.3.a	11. Did staff avoid use of technical jargon?	___	___	___	___
G.3.a	12. Were maps and other displays effectively used?	___	___	___	___
G.3.a	13. Were copies of news releases provided to the "media"?	___	___	___	___
	Were they provided in a timely manner? (Obtain copies of news releases and note time released.)	___	___	___	___
G.3.a	14. Was a log maintained of information provided to the media?	___	___	___	___
	Were copies of all releases kept and accessible to all staff?	___	___	___	___
G.3.a	15. Did staff maintain sufficient contact with decision makers to keep abreast of current events and information?	___	___	___	___

OBSERVER	CONTROLLER	LOCATION	DATE	REFERENCE PROCEDURES

OBJECTIVE 14: ALERT, NOTIFICATION AND EMERGENCY INFORMATION - MEDIA

NUREG REF	POINTS OF REVIEW	YES	NO	N/A	N/O
G.3.a	16. Did staff monitor radio and television broadcasts to keep track of information provided to the public?	___	___	___	___
G.3.a	17. Did staff provide kits to the media?	___	___	___	___
	If so, describe the types of information contained in the media kits: _____ _____ _____				
G.3.a	18. Were the relevant functions and activities implemented in a manner that is consistent with the organization's emergency plan and procedures?	___	___	___	___
	If not, explain why the organization did not follow its emergency plan and procedures: _____ _____ _____ _____ _____				

EVALUATION FORM

OBSERVER	CONTROLLER	LOCATION	DATE	REFERENCE PROCEDURES

EVALUATION: Objective 14 was: _____ met _____ not met

NARRATIVE SUMMARY: (Describe play of objective, including points of review)

EVALUATION FORM

OBSERVER	CONTROLLER	LOCATION	DATE	REFEPENCE PROCEDURES

OBJECTIVE 15: USE OF KI

Demonstrate the ability to make the decision to recommend the use of KI to emergency workers based on predetermined criteria, as well as to distribute and administer it once the decision is made, if necessitated by radioiodine releases.

NUREG REF	POINTS OF REVIEW	YES	NO	N/A	N/O
J.10.e J.10.f	1. Did authorizing official(s) make decision to recommend use of KI?	___	___	___	___
	What official(s) made the decisions? _____				
	When was the decision made? _____				
	When were instructions to use KI transmitted? _____				
	For what group? -Emergency workers?	___	___	___	___
J.10.f	2. Was the decision made to use KI based on projected or actual dose to thyroid? _____ Projected _____ Actual				
	Was the decision to use KI made on the basis of FDA PAG's?	___	___	___	___
J.10.f	3. Were the affected emergency workers/institutions informed of this decision/instructions in a timely manner?				
	For emergency workers? When? _____	___	___	___	___

EVALUATION FORM

OBSERVER	CONTROLLER	LOCATION	DATE	REFERENCE PROCEDURES

OBJECTIVE 15: USE OF KI

NUREG REF	POINTS OF REVIEW	YES	NO	N/A	N/O
J.10.e J.10.f	5. Did personnel responsible for direct administration/use of KI follow instructions given them by authorizing official(s)?				
	For emergency workers?	___	___	___	___
J.10.f	6. Was the KI distributed in sufficient quantities and in a timely manner according to decisions?				
	For emergency workers? When? _____	___	___	___	___
J.10.e J.10.f	7. If the administration (i.e., ingestion) of KI was simulated, was the date, time and dose for the use of KI for each individual recorded?	___	___	___	___
J.10.e J.10.f	8. Were the relevant functions and activities implemented in a manner that is consistent with the organization's emergency plan and procedures?	___	___	___	___
	If not, explain why the organization did not follow its emergency plan and procedures: _____ _____ _____ _____ _____ _____ _____ _____ _____ _____				

EVALUATION FORM

OBSERVER	CONTROLLER	LOCATION	DATE	REFFRENCE PROCEDURES

EVALUATION: Objective 15 was: _____ met _____ not met

NARRATIVE SUMMARY: (Describe play of objective, including points of review)

EVALUATION FORM

OBSERVER	CONTROLLER	LOCATION	DATE	REFERENCE PROCEDURES

OBJECTIVE 16: SUPPLEMENTARY ASSISTANCE (FEDERAL/OTHER)

Demonstrate the ability to identify the need for and call upon Federal and other outside agencies' assistance.

NUREG REF	POINTS OF REVIEW	YES	NO	N/A	N/O
C.1.a	1. Was Federal assistance requested?	___	___	___	___
	If so, did an authorized official make this request?	___	___	___	___
	Identify this official and his/hr organization: _____				
C.1.a	2. Which Federal agency(ies) was contacted to request assistance? _____				
C.1.a C.1.b	3. What Federal assistance was requested and when was it expected to be provided?				

Assistance	When Expected
_____	_____
_____	_____
_____	_____
_____	_____

C.1.c 4. What actions and resources were required by the requesting organization(s) to support the Federal response? _____

EVALUATION FORM

OBSERVER	CONTROLLER	LOCATION	DATE	REFERENCE PROCEDURES

OBJECTIVE 16: SUPPLEMENTARY ASSISTANCE (FEDERAL/OTHER)

NUREG REF	POINTS OF REVIEW	YES	NO	N/A	N/O
	Were these actions taken and resources provided?	___	___	___	___
	If not, why were they not provided?				
C.4	5. Was other assistance and resources requested from non-Federal organizations or individuals?	___	___	___	___
	If so, which organization(s) requested assistance and resources?				
C.4	6. From what non-Federal sources was assistance and resources expected?				
C.4	7. What assistance was requested from non-Federal sources and when was it expected?				

Assistance	When Expected

EVALUATION FORM

OBSERVER	CONTROLLER	LOCATION	DATE	REFERENCE PROCEDURES

OBJECTIVE 16: SUPPLEMENTARY ASSISTANCE (FEDERAL/OTHER)

NUREG REF	POINTS OF REVIEW	YES	NO	N/A	N/O
C.4	8. What actions and resources were required by the requesting organization(s) to support non-Federal response? _____ _____ _____ _____				
	Were these actions taken and resources provided?	___	___	___	___
	If not, why were they not provided? _____ _____ _____ _____				
C.1.b	9. Was appropriate assistance and resources requested from Federal and/or non-Federal sources?	___	___	___	___
	Was this assistance requested in a timely manner?	___	___	___	___
C.1.a C.1.b C.1.c C.4	10. Were the relevant functions and activities implemented in a manner that is consistent with the organization's emergency plan and procedures?	___	___	___	___
	If not, explain why the organization did not follow its emergency plan and procedures: _____ _____ _____ _____				

EVALUATION FORM

OBSERVER	CONTROLLER	LOCATION	DATE	REFERENCE PROCEDURES

EVALUATION: Objective 16 was: _____ met _____ not met

NARRATIVE SUMMARY: (Describe play of objective, including points of review)

EVALUATION FORM

OBSERVER	CONTROLLER	LOCATION	DATE	REFERENCE PROCEDURES

OBJECTIVE 17: EVACUATION OF ONSITE PERSONNEL

Demonstrate the ability to coordinate the evacuation of onsite personnel.

NUREG REF	POINTS OF REVIEW	YES	NO	N/A	N/O
J.2	1. Was evacuation of onsite individuals demonstrated?	___	___	___	___
J.2	2. If so, was it: Promptly activated?	___	___	___	___
	Well organized?	___	___	___	___
J.2	3. Did the licensee contact offsite authorities to coordinate evacuation of onsite individuals?	___	___	___	___
J.2	4. Was an evacuation route specified?	___	___	___	___
J.2	5. Was monitoring and decontamination provided for, if appropriate?	___	___	___	___
	Were the relevant functions and activities implemented in a manner that is consistent with the organization's emergency plan and procedures?	___	___	___	___
	If not, explain why the organization did not follow its emergency plan and procedures:				

_____

_____

_____

_____

_____

EVALUATION FORM

OBSERVER	CONTROLLER	LOCATION	DATE	REFERENCE PROCEDURES

EVALUATION: Objective 17 was: _____ met _____ not met

NARRATIVE SUMMARY: (Describe play of objective, including points of review)

17.2

EVALUATION FORM

OBSERVER	CONTROLLER	LOCATION	DATE	REFERENCE PROCEDURES

OBJECTIVE 18: OFFSITE RESPONSE ORGANIZATIONS

Demonstrate the capability of offsite response organization personnel to interface with through their mobilization and provision of advice and assistance.

NUREG REF	POINTS OF REVIEW	YES	NO	N/A	N/O
	1. Identify utility offsite response organization liaisons contacted.				

Liaison	Time Notified

C.5 N.1.b	2. At what time did the liaisons arrive at their assigned duty station?				

Liaison	Time of Arrival

C.5 N.1.b	3. Did liaisons provide advice and information to the control cell?	___	___	___	___

If yes, indicate the nature of the advice and information provided.

Nature of Advice/Assistance

OBSERVER	CONTROLLER	LOCATION	DATE	REFERENCE PROCEDURES

OBJECTIVE 18: UTILITY OFFSITE RESPONSE ORGANIZATIONS

NUREG REF	POINTS OF REVIEW	YES	NO	N/A	N/O
C.5	4. Did the liaisons provide accurate and timely information, if appropriate?	___	___	___	___
C.5 N.1.b	5. Were the relevant functions and activities implemented in a manner that is consistent with the organization's emergency plan and procedures?	___	___	___	___

If not, explain why the organization did not follow its emergency plan and procedures:

_____
_____
_____
_____
_____

EVALUATION FORM

OBSERVER	CONTROLLER	LOCATION	DATE	REFERENCE PROCEDURES

EVALUATION: Objective 18 was: _____ met _____ not met

NARRATIVE SUMMARY: (Describe play of objective, including points of review)

FORM SAP 6.3

EMERGENCY PLAN CORRECTIVE ACTION REPORT

Prepared By: _____ Date: _____
Name/Title

Approved: _____
Superintendent of Power

Individual Assigned to Correct Deficiency _____
Name/Title

Reviewed: _____
PORC Date

TO BE COMPLETED BY INDIVIDUAL PREPARING FORM

ITEM DEFICIENCY-

DESCRIPTION AND/OR CAUSE-

RECOMMENDED CORRECTIVE ACTION-

CORRECTIVE ACTION TO BE COMPLETED BY _____
DATE

TO BE COMPLETED BY INDIVIDUAL ASSIGNED TO CORRECT DEFICIENCY

RESOLUTION- (Include Applicable Work Authorization Numbers or Purchase Authorizations)

SIGNATURE: _____
DATE: _____

PLEASE RETURN COMPLETED FORM TO THE JAFNPP EMERGENCY PLANNING COORDINATOR.

FORM SAP 6.4

EMERGENCY PLAN CORRECTIVE ACTION LOG

NO.	DEFICIENCY (Include references to forms, memos, etc.)	SCHEDULED COMPLETION DATE	PERSON ASSIGNED	ACTION TAKEN	DATE EVALUATED

EMERGENCY PLANNING COORDINATOR _____ DATE REVIEWED _____