# LICENSEE EVENT REPORT

EIGENGEE EVENT HET ON T
CONTROL BLOCK: (PLEASE PRINT OR TYPE ALL REQUIRED INFORMATION)
0 1 1 L Q A D 2 2 0 0 0 - 0 0 0 - 0 0 3 4 1 1 1 1 4 5 5  7 8 9 LICENSEE CODE 14 15 LICENSE NUMBER 25 26 LICENSE TYPE 30 57 CAT 58
CON'T    O   1   SOURCE   C   O   O   O   O   O   O   O   O   O
EVENT DESCRIPTION AND PROBABLE CONSEQUENCES (10)  [0   2   On October 1, 1932, at 1600 hours, a small leak was discovered in the High Pressure ]
Coolant Injection System (HPCI) supply steam line break flange. A reduction in power
0 4 was immediately started in preparation for a shutdown as required in Technical
Specification 3.5.C.3. The flange gasket was replaced and HPCI was satisfactorily tested and declared operable. The flange still leaked slightly, and further repairs were deferred until a scheduled outage on October 8, 1982. When a leak was detected upon startup from the outage, all required Emergency Core Cooling Systems were tested as per Technical Specification 3.5.C.2. Following flange adjustments, HPCI was successfully tested and declared operable. Leakage was contained to the Reactor Building drain system and did not affect operation of the HPCI System.  SYSTEM CAUSE CAUSE  COMP VALVE
CODE SUBCODE S
The leak was a result of a failed flange gasket. The gasket was replaced like-for-
like, both during and prior to the outage. Subsequently, leakage was effectively
1 2 stopped with flange bolt adjustments.
13
14
7 8 9 FACILITY STATUS SPOWER OTHER STATUS 30 METHOD OF DISCOVERY DESCRIPTION 32  1 5 7 28 0 10 12 13
ACTIVITY CONTENT RELEASED OF RELEASE AMOUNT OF ACTIVITY (35) NA LOCATION OF RELEASE 36  NA LOCATION OF RELEASE 36  NA LOCATION OF RELEASE 36
PERSONNEL EXPOSURES NUMBER TYPE DESCRIPTION 39  1 7 0 0 0 0 37 Z 38 NIA  7 8 9 PERSONNEL INJURIES NUMBER DESCRIPTION 41
1 8 0 0 0 0 40 NA
LOSS OF OR DAMAGE TO FACILITY 43  TYPE DESCRIPTION  19 Z 42 8211020072 821021 PDR ADDCK 05000265 PDR ADDCK 05000265
ISSUED DESCRIPTION (45) 5
D. Cook  NAME OF PREPARER  D. Cook  PHONE:  9 309-654-2241, ext 178

I. LER NUMBER: LER 82-17/03L-0

II. LICENSEE NAME: Commonwealth Edison Company

Quad-Cities Nuclear Power Station

III. FACILITY NAME: Unit Two

IV. DOCKET NUMBER: 050-265

#### V. EVENT DESCRIPTION:

On October 1, 1982, at 1600 hours, Unit Two was at 630 MWe when a Shift Foreman, on a routine surveillance, found a steam leak in the High Pressure Coolant Injection (HPCI) System. The leak was determined to be coming from the break flange just upstream from adjoining line 2-2339. At 1615 hours, the HPCI System was taken out of service to repair the flange. An orderly shutdown of the unit was deemed appropriate as stated in Technical Specification 3.5.C.3. The gasket on the flange was then replaced by the Mechanical Maintenance Department. At 0300 hours, on October 2, 1982, HPCI was tested and found operable as per Technical Specification 4.5.C.1. The flange was inspected and a small leak was discovered. Since this small leak would not affect the proper operation of the HPCI System, a decision was made to defer further maintenance on the flange until an upcoming scheduled outage on October 8, 1982. During this outage, HPCI was removed from service to facilitate additional flange repairs. On October 12, 1982, at 1630 hours, following the maintenance outage, unit startup commenced. Once sufficient pressure was obtained, the flange was again checked and was found to still have a slight leak. The system was isolated and declared inoperable. Testing of the required Emergency Core Cooling Systems, as stated in Technical Specification 3.5.C.2, was initiated, and by 0230 hours, on October 13, 1982, all systems were found to be operable. At 0443 hours, the flange was misting slightly and the leakage was effectively stopped. HPCI was tested, as per Technical Specification 4.5.C.1, and at 1830 hours was declared operable and returned to service.

#### VI. PROBABLE CONSEQUENCES OF THE OCCURRENCE:

The flange leak was isolatable at all times via the HPCI steam isolation valves, and all leakage was confined to the Reactor Building. The original steam leakage may have been sufficient, after prolonged operation, to cause an isolation of the HPCI System on a high HPCI area temperature signal. Subsequent leakage was not significant and would have had no affect on HPCI System operation.

This occurrence did not jeopardize the safe operation of the Reactor. The Technical Specifications were not exceeded, as either an orderly shutdown of the Reactor was initiated or the operability of all the required Emergency Core Cooling Systems were confirmed, as per Technical Specification 3.5.C.2.

### VII. CAUSE:

The original leak was caused by normal wear of the flange gasket material. Subsequent leakage was a result of insufficient adjustment of the flange bolts.

## VIII. CORRECTIVE ACTION:

The gasket was replaced, like-for-like, both prior to, and during, the scheduled outage. Subsequent repairs consisted of adjusting the flange bolts.