123 Main Street
White Plains, New York 10601
914 681.6240

NewYork Power Authority

John C. Brons
Executive Vice President
Nuclear Generation

December 19, 1990
JPN-90-075

U.S. Nuclear Regulatory Commission
Attn: Document Control Desk
Mail Stop P1-137
Washington, DC 20555

Subject: James A. FitzPatrick Nuclear Power Plant
Docket No. 50-333
In-Service Inspection Summary Report
Spring 1990 Outage (Reload 9/Cycle 10)

References: 1. NYPA letter, J.C. Brons to NRC, dated May 25, 1990 (JPN-90-040), "Reactor Pressure Vessel Head Flaw Indication Inspections and Evaluation Analyses."

Dear Sir:

ASME Section XI requires submittal of an outage report to the NRC. The attached reports detail the FitzPatrick plant In-Service Inspections (ISI) performed during the Spring 1990 outage. This submittal contains the following three reports:

1. "James A. FitzPatrick Nuclear Power Plant Inservice Inspection of Components Spring 1990," Volumes I thru VI. This report includes:

   - Intergranular stress corrosion cracking inspection data. A total of 76 welds were inspected, 8 welds required weld overlays where 7 overlays were for new IGSCC indications, and 1 was for a preexisting indication;
   - ISI weld and support inspection data as required by ASME Section XI; and
   - Erosion/corrosion data for inspections as required by IE Bulletin 87-01, "Thinning of Pipe Walls in Nuclear Power Plants" and Generic Letter 89-13, "Service Water System Problems Affecting Safety-Related Equipment."

A047
1/0

2. "James A. FitzPatrick Nuclear Power Plant Reactor Pressure Vessel Inservice Inspection Spring 1990," Volume I. This report includes:

   - Reactor pressure vessel weld inspection data as required by ASME Section XI;
   - The results of the main steam nozzle N-3C examination, which were performed using the GERIS system.

   This report also documents the recent scheduled ISI examination during which the Authority identified flaws in the reactor pressure vessel head. The flaws were evaluated using fracture mechanics, as recommended in IWB-3122.4 and found to be acceptable for the continued operation of the plant until the next cycle. In Reference 1, the Authority reported the results from the analysis to the NRC as required by IWB-3125. In lieu of the requirements of IWB-2420(b), "Reexamination of flaws qualified as acceptable for continued service," the Authority will inspect these flaws during each of the next three refueling outages.

3. "James A. FitzPatrick Nuclear Power Plant Inservice Inspection of In-Vessel Components Spring 1990," Volume I. This report consists of:

   - Reactor in-vessel inspection data as required by ASME Section XI. This portion of the report describes the inspections requested by IE Bulletins 80-07, "Jet Pump Assembly Failure" and 80-13, "Cracking in Core Spray Spargers."

If you have any questions, please contact Mr. J. A. Gray, Jr.

Very truly yours,

John C. Brons
Executive Vice President
Nuclear Generation

cc: see next page

cc: Regional Administrator
U. S. Nuclear Regulatory Commission
475 Allendale Road
King of Prussia, PA 19406

Office of the Resident Inspector
U. S. Nuclear Regulatory Commission
P. O. Box 136
Lycoming, NY 13093

Mr. David E. LaBarge
Project Directorate I-1
Division of Reactor Projects -I/II
U. S. Nuclear Regulatory Commission
Mail Stop 14 B2
Washington, DC 20555

Mr. Charles Lazart
Hartford Steam Boiler and Insurance Co.
Hartford Branch Office
One State Street
Hartford, Connecticut 06101-9990