

UNITED STATES NUCLEAR REGULATORY COMMISSION

NORTHERN STATES POWER COMPANY

PRAIRIE ISLAND NUCLEAR GENERATING PLANT

Docket No. 50-282
50-306

REQUEST FOR AMENDMENT TO
OPERATING LICENSE NOS. DPR-42 & DPR-60

(License Amendment Request Dated October 29, 1982)

Northern States Power Company, a Minnesota corporation, requests authorization for changes to the Technical Specifications as shown on the attachments labeled Exhibit A and Exhibit B. Exhibit A describes the proposed changes along with reasons for the change. Exhibit B is a set of Technical Specification pages incorporating the proposed changes.

This letter contains no restricted or other defense information.

NORTHERN STATES POWER COMPANY

By David Musolf
David Musolf
Manager - Nuclear Support Services

On this 29th day of October, 1982 before me a notary public in and for said County, personally appeared David Musolf, Manager - Nuclear Support Services, and being first duly sworn acknowledged that he is authorized to execute this document on behalf of Northern States Power Company, that he knows the contents thereof and that to the best of his knowledge, information and belief, the statements made in it are true and that it is not interposed for delay.

Betty J. Dean



8211020005 821029
PDR ADOCK 05000282
P PDR

EXHIBIT A

Prairie Island Nuclear Generating Plant

License Amendment Request Dated October 29, 1982

Proposed Changes to the Technical Specifications
Appendix A of Operating Licenses DPR-42 and DPR-60

Pursuant to 10 CFR 50.59 and 50.90, the holders of operating licenses DPR-42 and DPR-60 hereby propose the following changes to Appendix A, Technical Specifications:

1. Specification: Containment System TS 3.6 & Table TS.4.4-1

PROPOSED CHANGE

The Technical Specifications on page TS 3.6-2 and Table TS.4.4-1 are to be revised as noted in Exhibit B.

REASON FOR CHANGE

These changes revise the Technical Specifications as required by the NRC Staff Safety Evaluation Report dated September 9, 1982, "Completion of Containment Purge Generic Issue B-24 NUREG-0737 Item II.E.4.2 - Prairie Island Unit Nos. 1 and 2."

Double gasketed blind flanges will be installed on both the normal and the inservice purge systems to eliminate the potential for valve leakage during normal operation. Debris screens are being installed on the inservice purge system penetrations. This system may be used during plant operation following verification of valve leak tightness. The system will be used only when required for safe plant operation.

SAFETY EVALUATION

Refer to the NRC Staff Safety Evaluation Report dated September 9, 1982.

2. Specification: Refueling and Fuel Handling TS 3.8

PROPOSED CHANGE

An explanation is added to the basis to clarify the intent of Specification 3.8.A.7. The words "when the cavity is being drained for head replacement" were deleted from the specification. See Exhibit B.

The typographical error in the word "head" was corrected.

The basis used the words "adsorber" and "absorber" interchangeably in discussing charcoal efficiency testing. All were changed to adsorber for accuracy.

REASON FOR CHANGE

A clarification was needed to remove confusion in interpreting TS.3.8.A.(7).

Typographical errors are corrected.

SAFETY EVALUATION

None required.

3. Specification: TS 3.12 Snubbers

PROPOSED CHANGE

Additions, deletions, and corrections to table TS.3.12-1 Safety Related Snubbers.

REASON FOR CHANGE

To maintain the table current as modifications are completed and to correct errors.

SAFETY EVALUATION

Any snubber addition or removal is evaluated in accordance with the design change and Work Request Authorization processes.

4. Specification: TS 3.15 Event Monitoring Instrumentation

PROPOSED CHANGE

Changes to TS.3.15, Table TS.4.1-1 and the addition of a new table TS.3.15-2 for radiation event monitoring instrumentation. See Exhibit B.

REASON FOR CHANGE

This change provides limiting conditions for operation and surveillance requirements for plant modifications implementing NUREG-0737 Items II.F.1.1 and II.F.1.2.

SAFETY EVALUATION

These radiation monitors assist the operator to assess the extent of an accident.

5. Specification: TS 4.5 Engineered Safety Feature

PROPOSED CHANGE

Revise TS 4.5.A.5(b) to read, "At least once every 18 months, subject each diesel engine to a thorough inspection in accordance with procedures prepared in conjunction with the manufacturer's recommendations for this class of standby service".

REASON FOR CHANGE

To clarify the inspection interval. Prairie Island plant has two refuelings each year and the existing wording does not identify which refueling outage requires the engine inspection.

SAFETY EVALUATION

The philosophy which applies to diesel generator inspection also applies to the inspections of the diesel cooling water engine. The 18-month inspection interval is adequate to insure these engines are well maintained without being out of service unnecessarily. The surveillances being performed to satisfy TS requirements 4.5.A.5(a) and 4.5.B.1.(b) give early indication of problems with the diesel engines.