NRC FORM 366 (7-77)

U. S. NUCLEAR REGULATORY COMMISSION

LICENSEE EVENT REPORT

CONTROL BLOCK: | | | | | | | (1) (PLEASE PRINT OR TYPE ALL REQUIRED INFORMATION)

0 1	LICENSEE CODE: M E M Y P 1 (2) LICENSE NUMBER: 0 0 - 0 0 0 0 0 - 0 0 (3) LICENSE TYPE: 4 1 1 1 1 (4) CAT: (5)
CON'T 0 1	REPORT SOURCE: L (6) DOCKET NUMBER: 0 5 0 0 0 3 0 9 (7) EVENT DATE: 0 9 2 4 8 2 (8) REPORT DATE: 1 0 2 1 8 2 (9)

EVENT DESCRIPTION AND PROBABLE CONSEQUENCES (10)

0 2 While in hot shutdown, immediately prior to refueling outage, scheduled inservice
0 3 inspection of shock suppressors required to be operable by Technical Specifications
0 4 revealed an empty hydraulic reservoir on snubber number 72, SHP-HSS-110, making it
0 5 inoperable. The snubber is located on the number three main steam line immediately
0 6 downstream of the excess flow check valve and is primarily required to accomodate
0 7 pipe stresses associated with plant trip and excess flow check valve closure at
0 8 power. The shock suppressor is included in the piping seismic analysis. There is

	SYSTEM CODE	CAUSE CODE	CAUSE SUBCODE	COMPONENT CODE	COMP. SUBCODE	VALVE SUBCODE
0 9	H B (11)	E (12)	F (13)	S U P O R T (14)	D (15)	Z (16)

(17) LER/RO REPORT NUMBER	EVENT YEAR		SEQUENTIAL REPORT NO.		OCCURRENCE CODE	REPORT TYPE		REVISION NO.
	8 2	—	0 3 1	/	0 3	L	—	0

ACTION TAKEN	FUTURE ACTION	EFFECT ON PLANT	SHUTDOWN METHOD	HOURS (22)	ATTACHMENT SUBMITTED	NPRD-4 FORM SUB.	PRIME COMP. SUPPLIER	COMPONENT MANUFACTURER
A (18)	X (19)	Z (20)	Z (21)	0 0 0 0	Y (23)	Y (24)	A (25)	T 2 4 4 (26)

CAUSE DESCRIPTION AND CORRECTIVE ACTIONS (27)

1 0 The empty hydraulic fluid reservoir of the Tomkins-Johnson model number SH-9B six
1 1 inch snubber resulted primarily from leakage thru the ethylene propylene cylinder
1 2 seals and minor seepage around snubber valve fittings. The degree of seal wear is
1 3 not considered unusual since this snubber has not been rebuilt since 1976. The
1 4 minor seepage is an expected result of inservice conditions. The snubber has been

	FACILITY STATUS	% POWER	OTHER STATUS (30)	METHOD OF DISCOVERY	DISCOVERY DESCRIPTION (32)
1 5	G (28)	0 0 0 (29)	NA	B (31)	Inservice Inspection

	ACTIVITY RELEASED	CONTENT OF RELEASE	AMOUNT OF ACTIVITY (35)	LOCATION OF RELEASE (36)
1 6	Z (33)	Z (34)	NA	NA

PERSONNEL EXPOSURES

	NUMBER	TYPE	DESCRIPTION (39)
1 7	0 0 0 (37)	Z (38)	NA

PERSONNEL INJURIES

	NUMBER	DESCRIPTION (41)
1 8	0 0 0 (40)	NA

LOSS OF OR DAMAGE TO FACILITY (43)

	TYPE	DESCRIPTION
1 9	Z (42)	NA

PUBLICITY

	ISSUED	DESCRIPTION (45)	NRC USE ONLY
2 0	N (44)		

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PDR ADOCK 05000309
S PDR

NAME OF PREPARER Robert H. Nelson PHONE: 207-882-6321

EVENT DESCRIPTION AND PROBABLE CONSEQUENCES (Cont'd)

reasonable assurance that an assumed failure of this snubber at power would not have placed the plant in an unanalyzed condition as a result of a transient affecting number three main steam line. There was no impact on the health and safety of the public.

CAUSE DESCRIPTION AND CORRECTIVE ACTIONS (Cont'd)

rebuilt with new seals. Under the inservice testing program the shock suppressor was originally scheduled to be rebuilt during the shutdown. Other snubbers are also being rebuilt during this outage under the inservice testing program due to service history. The snubber surveillance inspection frequency has been increased in accordance with Technical Specifications as a result of this discrepancy.