Maine Yankee
RELIABLE ELECTRICITY SINCE 1972

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May 11, 1994
MN-94-45 JRH-94-106

UNITED STATES NUCLEAR REGULATORY COMMISSION
Attention: Document Control Desk
Washington, DC 20555

References: (a) License No. DPR-36 (Docket No. 50-309)
(b) MYAPCo Letter to USNRC dated March 11, 1981 (FMY-81-33)
(c) MYAPCo Letter to USNRC dated April 30, 1993 (MN-93-46)

Subject: Annual Report of Facility Changes and Relief and Safety Valve Failures and Challenges

Gentlemen:

In accordance with 10 CFR 50.59, Attachment "A" to this letter is a report containing a brief description of the facility changes completed at the Maine Yankee Atomic Power Station during 1993 and a summary of the safety evaluation for the change.

Attachment "B" to this letter is a report containing a brief description of the active temporary plant changes (yellow tags) during 1993 and a summary of the safety evaluation for the change.

In Reference (b), Maine Yankee committed to reporting any challenges and/or failures of PORV and pressurizer safety valves. During 1993 there were no such events.

Very truly yours,

James R. Hebert

James R. Hebert, Manager
Licensing & Engineering Support Department

JVW/mwf

Attachment

c: Mr. Thomas T. Martin
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ATTACHMENT A

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SUMMARY OF DESIGN CHANGES COMPLETED IN 1993

EDCR NUMBER	DESCRIPTION
E90-039	TURBINE GENERATOR VIBRATION MONITORING SYSTEM
E90-070	ABANDONMENT OF FERROUS SULFATE TANK (TK-33)
E91-027	LETDOWN SYSTEM VALVE (LD-M-2) REPLACEMENT
E91-047	CHARGING PUMP SYSTEM LOGIC & CIRCUITRY MOD (ANTI PUMP CIRCUITRY)
E91-051	REVENUE METERING UPGRADE/SCADA
E91-055	BLOWDOWN CONTROL VALVE & TANK NOZZLE MODS/FILTER UPGRADE
E91-085	PRIMARY COMPONENT COOLING/SECONDARY COMPONENT COOLING (PCC/SCC) AND RHR TEMPERATURE UPGRADE
E91-093	DRY LAY-UP OF CONDENSATE AND FEEDWATER SYSTEMS
E91-097	MAIN TRANSFORMER REPLACEMENT
E91-411	ON-LINE CHEMISTRY MONITORING
E92-003	1ST POINT HEATER LEVEL CONTROL UPGRADE
E92-028	CONTAINMENT CONTROL AIR (CCA) COMPRESSOR UPGRADES (C-5A, B)
E92-033	UNIT STATION SERVICE TRANSFORMER (X-24) REPLACEMENT
E92-041-1	NUCLEAR INSTRUMENTATION CHANNEL UPGRADE - WIDE RANGE LOG CHANNELS
E92-055	ON-LINE CHEMISTRY MONITORING PHASE II
E92-056	SECURITY SYSTEMS - ALLEY WAY ALARMS MODS
E92-075	REACTOR COOLANT PUMP (RCP) LUBE OIL SIGHT GLASS ACCESS UPGRADES
E92-076	ANNUBAR FOR SERVICE WATER
E92-094	DEAERATION OF CONDENSER SURGE TANK TK-122
E92-099	LINING SERVICE WATER PIPING
E92-2002	NEUTRON NOISE MONITORING SYSTEM
E92-2004	CIRCULATING WATER BOX PRESSURE GAGE TUBING REMOVAL
E93-004	REACTOR PROTECTION SYSTEM (RPS) ACOPIAN POWER SUPPLY ZENER DIODE ADDITION
E93-031	REPLACEMENT SERVICE AND INSTRUMENT AIR COMPRESSORS (C-1A,B,S)

SUMMARY OF DESIGN CHANGES COMPLETED IN 1993

EDCR NUMBER	DESCRIPTION
E93-2001	DEMINERALIZED WATER STORAGE TANK (DWST) TUBING MODIFICATION
E93-2002	INSTALLATION OF FUSES ON CONDENSATE PUMP CONTROL AND INDICATION POWER SUPPLIES (X-1300A,B,C)
E93-2003	RESIDUAL HEAT REMOVAL VALVES (RH-M-1 AND RH-M-2) CABLE REPLACEMENT
E93-2004	FEEDWATER LINE NO.3 SHOCK SUPPRESSOR REDUCTION
E93-2006	REFUELING PURIFICATION PUMP (P-8) LOW FLOW SWITCH INSTALLATION
E93-2007	INSTALLATION OF TEST PORTS FOR VENTILATION FLOW MEASUREMENT

EDCR E90-039

TURBINE GENERATOR VIBRATION MONITORING SYSTEM

This design change replaced the existing vibration monitors on bearings #4 and #5 of the turbine generator with state-of-the-art proximity probes. The original monitors were transducers which were in direct contact with the rotating shaft.

The probability of occurrence of an accident previously evaluated in the FSAR was not increased. This design change did not involve an unreviewed safety question as defined in 10CFR50.59.

EDCR E90-070

ABANDONMENT OF FERROUS SULFATE TANK (TK-33)

This design change incorporated the proper statutory abandonment of the ferrous sulfate tank (TK-33) and the installation of a monitoring well as required by state law.

The probability of occurrence of an accident previously evaluated in the FSAR was not increased. This change did not involve an unreviewed safety question as defined in 10CFR50.59.

EDCR E91-027

LETDOWN SYSTEM VALVE (LD-M-2) REPLACEMENT

This design change replaced LD-M-2 to comply with NRC Generic Letter 89-10. The new LD-M-2 was tested to demonstrate operability under normal and maximum service conditions.

The probability of occurrence of an accident previously evaluated in the FSAR was not increased. This design change did not involve an unreviewed safety question as defined in 10CFR50.59.

E91-047

CHARGING PUMP SYSTEM LOGIC & CIRCUITRY MOD (ANTI PUMP CIRCUITRY)

This design change was implemented to correct an operational concern by equipping the 4,160 V breakers with an anti-pump feature that is designed to protect the breaker and pump motor in the event of simultaneous and constant start/stop signals. This design feature was determined to also inhibit the auto-start of the chemical volume and control system (CVCS) charging pump that is in the stand-by mode under certain accident conditions.

This modification will result in the permanent elimination of the CVCS high pressurizer level and high discharge pressure and high pressurizer level auto trips of the standby charging pump.

The probability of occurrence of an accident previously evaluated in the FSAR was not increased. This design change did not involve an unreviewed safety question as defined in 10CFR50.59.

EDCR E91-051

REVENUE METERING UPGRADE/SCADA

This design change was implemented to upgrade the existing revenue metering equipment. The obsolete revenue metering equipment was removed from the electrical control board and replaced with state-of-the-art equipment. Supervisory control and data acquisition (SCADA) equipment was installed on the 115KV system to allow Central Maine Power (CMP) direct access to Maine Yankee (MY) equipment status.

This upgrade provided greater serviceability of the equipment and brought Maine Yankee in-line with the remainder of the generation facilities within the CMP system from a revenue metering equipment standpoint.

The probability of occurrence of an accident previously evaluated in the FSAR was not increased. This design change did not involve an unreviewed safety question as defined in 10CFR50.59.

EDCR E91-055

BLOWDOWN CONTROL VALVE & TANK NOZZLE MODS/FILTER UPGRADE

The steam generator blowdown flow was restricted such that the maximum flow per steam generator was approximately 45 gpm. These flow restrictions resulted in significant time delays in the steam generator cleanup coming out of plant shutdowns (on the order of two days cleanup time). Modifications made per this design change increased the steam generator blowdown capacity by replacing the existing blowdown flow control valves and exit nozzles with larger, less flow restrictive valves and nozzles. A new auxiliary pump was installed per this design change to accommodate the increased flow rate through the blowdown valves.

The elimination of the flow restrictions via this modification should reduce the cleanup time necessary to achieve desired steam generator cleanliness.

This design change had administrative controls added to ensure that the total steam generator blowdown flow would not exceed any design basis assumed in accidents previously evaluated in the FSAR. This modification did not involve an unreviewed safety question as described in 10CFR50.59.

EDCR E91-085

PRIMARY/SECONDARY COMPONENT COOLING (PCC/SCC) AND RHR TEMPERATURE UPGRADE

This design change brought the existing primary component cooling (PCC), secondary component cooling (SCC), and residual heat removal (RHR) outlet temperature loops into compliance with Regulation Guide 1.97. Two new temperature loops were added which provides main control board (MCB) indication of the containment sump temperature during recirculation. This design change included the addition of new inlet temperature indication for the RHR heat exchangers, which in addition to providing containment sump temperature, could be used in conjunction with the existing outlet temperature loops to provide indication of heat sink. During cool down and cool shutdown, operators could use either RHR trains A or B and have current indication of RHR inlet temperature.

The probability of occurrence of an accident previously evaluated in the FSAR was not increased. This design change did not involve an unreviewed safety question as defined in 10CFR50.59.

EDCR E91-093

DRY LAY-UP OF CONDENSATE AND FEEDWATER SYSTEMS

This design change installed five permanent dehumidifiers in the turbine hall, one electrical outlet for a fan, and a one-inch diameter drain valve in a section of condensate piping which did not have a low point drain. The dehumidifiers and the fan will be used to dry out carbon steel secondary plant components and piping during extended outages. This will reduce the corrosion of the carbon steel components and transport of corrosion products to the steam generators, which in turn retards the steam generator pressure loss degradation rate.

The probability of occurrence of an accident previously evaluated in the FSAR was not increased. This installation did not involve an unreviewed safety question as defined in 10CFR50.59.

EDCR E91-097

MAIN TRANSFORMER REPLACEMENT

This design change replaced the existing generator step-up (GSU) transformers X-1S and X-1B with new transformers. The new transformers are equipped with a "conservator" instead of the "nitrogen blanket" type oil preservation system. The original GSU transformers required thirteen coolers for normal operation, whereas the new GSU transformers only require two coolers because they are much more efficient. Design weaknesses of the original GSU transformers were eliminated and state-of-the-art design features and oil monitoring systems were incorporated into the design and construction of the new GSU transformers.

The probability of occurrence of an accident previously evaluated in the FSAR was not increased. This design change did not involve an unreviewed safety question as defined in 10CFR50.59.

EDCR E91-411

ON-LINE CHEMISTRY MONITORING

This design change upgraded the secondary system chemistry monitoring capabilities. The original monitoring system provided indications of major changes of the two most probable contaminants, chloride and oxygen, but did not provide the capability to reliably and continuously monitor intrusions of other potential contaminants.

This design change consisted of an improved dissolved oxygen monitor and new pH monitor for the condensate system, and cation conductivity monitors for the steam generator blowdown and at the effluent of the blowdown demineralizer. Each monitor has both an analyzer and a recorder to provide both instantaneous monitoring and long term trending. A multipoint alarm was also installed to identify when monitors are out of range.

The probability of occurrence of an accident previously evaluated in the FSAR was not increased. This design change did not involve an unreviewed safety question as defined in 10CFR50.59.

EDCR E92-003

1ST POINT HEATER LEVEL CONTROL UPGRADE

This design change provided the instrumentation necessary to upgrade the first point heater level control system on E-11A & B. Locally mounted differential pressure transmitters were installed which feed their signals to the Foxboro IA system, replacing the level controller and switch functions. The existing level control valves were fitted with E/P transducers which convert the electrical signal sent from the Foxboro IA system to a pneumatic signal to the valve.

The probability of occurrence of an accident previously evaluated in the FSAR was not increased. This design change did not involve an unreviewed safety question as defined in 10CFR50.59.

EDCR E92-028

CONTAINMENT CONTROL AIR (CCA) COMPRESSOR UPGRADES (C-5A, B)

This design change upgraded the operation of the containment control air compressors (C-5A and C-5B). The original systems operating mode was to have one compressor running and loading on demand and the other compressor on standby. The running compressor was manually alternated every week. This design change allows both compressors to run continuously. An alternating circuit was also added to the unloader valve circuit causing the compressor loading to cycle between both compressors.

The probability of occurrence of an accident previously evaluated in the FSAR was not increased. This design change did not involve an unreviewed safety question as defined in 10CFR50.59.

E92-033

UNIT STATION SERVICE TRANSFORMER (X-24) REPLACEMENT

This design change replaced the unit station service transformer in order to lower the maximum available fault current to within the interrupting ratings of the switchgear breakers on buses 3 and 4, and emergency buses 5 and 6. This change resolved the NRC concern on the safety related 4.16KV circuit breakers having a marginal interrupting capability when fed from unit station service transformer X-24.

The probability of occurrence of an accident previously evaluated in the FSAR was not increased. This design change did not involve an unreviewed safety question as defined in 10CFR50.59.

EDCR E92-041-1

NUCLEAR INSTRUMENTATION CHANNEL UPGRADE - WIDE RANGE LOG CHANNELS

This design change replaced the wide range logarithmic channel portion of the excore nuclear instrumentation system. The new wide range nuclear instrument (WRNI) channel improved overall reliability, provided more accurate and user-friendly displays to operations personnel, reduced maintenance and calibration time and cost, and used equipment for which spare parts are readily available.

The probability of occurrence of an accident previously evaluated in the FSAR was not increased. This design change did not involve an unreviewed safety question as defined on 10CFR50.59.

EDCR E92-055

ON-LINE CHEMISTRY MONITORING PHASE II

This design change upgraded the Larson Lane conductivity recorder (CR-2201) with a more reliable means of obtaining conductivity data. The upgrade allowed for the monitoring of the three main steam, the four condenser waterboxes, and the four hotwell conductivity points that are currently fed into CR-2201. This design change also added dissolved oxygen and conductivity monitoring to the feedwater system.

The probability of occurrence of an accident previously evaluated in the FSAR was not increased. This design change did not involve an unreviewed safety question as defined in 10CFR50.59.

EDCR E92-056

SECURITY SYSTEMS - ALLEY WAY ALARMS MODS

This design change installed an alarm system on the existing fence between the southwest corner of the service building and containment air handling rooms. This change will eliminate the possibility of an "uncompensated degradation in a security system protected area in an access controlled area," and ensures prompt controlled access to vital equipment areas and accommodates the rapid ingress and egress to all areas in case of an emergency.

The probability of occurrence of an accident previously evaluated in the FSAR was not increased. This design change did not constitute an unreviewed safety question as defined in 10CFR50.59.

EDCR E92-075

REACTOR COOLANT PUMP (RCP) LUBE OIL SIGHT GLASS ACCESS UPGRADES

This design change installed permanent ladders to allow safer personnel access to the lower lubrication oil reservoir sight glass on each reactor coolant pump. The ladders extend up from existing platforms around each reactor coolant pump up to the sight glass area.

The probability of occurrence of an accident previously evaluated in the FSAR was not increased. This design change did not pose an unreviewed safety question as defined in 10CFR50.59.

EDCR E92-076

ANNUBAR FOR SERVICE WATER

This design change installed an annubar flow meter on each of the service water (SW) supply lines and constructed a pit beneath the turbine hall floor for access to the underground cast iron headers. The addition of the annubar flow meter system allows the service water system flow to be accurately measured for pump and valve testing, and to monitor heat exchanger performance.

The probability of occurrence of an accident previously evaluated in the FSAR was not increased. This design change did not involve an unreviewed safety question as defined in 10CFR50.59.

EDCR E92-094

DEAERATION OF CONDENSER SURGE TANK TK-122

This design change installed a fiberglass composite, floating cover in the condensate surge tank (CST). The floating cover prevents the introduction of oxygen into the water from the open tank vent by maintaining an impermeable barrier between the atmosphere and the makeup water in the tank. Other modifications to existing lines were made for the future addition of a hydrazine injection/recirculation system. This modification provided the means to reduce and maintain the oxygen concentration to less than the 100 ppb upper limit recommended by EPRI.

The probability of occurrence of an accident previously evaluated in the FSAR was not increased. This design change did not involve an unreviewed safety question as defined in 10CFR50.59.

EDCR E92-099

LINING SERVICE WATER PIPING

This modification installed an "insituform" liner in the 24" cast iron intake service water pipes running from the pump house to the service water headers at the service water heat exchangers (E-4 and 5). Insituform liners provide a corrosion resistant liner for the cast iron pipe which prevents additional degradation of the cast iron by corrosion.

The probability of occurrence of an accident previously evaluated in the FSAR was not increased. This modification did not involve an unreviewed safety question as defined in 10CFR50.59.

EDCR E92-2002

NEUTRON NOISE MONITORING SYSTEM

This design change installed the TEC Model 1327 Sentry system to provide continuous in-house monitoring and analysis of the excore neutron detectors. This automated surveillance system allows in-house trending of the power spectral density (PSD) functions, the in-phase PSD functions, and the out-of-phase PSD functions for the four upper and the four lower excore detectors. This modification provides Maine Yankee the ability to do the same analysis as the vendors, with the exception that the detector coherences cannot be obtained with this system. The biggest advantage of the system is that the signals will be monitored continuously and analysis can be performed as often as necessary.

The probability of occurrence of an accident previously evaluated in the FSAR was not increased. This design change did not involve an unreviewed safety question as defined in 10CFR50.59.

EDCR E92-2004

CIRCULATING WATER BOX PRESSURE GAGE TUBING REMOVAL

This design change removed the existing tubing from the condenser vacuum/pressure lines and removed the existing gauges from the system. New tubing and new gauges were installed.

The probability of occurrence of an accident previously evaluated in the FSAR was not increased. This design change did not involve an unreviewed safety question as defined in the 10CFR50.59.

EDCR E93-004

REACTOR PROTECTION SYSTEM (RPS) ACOPIAN POWER SUPPLY ZENER DIODE ADDITION

This minor modification installed zener diodes to the RPS power supplies which utilize remote sensing, so that when a bistable trip unit is removed from service, it will not cause failure of the power supply.

The probability of occurrence of an accident previously evaluated in the FSAR was not increased. This design change did not involve an unreviewed safety question as defined in 10CFR50.59.

EDCR E93-031

REPLACEMENT SERVICE AND INSTRUMENT AIR COMPRESSORS (C-1A,B,S)

This design change replaced the control and service air compressors, C-1A 1B, and 1C. These new compressors are air cooled versus the original design which relied on the secondary component cooling system for cooling. The replacement compressors are of the oil free rotary lobe type units and are mounted on skids. This design allows complete portability of the air compressors for ease of relocation during major overhauls.

The probability of occurrence of an accident previously evaluated in the FSAR was not increased. This design change did not involve an unreviewed safety question as defined in 10CFR50.59.

EDCR E93-2001

DEMINERALIZED WATER STORAGE TANK (DWST) TUBING MODIFICATION

This minor modification relocated the sensing line for LT-1201 (DWST level transmitter) from TK-21 tap N5 to N8 and moved the sensing line from LS-1202, 1203, 1204 (DWST level alarm switches) and LI-1216 from TK-21 tap to N5. This design change implemented the setpoint changes as required per Maine Yankee Cal. 002-92.

The probability of occurrence of an accident previously evaluated in the FSAR was not increased. This design change did not involve an unreviewed safety question as defined in 10CFR50.59.

EDCR E93-2002

INSTALLATION OF FUSES ON CONDENSATE PUMP CONTROL
AND INDICATION POWER SUPPLIES (X-1300A,B,C)

This design change installed three new fuse blocks on X-1300A, B, and C. In the past, if a fault occurred on any one of the field devices, the internal fuse would blow and all control loops (up to 15) would be deenergized. Also, six of the control loops for the travelling water screens were hard wired with no slide links. This posed a maintenance problem since the leads had to be lifted and taped hot, or all 15 control loops would have to be deenergized to perform work on these devices. The addition of the three fuse blocks resolved all of these issues.

The probability of occurrence of an accident previously evaluated in the FSAR was not increased. This design change did not involve an unreviewed safety question as defined in 10CFR50.59.

EDCR E93-2003

RESIDUAL HEAT REMOVAL VALVES (RH-M-1 AND RH-M-2) CABLE REPLACEMENT

This design change upgraded the motor feed cabling for motor operated valves (MOV's) RH-M-1 and RH-M-2. This modification allowed for physical separation of the cables throughout the entire route, improved the voltage drop losses and to provide additional torque output for the MOV's.

The probability of occurrence of an accident previously evaluated in the FSAR was not increased. This design change did not involve an unreviewed safety question as defined in 10CFR50.59.

EDCR E93-2004

FEEDWATER LINE No.3 SHOCK SUPPRESSOR REDUCTION

This minor modification installed rigid struts in four of the suppressor locations to reduce the amount of line "movement". Shock suppressors were installed to help support the line during earthquakes and/or other transients.

The probability of occurrence of an accident previously evaluated in the FSAR was not increased. This design changes did not involve an unreviewed safety question as defined in 10CFR50.59.

EDCR E93-2006

REFUELING PURIFICATION PUMP (P-8) LOW FLOW SWITCH INSTALLATION

This design change installed a new flow switch in the P-8 control circuitry on the discharge side of the pump and also installed a gauge on the suction side of the pump. This modification improved operational reliability of the pump by ensuring that the pump would not operated when the discharge valve was closed.

The probability of occurrence of an accident previously evaluated in the FSAR was not increased. This design change did not involve an unreviewed safety question as defined in 10CFR50.59.

EDCR E93-2007

INSTALLATION OF TEST PORTS FOR FLOW VENTILATION MEASUREMENT

This design change installed an array of test ports across the vent duct perpendicular to the air flow at three locations in the plant. Additional connections were needed so that traverse velocity measurements could be performed to determine an accurate flow rate. Five test connections were installed in the unprotected switchgear room supply duct. Four test connections were installed in the exhaust duct from the computer room and four in the common exhaust duct from F-15 which are both located in the unprotected cable tray room.

The probability of occurrence of an accident previously evaluated in the FSAR was not increased. This design change did not involve an unreviewed safety question as defined in 10CFR50.59.

Summary of Active Temporary Plant Changes (yellow tags)

86-58	Liquid Waste
87-15	Spent Fuel Pool Jib Crane
87-203	Safety Parameter Display System (SPDS)
87-205	Safety Parameter Display System (SPDS)
88-10	Control Room Air Conditioning System
92-07	115 KV (section 69)
92-25	Safety Injection Tank Pressure Switches
92-49	Reactor Protection System (RPS)
92-50	Reactor Coolant System Flow Indication
92-54	Subcooled Margin Monitor
92-69	Emergency Core Cooling System Lightbox
93-04	Pressurizer Proportional Heaters
93-67	Emergency Diesel Generator (DG-1A)
93-78	Reactor Coolant Pump Motor
93-79	Reactor Coolant Pump (RCP)
93-80	Control Element Drive System (CEDS)
93-82	Control Element Drive System (CEDS)
93-85	Steam Driven Feed Pump (P-2C)
93-96	First Point Feed Water Heaters

Yellow Tag 86-58 Liquid Waste

A temporary modification to allow the Duratek System to process liquid waste from the Aerated Drain Tank and discharge to the Test Tanks.

The probability of occurrence of an accident previously evaluated in the FSAR was not increased. This temporary change did not involve an unreviewed safety question as defined in 10CFR50.59.

Yellow Tag 87-15 Spent Fuel Pool Jib Crane

A temporary modification used in fuel reconstitution by installing a jib crane over the spent fuel pool.

The probability of occurrence of an accident previously evaluated in the FSAR was not increased. This temporary change did not involve an unreviewed safety question as defined in 10CFR50.59.

Yellow Tag 87-203, Safety Parameter Display System

Lifted the leads in the Safety Parameter Display System cabinet removing a redundant ground wire from Reactor Protection System Channel "A", Temperature Cold (T_c) and Temperature Hot (T_H) eliminating a ground loop.

The probability of occurrence of an accident previously evaluated in the FSAR was not increased. This temporary change did not involve an unreviewed safety question as defined in 10CFR50.59.

Yellow Tag 87-205, Safety Parameter Display System

Lifted the leads in the Safety Parameter Display System cabinet removing a redundant ground wire eliminating a ground loop from steam generator pressure input to the Reactor Protection System, Channel "A".

The probability of occurrence of an accident previously evaluated in the FSAR was not increased. This temporary change did not involve an unreviewed safety question as defined in 10CFR50.59.

Yellow Tag 88-10, Control Air Conditioning System

Control room air conditioning dampers D-1A and D-1B provide a potential leakage path for the computer room air to leak into the control room during an accident. Damper 1A will be closed by manually closing the damper door.

The probability of occurrence of an accident previously evaluated in the FSAR was not increased. This temporary change did not involve an unreviewed safety question as defined in 10CFR50.59.

Yellow Tag 92-07, Electrical 115 KV Offsite Power

Wire existing construction transformer to provide temporary power for any refueling activities at the containment equipment hatch.

The probability of occurrence of an accident previously evaluated in the FSAR was not increased. This temporary change did not involve an unreviewed safety question as defined in 10CFR50.59.

Yellow Tag 92-25, Safety Injection Tank Pressure Switches

A temporary modification to jumper out the Safety Injection Tanks "high" and "low" pre-alarms, associated with the main control board alarm annunciator windows, which are unreliable. The alarms will still function for the "Alarm" condition.

The probability of occurrence of an accident previously evaluated in the FSAR was not increased. This temporary change did not involve an unreviewed safety question as defined in 10CFR50.59.

Yellow Tag 92-49, Reactor Protection System

A temporary modification that disconnects the non-nuclear system computer points from the Thermal Margin Low Pressure Calculator to allow evaluation for separation criteria.

The probability of occurrence of an accident previously evaluated in the FSAR was not increased. This temporary change did not involve an unreviewed safety question as defined in 10CFR50.59.

Yellow Tag 92-50, Reactor Coolant System Flow Indication

A temporary modification that disconnected a non-nuclear system computer point from a 1E circuit to evaluate the circuit for separation criteria.

The probability of occurrence of an accident previously evaluated in the FSAR was not increased. This temporary change did not involve an unreviewed safety question as defined in 10CFR50.59.

Yellow Tag 92-54, Subcooled Margin Monitor

Provide the Subcooled Margin Monitor Calculator input channels (core exit thermocouples) protection from excessive voltage from the temperature transmitter output. This temporary modification ensures that the temperature transmitters output maintains their minimum valve (~2 Volts DC).

The probability of occurrence of an accident previously evaluated in the FSAR was not increased. This temporary change did not involve an unreviewed safety question as defined in 10CFR50.59.

Yellow Tag 92-69, Emergency Core Cooling System (ECCS) Light Box

The operational mode for LSI-M-11, 21 and 31 Valves have been changed from normally closed to normally open. This temporary modification changed the circuit board switch position to reflect the logic of a normally open valve which stays open on SIAS.

The probability of occurrence of an accident previously evaluated in the FSAR was not increased. This temporary change did not involve an unreviewed safety question as defined in 10CFR50.59.

Yellow Tag 93-04, Pressurizer Proportional Heaters

A cooling fan was temporarily installed in the existing convection-cooled control cabinet to aid in the dissipation of heat within the enclosure and increase the reliability/performance of the "B" proportional heater.

The probability of occurrence of an accident previously evaluated in the FSAR was not increased. This temporary change did not involve an unreviewed safety question as defined in 10CFR5C.59.

Yellow Tag 93-67, Emergency Diesel Generator (DG-1A)

An electrical jumper was added between DG-1A under voltage relay and the incoming voltage meter. This provides a separate return path for the DG-1A under voltage relay thereby removing a potential common mode failure.

The probability of occurrence of an accident previously evaluated in the FSAR was not increased. This temporary change did not involve an unreviewed safety question as defined in 10CFR50.59.

Yellow Tag 93-78, Reactor Coolant Pump (RCP) Motor

Tygon tubing was added to the RCP motor upper bearing oil reservoir vent caps allowing lubricating oil to be added from the containment charging floor while at power.

The probability of occurrence of an accident previously evaluated in the FSAR was not increased. This temporary change did not involve an unreviewed safety question as defined in 10CFR50.59.

Yellow Tag 93-79, Reactor Coolant Pump (RCP)

Opened the slide link to remove erroneous flow indication light for RCP#3 backstop pump "A", there by allowing the "B" backstop pump to auto start when required rather than run constantly.

The probability of occurrence of an accident previously evaluated in the FSAR was not increased. This temporary change did not involve an unreviewed safety question as defined in 10CFR50.59.

Yellow Tag 93-80, Control Element Drive system (CEDS)

The upper electrical limit switch for control element assemble (CEA) number 61 upper electrical limit light failed. Power to this failed switch was removed by opening the slide link.

The probability of occurrence of an accident previously evaluated in the FSAR was not increased. This temporary change did not involve an unreviewed safety question as defined in 10CFR50.59.

Yellow Tag 93-82, Control Element Drive System (CEDS)

The lower electrical limit for control element assembly (CEA) number 7 was not operating properly because the reed switch failed. Power to this reed switch was removed by opening the slide link.

The probability of occurrence of an accident previously evaluated in the FSAR was not increased. This temporary change did not involve an unreviewed safety question as defined in 10CFR50.59.

Yellow Tag 93-85, Steam Driven Feed Pump (P-2C)

The speed sensor on the Woodward governor control failed. A Jumper was installed from the governor speed pickup to the speed sensor assemblies there by restoring P-2C redundancy of the speed input assemblies.

The probability of occurrence of an accident previously evaluated in the FSAR was not increased. This temporary change did not involve an unreviewed safety question as defined in 10CFR50.59.

Yellow Tag 93-96, First Point Feedwater Heater

The first point feedwater heater normal level control transmitters was temporarily modified to allow the low and high side remote pressure sensors to be equalized. This equalization, when returning the transmitter back to service, prevents overranging of the transmitter.

The probability of occurrence of an accident previously evaluated in the FSAR was not increased. This temporary change did not involve an unreviewed safety question as defined in 10CFR50.59.