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UNITED STATES NUCLEAR REGULATORY COMMISSION WASHINGTON, D. C. 20555

POWER AUTHORITY OF THE STATE OF NEW YORK

## DOCKET NO. 50-333

JAMES A. FITZPATRICK NUCLEAR POWER PLANT

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 159 License No. DPR-59

- 1. The Nuclear Regulatory Commission (the Commission) has found that:
  - A. The application for amendment by Power Authority of the State of New York (the licensee) dated January 12, 1990, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act) and the Commission's rules and regulations set forth in 10 CFR Chapter I;
  - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
  - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
  - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
  - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.
- Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment, and paragraph 2.C.(2) of Facility Operating License No. DPR-59 is hereby amended to read as follows:

9012200111 900430 PDR ADOCK 05000333 PDR



(2) Technical Specifications

The Technical Specifications contained in Appendices A and B, as revised through Amendment No.159, are hereby incorporated in the license. The licensee shall operate the facility in accordance with the Technical Specifications.

 This license amendment is effective as of the date of its issuance to be implemented within 30 days.

FOR THE NUCLEAR REGULATORY COMMISSION

Robert a. Copen

Robert A. Capra, Director Project Directorate I=1 Division of Reactor Projects = I/II Office of Nuclear Reactor Regulation

Attachment: Changes to the Technical Specifications

Date of Issuance: April 30, 1990



# ATTACHMENT TO LICENSE AMENDMENT NO. 159

FACILITY OPERATING LICENSE NO. DPR-59

# DOCKET NO. 50-333

Revise Appendix A as follows:

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Remove Pages	Insert Pages
43a	43a
65	65



#### AFMPP

### FABLE 3.1-1 (cont d)

# REACTOR PROTECTION SYSTEM (SCRAM) INSTRUMENTATION REQUIREMENT

#### NOTES OF TABLE 3.1-1 (cont d)

- 14. The APRM flow biased high neutron flux signal is fed through a time constant circuit of approximately 6 seconds. The APRM fixed high neutron flux signal does not incorporate the time constant, but responds directly to instantaneous neutron flux.
- 15. This Average Power Range Monitor scram function is fixed point and is increased when the reactor mode switch is place in the Run positive
- 16. \*During the proposed Hydrogen Addition Test, the background radiation level will increase by approximately a factor of 5 for peak hydrogen concentration. Therefore, within 24 hours prior to performance of the test, the Main Steam Line Radiation Monitor Trip Level Setpoint will is raised to < three times the anticipated radiation levels. Upon completion of the Hydrogen Addition Test, the setpoint will be readjusted to its prior setting within 24 hours.</p>
- This APRM Flow Referenced Scram setting is applicable to two lcop operation. For one loop operation this setting becames S < (0.66W+54%-0.66ΔW) (FRP/MFLPD)</li>

Where:

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ΔW = Difference between two-loop and single-loop effective drive flow at the same core flow.

\* This specification is in effect only during Operating Cycle 10.

Amendment No. \$3, \$1, \$0, 98, 147, 159



#### JAFNPP

## TABLE 3.2-1 (Cont'd)

# INSTRUMENTATION THAT INITIATES PRIMARY CONTAINMENT ISOLATION

### NOTES FOR TABLE 3.2-1

- 1. Whenever Primary Containment integrity is required by Section 3.7, there shall be two operable or tripped trip systems for each function.
- From and after the time it is found that the first column cannot be met for one of the trip systems, that trip system shall be tripped or the appropriate action listed below shall be taken.
  - A. Initiate an orderly shutdown and have the reactor in cold shutdown condition in 24 hours.
  - B. Initiate an orderly load reduction and have main steam lines isolated within eight hours.
  - C. Isolate Reactor Water Cleanup System.
  - D. Isolate shutdown cooling.
- 3. Deleted
- 4. Deleted
- 5. Two required for each steam line.
- 6. These signals also start SBGTS and initiate secondary containment isolation.
- 7. Only required in run mode (interlocked with Mode Switch).
- 8. Bypassed when mode switch is not in run :node and turbine stop valves are closed.
- 9. The trip level setpoint will be maintained at < 3 times normal rated full power background. See note 16 to Table 3.1-1 for re-setting trip level setpoint just prior to and following the Hydrogen Addition Test.

