

# LICENSEE EVENT REPORT

CONTROL BLOCK: \_\_\_\_\_ (1) (PLEASE PRINT OR TYPE ALL REQUIRED INFORMATION)

0 1 | M S G G S I | 2 | 0 0 - 0 0 0 0 0 - 0 0 | 3 | 4 1 1 1 1 | 4 | | | 5

7 8 9 14 15 25 26 30 57 CAT 58

LICENSEE CODE LICENSE NUMBER LICENSE TYPE

CON T

0 1 | L | 6 | 0 5 0 0 0 4 1 6 | 7 | 0 9 2 3 8 2 | 8 | 1 0 2 3 8 2 | 9

7 8 60 61 68 69 74 75 80

REPORT SOURCE DOCKET NUMBER EVENT DATE REPORT DATE

EVENT DESCRIPTION AND PROBABLE CONSEQUENCES (10)

0 2 | While increasing reactor coolant temperature during Non-Nuclear Heatup, a review of

0 3 | Surveillance Procedures 06-EL-1E31-M-1002 revealed Drywell Equipment Drain Sump

0 4 | Timers did not perform a true system functional check as required by T.3.4.4.3.1.b.

0 5 | This increased the probability that a fault in the leak detection system would not be

0 6 | identified. There was no threat to the health or safety of the public or a threat to

0 7 | plant safety due to no fission product inventory present in the core.

0 9   C I   11   D   12   Z   13   Z Z Z Z Z Z   14   Z   15   Z   16	7 8 9 10 11 12 13 18 19 20	SYSTEM CODE CAUSE CODE CAUSE SUBCODE COMPONENT CODE COMP. SUBCODE VALVE SUBCODE
17   8 2     0 8 8     0 3     0	21 22 23 24 26 27 28 29 30 31 32	LER/RO REPORT NUMBER EVENT YEAR SEQUENTIAL REPORT NO. OCCURRENCE CODE REPORT TYPE REVISION NO.
G   18   Z   19   Z   20   Z   21   0 0 0 0   Y   23   N   24   Z   25   Z 9 9 9   26	33 34 35 36 37 40 41 42 43 44 47	ACTION TAKEN FUTURE ACTION EFFECT ON PLANT SHUTDOWN METHOD HOURS ATTACHMENT SUBMITTED NPRD-4 FORM SUB. PRIME COMP. SUPPLIER COMPONENT MANUFACTURER

CAUSE DESCRIPTION AND CORRECTIVE ACTIONS (27)

1 0 | The above surveillance procedure called for only a bench test of pump running timers

1 1 | and did not specify in-system testing. A temporary change notice was written to the

1 2 | above surveillance procedure for functionally testing the system. The system was

1 3 | declared inoperable at 1700 and a cooldown from 340°F was initiated. The system passed

1 4 | the modified surveillance test at 2315 at a reactor coolant temp of 290°F.

1 5 | B | 28 | 0 0 0 | 29 | NA | C | 31 | Surveillance Test Document Review

7 8 9 10 12 13 34 45 46 80

FACILITY STATUS % POWER OTHER STATUS (30) METHOD OF DISCOVERY DISCOVERY DESCRIPTION (32)

1 6 | Z | 33 | Z | 34 | NA | NA

7 8 9 10 11 44 45 80

ACTIVITY CONTENT RELEASED OF RELEASE AMOUNT OF ACTIVITY (35) LOCATION OF RELEASE (36)

1 7 | 0 0 0 | 37 | Z | 38 | NA

7 8 9 11 12 13 80

PERSONNEL EXPOSURES NUMBER TYPE DESCRIPTION (39)

1 8 | 0 0 0 | 40 | NA

7 8 9 11 12 80

PERSONNEL INJURIES NUMBER DESCRIPTION (41)

1 9 | Z | 42 | NA

7 8 9 10 80

LOSS OF OR DAMAGE TO FACILITY TYPE DESCRIPTION (43)

2 0 | N | 44 |

7 8 9 10 80

ISSUED PUBLICATION DESCRIPTION (45)

8210290345 821023  
PDR ADOCK 05000416  
S PDR

NRC USE ONLY

68 69 80

NAME OF PREPARER Original signed by Ron Byrd

PHONE: \_\_\_\_\_

SUPPLEMENTARY INFORMATION TO  
LER 82-088/03 L-0

Mississippi Power & Light Company  
Grand Gulf Nuclear Station - Unit 1  
Docket No. 50-416

Technical Specification Involved: 4.4.2.1.b

Reported Under Technical Specification: 6.9.1.13.b

Event Narrative:

On September 23, 1982, it was discovered that Surveillance Procedures 06-EL-1E31-M-1002, "Drywell Equipment Drain Sump Timers Functional Test" allowed for a bench test of sump pump running timers but did not include an in-circuit test. The procedure was revised by a Temporary Change Notice on September 23, 1982 to require an in-system functional test. The system passed the revised surveillance at 2315 on September 23, 1982 and was returned to service.

At the present time, the entire surveillance program is undergoing an in depth review to determine if any other problems exist. Upon completion of the review, a report will be submitted.