

#6 5/9



Commonwealth Edison  
1400 Opus Place  
Downers Grove, Illinois 60515

LR

May 2, 1994

Mr. William Russell, Director  
Office of Nuclear Reactor Regulation  
U.S. Nuclear Regulatory Commission  
Washington, D.C. 20555

Attn: Document Control Desk

Subject: Braidwood Station Unit 1  
Additional Information Regarding  
Emergency Technical Specification Amendment for  
Technical Specification 3/4.4.5  
NRC Docket No. 50-456

- Reference:
- 1) Teleconference dated May 2, 1994, between the Nuclear Regulatory Commission (NRC) and the Commonwealth Edison Company (CECo)
  - 2) D. Saccomando letter to W. Russell dated April 25, 1994, transmitting request for Emergency Technical Specification amendment for Specification 3/4.4.5

Dear Mr. Russell,

Reference 2 and the subsequent supplements transmitted CECo's request to process an Emergency Technical Specification Amendment to Specification 3/4.4.5 for Braidwood Unit 1. The proposed amendment modifies the Technical Specification to incorporate a 1.0 volt steam generator tube interim plugging criteria (IPC) for Cycle 5.

Per the Reference 2 teleconference, CECo would like to clarify that the finalized version of the WCAP-14046 will contain the following information:

At all support plate intersections where interference signals (dents, corrosion, deposits, mixed residuals), could "mask" the ability to detect a 1 volt bobbin signal, Rotating Pancake Coil (RPC) shall be used. Any RPC confirmed indications shall be plugged or repaired.

The NRC will be informed, prior to plant restart from the refueling outage, of any unexpected inspection findings relative to the assumed characteristics of the flaws at the at the tube support plate elevation. This includes any detectable circumferential indications or detectable indications extending outside the thickness of the tube support plate.

Clarification that: 100% of the tubes were inspected full length with a .610" bobbin probe. All straight length RPC inspection were performed with a .620" 3 coil motorized RPC probe.

9405160074 940502  
PDR ADDCK 05000456  
PDR

ADD 1/0

W. Russell

-2-

May 2, 1994

Clarification that all dents greater than 5.0 volts at support plate intersections were inspected with the MRPC probe.

The WCAP is expected to be issued to the NRC during the latter part of May.

Please address any comments or questions regarding this matter to this office.

Respectfully,



Denise M. Saccomando  
Nuclear Licensing Administrator

cc: R. Assa, Braidwood Project Manager-NRR  
S. Dupont, Senior Resident Inspector-Braidwood  
J. Martin, Regional Administrator-Region III  
Office of Nuclear Facility Safety-IDNS