

Illinois Power Company
Clinton Power Station
P.O. Box 678
Clinton, IL 61727
Tel 217 935-8881, Ext. 3200

John G. Cook
Vice President

**ILLINOIS
POWER**

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L45-94(05-06)LP
2C.220

Docket No. 50-461

10CFR50.73

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Washington, D.C. 20555

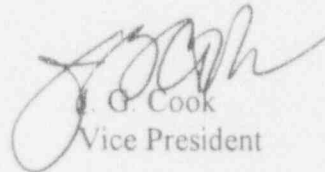
JGC-057-94
May 6, 1994

Subject: Clinton Power Station - Unit 1
Licensee Event Report No. 94-004-00

Dear Sir:

Enclosed is Licensee Event Report No. 94-004-00: Minimum Control Room Staffing Not Met When Only Licensed Senior Reactor Operator In the Control Room Failed to Request Relief Before Exiting. This report is being submitted in accordance with the requirements of 10CFR50.73.

Sincerely yours,


J. G. Cook
Vice President

RSF/csm

Enclosure

cc: NRC Clinton Licensing Project Manager
NRC Resident Office, V-690
Regional Administrator, Region III, USNRC
Illinois Department of Nuclear Safety
INPO Records Center

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LICENSEE EVENT REPORT (LER)

(See reverse for required number of digits/characters for each block)

ESTIMATED BURDEN PER RESPONSE TO COMPLY WITH THIS INFORMATION COLLECTION REQUEST: 50.0 HRS. FORWARD COMMENTS REGARDING BURDEN ESTIMATE TO THE INFORMATION AND RECORDS MANAGEMENT BRANCH (MNB 7714), U.S. NUCLEAR REGULATORY COMMISSION, WASHINGTON, DC 20555-0001, AND TO THE PAPERWORK REDUCTION PROJECT (3150-0104), OFFICE OF MANAGEMENT AND BUDGET, WASHINGTON, DC 20503.

FACILITY NAME (1) Clinton Power Station	DOCKET NUMBER (2) 05000461	PAGE (3) 1 OF 4
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TITLE (4) **Minimum Control Room Staffing Not Met When Only Licensed Senior Reactor Operator In the Control Room Failed to Request Relief Before Exiting**

EVENT DATE (5)			LER NUMBER (6)			REPORT DATE (7)			OTHER FACILITIES INVOLVED (8)	
MONTH	DAY	YEAR	YEAR	SEQUENTIAL NUMBER	REVISION NUMBER	MONTH	DAY	YEAR	FACILITY NAME	DOCKET NUMBER
04	13	94	94	004	00				None	05000
									None	05000

OPERATING MODE (9) 1	THIS REPORT IS SUBMITTED PURSUANT TO THE REQUIREMENTS OF 10 CFR §: (Check one or more) (11)									
	<input type="checkbox"/> 20.402(b)	<input type="checkbox"/> 20.405(c)	<input type="checkbox"/> 50.73(a)(2)(iv)	<input type="checkbox"/> 73.71(b)						
POWER LEVEL (10) 95	<input type="checkbox"/> 20.405(a)(1)(i)	<input type="checkbox"/> 50.36(c)(1)	<input type="checkbox"/> 50.73(a)(2)(v)	<input type="checkbox"/> 73.71(c)						
	<input type="checkbox"/> 20.405(a)(1)(ii)	<input type="checkbox"/> 50.36(c)(2)	<input type="checkbox"/> 50.73(a)(2)(vii)	OTHER						
	<input type="checkbox"/> 20.405(a)(1)(iii)	<input checked="" type="checkbox"/> 50.73(a)(2)(i)	<input type="checkbox"/> 50.73(a)(2)(viii)(A)	(Specify in Abstract below and in Text,						
	<input type="checkbox"/> 20.405(a)(1)(iv)	<input type="checkbox"/> 50.73(a)(2)(ii)	<input type="checkbox"/> 50.73(a)(2)(viii)(B)							
	<input type="checkbox"/> 20.405(a)(1)(v)	<input type="checkbox"/> 50.73(a)(2)(iii)	<input type="checkbox"/> 50.73(a)(2)(x)	NRC Form 366A)						

LICENSEE CONTACT FOR THIS LER (12)

NAME J. A. Neuschwanger, Assistant Director - Plant Operations	TELEPHONE NUMBER (Include Area Code) (217) 935-8881, Extension 3326
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COMPLETE ONE LINE FOR EACH COMPONENT FAILURE DESCRIBED IN THIS REPORT (13)

CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NPRDS	CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NPRDS

SUPPLEMENTAL REPORT EXPECTED (14)				YES	NO	EXPECTED SUBMISSION DATE (15)	MONTH	DAY	YEAR
(If yes, complete EXPECTED SUBMISSION DATE).				<input checked="" type="checkbox"/>	<input type="checkbox"/>				

ABSTRACT (Limit to 1400 spaces, i.e., approximately 15 single-spaced typewritten lines) (16)

With the plant in POWER OPERATION at about 95 percent reactor power, the Line Assistant Shift Supervisor (LASS), the only licensed senior reactor operator (SRO) in the main control room (MCR) area, experienced severe abdominal cramps, requiring him to leave the MCR area for about 3 minutes and 10 seconds to use the restroom. The Shift Supervisor (SS) and the Staff Assistant Shift Supervisor (SASS), the other shift crew SROs, were touring the screenhouse due to high lake water level caused by heavy rain. During the LASS' absence, no SRO was in the MCR, thus the requirements of Technical Specification (TS) 6.2.2.b were not met. TS 6.2.2.b requires at least one licensed SRO in the MCR while the plant is in POWER OPERATION, STARTUP, or HOT SHUTDOWN. No evolutions or power changes were in progress during the absence of SROs from the MCR. The cause of this event is attributed to untimely action by the LASS. When the LASS became aware of his developing discomfort, he delayed contacting the SS or the SASS for relief. When the LASS realized he could no longer delay using the restroom, his condition was such that he had no time for contacting the SS or SASS. Corrective actions include counseling the LASS, and discussing this event with the reactor operators and senior reactor operators.

**LICENSEE EVENT REPORT (LER)
TEXT CONTINUATION**

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TEXT (If more space is required, use additional copies of NRC Form 366A) (17)

DESCRIPTION OF EVENT

On April 13, 1994, the plant was in Mode 1 (POWER OPERATION) at about ninety-five percent reactor [RCT] power. The plant had been operating at reduced reactor power level since about 1730 hours on April 12, 1994, when an orderly plant shutdown was initiated in accordance with off-normal procedure CPS 4303.02, "Abnormal Lake Level," due to abnormally high and increasing lake water level caused by heavy rain. At about 2200 hours on April 12, 1994, the lake water level had stabilized and the power reduction was stopped at ninety-five percent reactor power.

On April 13, 1994, at about 0256 hours and 0309 hours respectively, the Operations Shift Supervisor (SS) and the Staff Assistant Shift Supervisor (SASS), both licensed senior reactor operators (SROs), left the main control room (MCR) area and went to the screenhouse to check the status of efforts to restore the plant from the effects of high lake water level. The SS notified the Line Assistant Shift Supervisor (LASS) that he and the SASS were leaving the MCR and that their destination was the screenhouse. The LASS, a licensed SRO, and two licensed reactor operators remained in the MCR area.

At about 0354 hours, while the SS and the SASS were out of the MCR area, the LASS experienced severe abdominal cramps, requiring him to leave the MCR area to use the restroom. The LASS was out of the MCR area for about three minutes and ten seconds.

During the LASS' absence, a licensed SRO was not in the MCR area, thus, the requirement of Technical Specification (TS) 6.2.2.b was not met. TS 6.2.2.b requires that at least one licensed senior operator shall be in the control room while the plant is in Modes 1, 2 (STARTUP), or 3 (HOT SHUTDOWN).

The LASS was aware of the TS 6.2.2.b requirement but felt he did not have sufficient time to summon the SS or SASS to return from the screenhouse. One of the two licensed reactor operators in the MCR has an SRO license, but the license is not being maintained in active status. No evolutions or power changes were in progress during the absence of the SROs.

No attempts were made by the LASS or the reactor operators to contact the SS or the SASS to advise them of the situation although the LASS and the reactor operators were cognizant of the control room staffing requirements.

At about 0359 hours, when the SS returned to the MCR area from the screenhouse, the LASS reported the absence of a licensed SRO in the MCR.

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TEXT (If more space is required, use additional copies of NRC Form 366A) (17)

The LASS was immediately counseled by the SS about the importance of meeting the TS requirement for having a minimum of one licensed senior reactor operator in the Main Control Room.

Condition Report 1-94-04-029 was initiated to track a root cause and corrective action determination for this event.

No automatic or manually initiated safety system responses were necessary to place the plant in a safe and stable condition. No equipment or components were inoperable at the start of this event to the extent that their inoperable condition contributed to this event.

CAUSE OF EVENT

The cause of this event is attributed to untimely action by the LASS because he failed to request relief before exiting the MCR. When the LASS became aware of his developing discomfort, he delayed contacting the SS or the SASS for relief because of their involvement in assessing the recovery efforts from the high lake level. When the LASS realized he could no longer delay using the restroom, his condition was such that he had no time for contacting the SS or SASS.

CORRECTIVE ACTION

When the SS returned to the main control room from the screenhouse, the LASS immediately informed him of the incident.

The SS discussed this incident with the LASS, placing emphasis on the importance of maintaining compliance with the Technical Specification requirement for having an SRO in the MCR. The SS also counseled the LASS that he should have notified the SS or had a reactor operator notify the SS about needing to leave the MCR.

This event will be discussed with reactor operators and senior reactor operators to emphasize their responsibilities about informing the shift supervisor when the only available licensed senior reactor operator becomes incapacitated or has to leave the main control room without proper relief.

ANALYSIS OF EVENT

This event is reportable under the provisions of 10CFR50.73(a)(2)(i)(B) because operating the plant in Mode 1 without having a qualified senior reactor operator in the main control room constitutes an operation prohibited by the plant's Technical Specifications.

**LICENSEE EVENT REPORT (LER)
TEXT CONTINUATION**

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Assessment of the safety consequences and implications of this event indicates that this event was not nuclear safety significant. One of the two licensed reactor operators has an SRO license; however, the SRO license is not being maintained active. The SS or the SASS could have been contacted quickly if a plant emergency arose. The SS was in the power block and enroute to the MCR when the LASS exited the MCR; however, because the LASS did not attempt to contact the SS, the LASS was not aware that the SS had already returned to the power block. The plant was in steady state operations with no evolutions or power changes in progress. The ability to conduct plant operations was not impaired. The return of the LASS to the MCR restored the shift staffing to Technical Specification requirements.

ADDITIONAL INFORMATION

No equipment or components failed during this event.

Clinton Power Station has not reported similar events in the past.

For further information regarding this event, contact J. A. Neuschwanger, Assistant Director-Plant Operations at (217) 935-8881, extension 3326.