NHC FORM 366" U. S. NUCLEAR REGULATORY COMMISSION (7.77) LICENSEE EVENT REPORT PREVIOUS REPORT DATE 7/29/82 UPDATE REPORT CONTROL BLOCK: 10 (PLEASE PRINT OR TYPE ALL REQUIRED INFORMATION) A L B R F 0 1 (5) LICENSEE CODE CON'T REPORT L 6 0 5 0 0 0 2 5 9 7 0 5 1 0 8 2 3 1 0 1 3 8 2 0 61 DOCKET NUMBER 68 69 EVENT DATE 74 75 REPORT DATE 8 0 1 SOURCE EVENT DESCRIPTION AND PROBABLE CONSEQUENCES (10) During normal operation, the unit 1 "B" H, Analyzer was discovered to be inoperable 0 2 (T.S 3.7.H.2) due to the failure of the H, sample inlet pump to run. There was 03 no effect on public health and safety in that T.S. 3.7.H.2 permits operation for 0 4 30 days with one hydrogen analyzer. inoperable. Analyzer "B" was inoperable for 0 5 about 26 hours. "A" hydrogen analyzer was available and operable. 0 6 0 7 0 8 SYSTEM CODE CAUSE CAUSE SUBCODE COMP VALVE CODE COMPONENT CODE SUBCODE A (11 E (12) A (13) R L A Y X (14) A (15 ZI (16) 13 OCCURRENCE SEQUENTIAL REPORT REVISION EVENT YEAR REPORT NO. CODE LER/RO TYPE 2 (17 REPORT 8 2 0 3 1 0 3 XI NUMBER 28 32 23 NPRD-4 FORM SUB. ACTION ACTION EFFECT ON PLANT METHOD ATTACHMENT SUBMITTED PRIME COMP. COMPONENT HOURS (22) SUPPLIER MANUFACTUREP LY Y 23 010101 (18) X Z (21) 0 24 L 25 A |1 |6 0 (26) 41 CAUSE DESCRIPTION AND CORRECTIVE ACTIONS (27) sample inlet pump would not run due to failure of Allen Bradley relay model 10 "B" Ho 700-N400Al contacts. Relay was replaced and SI 4.7.H successfully completed. 1 1 Evaluation of the application of the relay is still in progress. Investigation 1 2 of the operating parameters of the relay under various system configurations has been initiated. Results are expected by 2/15/83. 1 4 80 FACILITY METHOD OF DISCOVERY OTHER STATUS (30) % POWER DISCOVERY DESCRIPTION (32) F (28) A (31) Operator observation 9 9 0 ACTIVITY CONTENT 80 RELEASED OF RELEASE AMOUNT OF ACTIVITY (35) LOCATION OF RELEASE (36) Z (33) Z (34) 6 NA NA 1.1 80 PERSONNEL EXPOSURES NUMBER TYPE DESCRIPTION (39) 0 (37) Z 2 0 NA PERSONNEL INJURIES 80 DESCRIPTION (41) NUMBER 0 (40) 0 NA 80 LOSS OF OR DAMAGE TO FACILITY 1 (43) DESCRIPTION TYPE Z (42) NA 8210290321 821013 80 PUBLICITY NRC USE ONLY DESCRIPTION (15) LN 44 PDR ADOCK 05000259 PDR NA 6.8 69 80 PHONE (205) 729-0800 NAME OF PREPARER ____ Stanley Carter

Tennessee Valley Authority Browns Ferry Nuclear Plant

Form BF 17 BF 15.2 2/12/82

LER SUPPLEMENTAL INFORMATION

BFRO-50- 259 / 82031 R2 Technical Specification Involved 3.7.H.2

Reported Under Technical Specification 6.7.2.b.(2) Date Due NRC N/A

Event Narrative:

Unit 2 was operating at 86-percent power, unit 3 was in a turbine outage and both were unaffected by this event. During normal operation, with unit 1 operating at 99-percent power, "B" H₂ analyzer was discovered to be inoperable due to failure of the H₂ sample inlet pump to run. Contacts 3-7 and 4-8 of relay R2, which supply power to the motor of the H₂ sample inlet pump, failed, thereby preventing operation of the H₂ sample inlet pump. Technical Specification 3.7.H.2 allows operation for 30 days with one hydrogen analyzer operable; "A" hydrogen analyzer was available and operable. Relay R2 was replaced and SI 4.7.H was successfully completed. Initial response from the supplier was that there was not a misapplication of the relay. Evaluation of the relay is still in progress. Investigation of the operating parameters of the relay under various system configurations has been initiated. Results of the investigation are expected by February 15, 1983. Past performance of these relays justifies their continued usage until the evaluation is completed.

* Previous Similar Events:

None.

Retention: Period - Lifetime; Responsibility - Document Control Supervisor *Revision: