



Wisconsin Electric POWER COMPANY

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October 22, 1982

Mr. H. R. Denton, Director
Office of Nuclear Reactor Regulation
U. S. NUCLEAR REGULATORY COMMISSION
Washington, D. C. 20555

Attention: Mr. R. A. Clark, Chief
Operating Reactor Branch 3

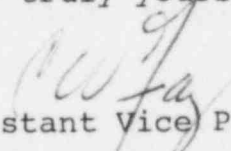
Gentlemen:

DOCKET NOS. 50-266 AND 50-301
OFF-SITE DOSE ASSESSMENT
POINT BEACH NUCLEAR PLANT, UNITS 1 AND 2

On October 4, 1982 the Commission issued Amendment 66 to Facility Operating License DPR-24 and Amendment 69 to Facility Operating License DPR-27 for Point Beach, Units 1 and 2, respectively. These amendments approved changes to the plant Technical Specifications which modified the operation and surveillance testing requirements for the containment purge supply and exhaust valves.

The safety evaluation included with these license amendments imposed the additional requirement that the licensee complete a confirmatory analysis demonstrating that an unacceptable radioactive release from containment would not take place as a result of a small-break LOCA in which operator action is required to isolate the continuous vent through the R-11 and R-12 sample lines. This analysis has been completed consistent with the assumptions given by the NRC. These assumptions included containment pressure at 6 psig, specific activity of the reactor coolant at 1.0 $\mu\text{C/gm}$ dose equivalent I-131, reactor coolant leak rate at 150 gpm, and 30 minutes required for operator action to isolate the containment one-inch vent path. The results of this analysis indicate a limiting site boundary thyroid dose of less than 0.2 mRem and confirm that the radioactivity released during a small-break LOCA from this potential source would be negligible.

Very truly yours,


Assistant Vice President

C. W. Fay

Copy to NRC Resident Inspector

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