

#### **BOSTON EDISON**

Pilgrim Nuclear Power Station Rocky Hill Road Plymouth, Massachusetts 02360

> December 12, 1990 BECo Ltr. #90-155

U.S. Nuclear Regulatory Commission Attention: Document Control Desk Washington, DC 20555

> License No. DPR-35 Docket No. 50-293

Subject: November 1990 Monthly Report

Dear Sir:

In accordance with PNPS Technical Specification 6.9.A.2, a copy of the Operational Status Summary for Pilgrim Nuclear Power Station is attached for your information and planning. Should you have any questions concerning this report please contact me directly.

R.A. Anderson

Station Director and

Vice President, Nuclear Operations

GJB/bal

Attachment

cc: Regional Administrator, Region 1 U.S. Nuclear Regulatory Commission 475 Allendale Rd. King of Prussia, PA 19406

Senior Resident Inspector

9012190118 901130 PDR ADJOCK 05000293 PDR SEPA

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#### AVERAGE DAILY UNIT POWER LEVEL

DOCKET NO. 50-293
UNIT Pilgrim 1
DATE December 12, 1990
COMPLETED BY G. Basilesco
TELEPHONE (508) 747-8534

# MONTH November 1990

DAY	AVERAGE DAILY POWER LEVEL (MWe-Net)	DAY	AVERAGE DAILY POWER LEVEL (MWe-Net)
1	663	17	662
2	586	18	C64
3	428	19	662
4	546	20	663
5	656	21	663
6	663	22	662
7	664	23	663
8	663	24	662
9	663	25	663
10	661	26	663
11	664	27	663
12	664	28	663
13	664	29	663
14	664	30	661
15	663	31	N/A
16	663		

This format lists the average daily unit power level in MWe-Net for each day in the reporting month, computed to the nearest whole megawatt.

## OPERATING DATA REPORT

DOCKET	DOCKET NO.		. 50-293				
DATE			Dec	embe	r	12,	1990
COMPLET	ED						
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# OPERATING STATUS

		Notes						
	it Name Ported November 1990							
. Ke	porting Period <u>November 1990</u> censed Thermal Power (MWt)	1998						
. Na	meplate Rating (Gross MWe)	678						
. De	sign Electrical Rating (Net MWe)	655						
. Ma	ximum Dependable Capacity (Gross MWe)_ximum Dependable Capacity (Net MWe)	670						
. If		ges Occur in Capacity Ratings (Items Number 3 Through 7) Since Las Give Reasons:						
. Po	wer Level To Which Restricted, If Any	(Net MWe)	None					
O. Re	easons For Restrictions, If Any N	/A						
		This Month	Yr-to-Date	Cumulative				
	ours In Reporting Period	720.0	8016.0	157584.0				
	umber Of Hours Reactor Was Critical	720.0	6451.0	91856.7				
	eactor Reserve Shutdown Hours ours Generator On-Line	720.0	6042 8	88172.1				
	nit Reserve Shutdown Hours	0.0	6042.8	001/6.				
	ross Thermal Energy Generated(MWH)	1408056.0		151920480.0				
	ross Electrical Energy Generated(MWH)	484810.0		51158294.0				
	et Electrical Energy Generated (MWH)	466780.0		49155476.0				
	nit Service Factor	100.0	75.4	56.0				
	nit Availability Factor	100.0	75.4	56.0				
	nit Capacity Factor (Using MDC Net)	96.8	70.2					
23 11	nit Capacity Factor (Using DER Net) nit Forced Outage Rate	99.0	71.8	47.6				
	hutdowns Scheduled Over Next 6 Months (	Type Date a	nd Duration	of Each):				
Refue	ling Outage No. 8, May 1991, approximat	ely 70 days	ing purgeron	or eacher				
	f Shut Down At End Of Report Period, Es							
26. UI	nits In Test Status (Prior to Commercia	( Operation)						
			N/A					
	THEFTAL CONTROLLERS		forecast A	chieved				
	INITIAL CRITICALITY		-					
	INITIAL ELECTRICITY		-					
	COMMERCIAL OPERATION	and the second	-	-				

(9/77)

# BOSTON EDISON COMPANY PILGRIM NUCLEAR POWER STATION DOCKET NO. 50-293

#### Operational Summary for November 1990

The unit started and ended the reporting period at approximately 100 percent power. On November 1, 1990, the High Pressure Coolant Injection (HPCI) System was voluntarily taken out of service and maintenance was performed on the HPCI turbine. The system was returned to service on November 5, 1990. On November 3, 1990, power was reduced to approximately 50 percent to perform a thermal backwash of the main condenser and returned to 100 percent power on November 5, 1990. In addition, minor power reductions for control rod exercises were performed on November 10, 17 and 24.

#### Safety Relief Valve Challenge: Month of November 1990

Requirement: NUREG-0737 T.A.P. II.K.3.3

There were no safety relief valve challenges during this reporting period.

An SRV challenge is defined as anytime an SRV has received a signal to operate via reactor pressure, auto signal (ADS) or control switch (manual). Ref. BECo ltr. #81-01 dated 01/05/81.

#### REFUELING INFORMATION

The following refueling information is included in the Monthly Report as requested in an NRC letter to BECo, dated January 18, 1978:

For your convenience, the information supplied has been enumerated so that, each number corresponds to equivalent notation utilized in the request.

- The name of this facility is Pilgrim Nuclear Power Station, Docket Number 50-293.
- 2. Scheduled date for next Refueling Shutdown: Second Quarter 1991
- 3. Scheduled date for restart following refueling: Third Quarter 1991
- Due to their similarity, requests 4, 5, & 6 are responded to collectively under #6.
- 5. See #6.
- The new fuel loaded during the 1986/87 refueling outage was of the same design as loaded in the previous outage, and consisted of 192 assemblies.
- 7. (a) There are 580 fuel assemblies in the core.
  - (b) There are 1320 fuel assemblies in the spent fuel pool.
- (a) The station is presently licensed to store 2320 spent fuel assemblies. The actual usable spent fuel storage capacity is 2320 fuel assemblies.
  - (b) The planned spent fuel storage capacity is 2320 fuel assemblies.
- With present spent 'uel in storage, the spent fuel pool now has the capacity to accommodate an additional 1000 fuel assemblies.

#### PILGRIM NUCLEAR POWER STATION

#### MAJOR SAFETY RELATED MAINTENANCE

SYSTEM	COMPONENT	MALFUNCTION	CAUSE	MAINTENANCE	CORRECTIVE ACTION TO PREVENT RECURRENCE	ASSOCIATED LER
High Pressure Coolant Injection (HPCI) System	HPCI Turbine	Overspeed trip (F&MR 90-352)	Mechanical/ hydraulic mal- function of speed control system.	Changed pilot valve spring; aligned bushing ports; replaced EG-M.	Reference LER 90-017-00	LER 90-017-00

CHARTY	Present.	TOOLING	BAIR	DOLLED	DEDL	CTT	PARE
SWII	2HR	TDOWNS	ANU	PUMER	KEUL	スコエ	UNS

DOCKET NO. 50-293

NAME Pilgrim 1

DATE December 12, 1990 .

COMPLETED BY G. Basilesco .
TELEPHONE (508) 747-8534

## REPORT MONTH November 1990

NO.	DATE	TYPE <sup>1</sup>	DURATION (HOURS)	REASON <sup>2</sup>	METHOD OF SHUTTING DOWN REACTOR <sup>3</sup>	LICENSE EVENT REPORT #	SYSTEM CODE 4	COMPONENT CODE 5	CAUSE & CORRECTIVE ACTION TO PREVENT RECURRENCE
14	11/3/90	S	0.0	N/A	5	N/A	N/A	N/A	Power reduction to perform main condenser backwash.

1	2	2	3	4&5
	A-Equip Failure B-Maint or Test C-Refueling D-Regulatory Restric E-Operator Training & License Examinat		1-Manual 2-Manual Scram 3-Auto Scram 4-Continued 5-Reduced Load 9-Other	Exhibit F & H Instructions for Preparation of Data Entry Sheet Licensee Event Report (LER) File (NUREG-1022)