



UNITED STATES  
NUCLEAR REGULATORY COMMISSION  
WASHINGTON, D. C. 20555

ARKANSAS POWER & LIGHT COMPANY

DOCKET NO. 50-313

ARKANSAS NUCLEAR ONE - UNIT NO. 1

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 69  
License No. DPR-51

1. The Nuclear Regulatory Commission (the Commission) has found that:
  - A. The application for amendment by Arkansas Power and Light Company (the licensee) dated July 14, 1982, as supplemented August 24, 1982, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations set forth in 10 CFR Chapter I;
  - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
  - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
  - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
  - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.

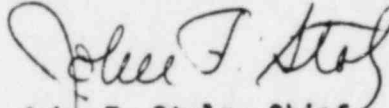
2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment, and paragraph 2.c.(2) of Facility Operating License No. DPR-51 is hereby amended to read as follows:

Technical Specifications

The Technical Specifications contained in Appendices A and B, as revised through Amendment No. 69, are hereby incorporated in the license. The licensee shall operate the facility in accordance with the Technical Specifications.

3. This license amendment is effective as of the date of its issuance.

FOR THE NUCLEAR REGULATORY COMMISSION



John F. Stolz, Chief  
Operating Reactors Branch #4  
Division of Licensing

Attachment:  
Changes to the Technical  
Specifications

Date of Issuance: September 21, 1982

ATTACHMENT TO LICENSE AMENDMENT NO. 69

FACILITY OPERATING LICENSE NO. DPR-51

DOCKET NO. 50-313

Replace the following pages of the Appendix "A" Technical Specifications with the enclosed pages. The revised pages are identified by Amendment number and contain vertical lines indicating the area of change.

Pages

42a

43b

45b

45d

72b

- 3.5.1.7 The Decay Heat Removal System isolation valve closure setpoints shall be equal to or less than 340 psig for one valve and equal to or less than 400 psig for the second valve in the suction line. The relief valve setting for the DHR system shall be equal to or less than 450 psig.
- 3.5.1.8 The degraded voltage monitoring relay settings shall be as follows:
- a. The 4.16 KV emergency bus undervoltage relay setpoints shall be > 3115 VAC but < 3177 VAC.
  - b. The 460 V emergency bus undervoltage relay setpoints shall be > 423 VAC but < 431 VAC with a time delay setpoint of 8 seconds  $\pm$  1 second.
- 3.5.1.9 The following Reactor Trip circuitry shall be operable as indicated:
1. Reactor trip upon loss of Main Feedwater shall be operable (as determined by Specification 4.1.a, items 2 and 36 of Table 4.1-2) at greater than 5% reactor power. (May be bypassed up to 10% reactor power.)
  2. Reactor trip upon Turbine Trip shall be operable (as determined by Specification 4.1.a, items 2 and 42) at greater than 5% reactor power. (May be bypassed up to 20% reactor power.)
  3. If the requirements of Specifications 3.5.1.9.1 or 3.5.1.9.2 cannot be met, restore the inoperable trip within 12 hours or bring the plant to a hot shutdown condition.
- 3.5.1.10 The control room ventilation chlorine detection system instrumentation shall be operable & capable of actuating control room isolation and filtration systems, with alarm/trip setpoints adjusted to actuate at a chlorine concentration of  $\leq$  5ppm.

The OPERABILITY of the accident monitoring instrumentation ensures that sufficient information is available on selected plant parameters to monitor and assess these variables during and following an accident. This capability is consistent with the recommendations of Regulatory Guide 1.97, "Instrumentation for Light-Water-Cooled Nuclear Power Plants to Assess Plant Conditions During and Following an Accident", December 1975 and NUREG-0578, "TMI-2 Lessons Learned Task Force Status Report and Short-Term Recommendations".

The OPERABILITY of the chlorine detection system ensures that sufficient capability is available to promptly detect and initiate protective action in the event of an accidental chlorine release. This capability is required to protect control room personnel and is consistent with the recommendations of Regulatory Guide 1.95, "Protection of Nuclear Power Plant Control Room Operators against an Accidental Chlorine Release", February 1975.

#### REFERENCE

FSAR, Section 7.1

Table 3.5.1-1 (Cont'd)

OTHER SAFETY RELATED SYSTEMS

	1	2	3	4	5
<u>Functional Unit</u>	<u>No. of channels</u>	<u>No. of channels for system trip</u>	<u>Min. operable channels</u>	<u>Min. degree of redundancy</u>	<u>Operator Action if conditions of column 3 or 4 cannot be met</u>
2. Steam line break instrumentation control system (SLBIC). (a) SLBIC Control & Logic Channels	2	1	2	1	Notes 9, 5
3. Pressurizer level channels	3	N/A	2	1	Note 10
4. Emergency Feedwater flow channels	2/S.G.	N/A	1	0	Note 10
5. RCS subcooling margin monitors	2	N/A	1	0	Note 10
6. Electromatic relief valve flow monitor	2	N/A	1	0	Note 11
7. Electromatic relief block valve position indicator	1	N/A	1	0	Note 12
8. Pressurizer code safety valve flow monitors	2/valve	N/A	1/valve	0	Note 10
9. Degraded Voltage Monitoring					
a. 4.16 KV Emergency Bus Undervoltage	2/Bus	1/Bus	2/Bus	0	Note 14
b. 460 V Emergency Bus Undervoltage	*1/Bus	1/Bus	1/Bus	0	Notes 13,14
10. Chlorine Detection Systems	2	1	2	0	Notes 17,18

\*Two undervoltage relays per bus are used with a coincident trip logic (2-out-of-2)

Table 3.5.1-1 (Cont'd)

Notes Cont'd.

13. Channels may be bypassed for not greater than 30 seconds during reactor coolant pump starts. If the automatic bypass circuit or its alarm circuit is inoperable, the undervoltage protection shall be restored within 1 hour, otherwise, Note 14 applies.
14. With the number of channels less than required, restore the inoperable channels to operable status within 72 hours or be in hot shutdown within the next 6 hours and in cold shutdown within the following 30 hours.
15. This trip function may be bypassed at up to 10% reactor power.
16. This trip function may be bypassed at up to 20% reactor power.
17. With no channel operable, within 1 hour restore the inoperable channels to operable status, or initiate and maintain operation of the control room emergency ventilation system in the recirculation mode of operation.
18. With one channel inoperable, restore the inoperable channel to operable status within 7 days or within the next 6 hours initiate and maintain operation of the control room emergency ventilation system in the recirculation mode of operation.

Table 4.1-1 (Cont'd)

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72b

	<u>Channel Description</u>	<u>Check</u>	<u>Test</u>	<u>Calibrate</u>	<u>Remarks</u>
47.	EFW Actuation Control Logic	HA	H	R	
48.	EFW Flow Indication	R	NA	R	
49.	RCS subcooling margin monitor	D	NA	R	
50.	Electromatic relief valve flow monitor	D	NA	R	
51.	Electromatic relief block valve position indicator	D	NA	R	
52.	Pressurizer safety valve flow monitor	D	NA	R	
53.	Pressurizer water level indicator	D	NA	R	
54.	Control Room Chlorine Detector	D	M	R	

Note: S-Each Shift  
W-Weekly  
M-Monthly  
D-Daily

T/W-Twice per Week  
Q-Quarterly  
P-Prior to each startup if not done previous week  
B/M-Every 2 Months

R-Once every 18 months  
PC-Prior to going Critical if not done within previous 31 days  
NA-Not applicable