**Commonwealth Edison**
LaSalle County Nuclear Station
Rural Route #1, Box 220
Marseilles, Illinois 61341
Telephone 815/357-6761

December 10, 1990

Director of Nuclear Reactor Regulation
United States Nuclear Regulatory Commission
Mail Station P1-137
Washington, D.C. 20555

ATTN: Document Control Desk

Gentlemen:

Enclosed for your information is the monthly performance report covering LaSalle County Nuclear Power Station for November, 1990.

Very truly yours,

for G. J. Diederich
Station Manager
LaSalle County Station

GJD/MJC/djf

Enclosure

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LASALLE NUCLEAR POWER STATION

UNIT 1

MONTHLY PERFORMANCE REPORT

NOVEMBER 1990

COMMONWEALTH EDISON COMPANY

NRC DOCKET NO. 050-373

LICENSE NO. NPF-11

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(UNIT 1)

I. INTRODUCTION (Unit 1)

The LaSalle County Nuclear Power Station is a two-unit facility owned by Commonwealth Edison Company and located near Marseilles, Illinois. Each unit is a Boiling Water Reactor with a designed net electrical output of 1078 Megawatts. Waste heat is rejected to a man-made cooling pond using the Illinois River for make-up and blowdown. The architect-engineer was Sargent and Lundy and the primary construction contractor was Commonwealth Edison Company.

Unit One was issued operating license number NPF-11 on April 17, 1982. Initial criticality was achieved on June 21, 1982 and commercial power operation was commenced on January 1, 1984.

This report was compiled by Michael J. Cialkowski, telephone number (815)357-6761, extension 2427.

II. MONTHLY REPORT

A. SUMMARY OF OPERATING EXPERIENCE (Unit 1)

| Day | Time | Event |
|---|---|---|
| 1 | 0000 | Reactor critical, generator on-line at 1120 MWE |
| 4 | 2330 | Reduced power to 1080 MWE for system load. |
| 5 | 1000 | Increased power to 1120 MWE. |
| | 2230 | Reduced power to 1080 MWE for CRD exercising. |
| 6 | 0600 | Increased power to 1120 MWE. |
| 13 | 0100 | Reduced power to 850 MWE for monthly surveillances. |
| | 1000 | Increased power to 1120 MWE. |
| 14 | 0130 | Reduced power to 740 MWE for scram time testing, rod set and strong back removal. |
| | 1300 | Increased power to 1120 MWE. |
| 19 | 0130 | Reduced power to 1050 MWE for condensate pump transfer. |
| | 0800 | Increased power to 1120 MWE. |
| 20 | 0200 | Reduced power to 850 MWE for system load and for CRD exercising. |
| | 1000 | Increased power to 1120 MWE. |
| | 2400 | Reduced power to 850 MWE for system load. |
| 21 | 1100 | Increased power to 1120 MWE. |
| 22 | 0200 | Reduced power to 850 MWE for system load. |
| 23 | 2000 | Increased power to 1120 MWE. |
| 26 | 2300 | Reduced power to 1000 MWE for system load. |
| 27 | 1000 | Increased power to 1120 MWE. |
| 28 | 0300 | Reduced power to 900 MWE for system load. |
| | 1300 | Increased power to 1120 MWE. |
| 29 | 1100 | Reduced power to 800 MWE due to Division 1 battery charger problems. |
| | 2100 | Increased power to 1000 MWE. |
| 30 | 1700 | Increased power to 1120 MWE. |
| | 2400 | Reactor critical, generator on-line at 960 MWE reducing power to 750 MWE for CRD problem. |

B. PLANT OR PROCEDURE CHANGES, TESTS, OR EXPERIMENTS (50.59) (Unit 1)

1. Amendments to the Facility License or Technical Specification.
(None)

2. Changes to procedures which are described in the Safety Analysis Report.

a) LaSalle Operating Procedure LOP-WF-30, "Transfer of Floor Drain Process System Tank Water to Equipment Drain and Waste Collector (WE) Tank (1WE03T)" was used to transfer evaporator head tank water to the liquid radwaste sample tank to make use of the 30,000 gallon volume of 1WE03T to hold water, while WE filter gasket repairs were underway. The safety function of radwaste is to limit radioactive concentrations released to the environment. Prior to going to the head tank, the water would have been processed through an evaporator. After getting to WE, it will be sampled and routed accordingly. The Safety Evaluation concluded that radwaste is not the basis for any Technical Specification margin of safety and the radioactive effluent dose limits of Technical Specification 3/4.11.2.3 still remain valid.

b) LaSalle Special Procedure LLP-90-075, "Auxiliary Building HVAC (VA) Purge", was performed to allow the purge of the Auxiliary Building office areas without endangering the required differential pressure criteria of the control room envelope. This change allowed cool outside air into the office spaces while the HVAC compressor is being repaired. The Safety Evaluation prepared for this test stated that the VA system is non-safety and its failure doesn't affect any safety systems. VA normally uses outside air for makeup.

3. Changes to facility which are described in the Safety Analysis Report.

a) LaSalle Temporary System Change (TSC) 1-766-90 was performed to defeat Drywell Drain Sump fill up rate alarm, due to having a faulty transmitter on its flow element, which was causing a nuisance alarm. The unidentified leakage totalizer will remain operable for sump monitoring and identified leakage technical specifications will not be affected. The safety evaluation concluded that the alarm circuit is non-safety and does not affect the consequences of an accident.

4. Tests and Experiments not described in the Safety Analysis Report.
(None)

C. MAJOR CORRECTIVE MAINTENANCE TO SAFETY-RELATED EQUIPMENT (including SOR differential pressure switch failure reports).
(See Table 1)

D. COMPLETED SAFETY-RELATED MODIFICATIONS.
(None)

C TABLE 1 (Unit 1)

MAJOR CORRECTIVE MAINTENANCE TO
SAFETY-RELATED EQUIPMENT

| WORK REQUEST NUMBER | COMPONENT | CAUSE OF MALFUNCTION | RESULTS AND EFFECTS ON SAFE PLANT OPERATION | CORRECTIVE ACTION |
|---|---|---|---|---|
| L03574 | 1VY04C | 74 Relay | None | Replaced 74 relay |
| L03582 | 1A Main Steam Line Radiation Monitor | Control Module | None | Replaced control module |
| L03601 | APRM "F" | Power Supply | Improper voltage to APRM | Replaced power supply |
| L03662 | Division 1 Post LOCA Oxygen Monitor | Regulator, cell | Erratic oxygen samples | Replaced regulator, cell, amplifier board |
| L03924 | Control Rod Number 50.47 | Pilot solenoid valves | None | Replaced pilot solenoid valves |

(No SOR failures this month)

E. LICENSEE EVENT REPORTS (Unit 1)

| LER Number | Date | Description |
|---|---|---|
| (NONE) | | |

F. DATA TABULATIONS (Unit 1)

1. Operating Data Report (See Table 3)

2. Average Daily Unit Power Level (See Table 4)

3. Unit Shutdowns and Significant Power Reductions (See Table 5)

TABLE 3
D.1 OPERATING DATA REPORT

DOCKET NO. 050-373
UNIT LASALLE ONE
DATE DECEMBER 10,1990
COMPLETED BY M.J. CIALKOWSKI
TELEPHONE (815)-357-6761

OPERATING STATUS

1. REPORTING PERIOD: NOVEMBER ,1990 — GROSS HOURS IN REPORTING PERIOD: 720
2. CURRENTLY AUTHORIZED POWER LEVEL (MWt): 3,323 — MAX DEPEND CAPACITY (MWe-Net): 1,036
DESIGN ELECTRICAL RATING (MWe-Net): 1,078
3. POWER LEVEL TO WHICH RESTRICTED (IF ANY) (MWe-Net): (NONE)
4. REASONS FOR RESTRICTION (IF ANY):

| | REPORTING PERIOD DATA | | |
|---|---|---|---|
| | THIS MONTH | YEAR-TO-DATE | CUMULATIVE |
| 5. REACTOR CRITICAL TIME (HOURS) | 720.0 | 7,731.3 | 39,865.4 |
| 6. REACTOR RESERVE SHUTDOWN TIME (HOURS) | 0.0 | 0.0 | 1,641.2 |
| 7. GENERATOR ON-LINE TIME (HOURS) | 720.0 | 7,586.5 | 39,004.8 |
| 8. UNIT RESERVE SHUTDOWN TIME (HOURS) | 0.0 | 0.0 | 1.0 |
| 9. THERMAL ENERGY GENERATED (MWH) | 2,317,784 | 23,961,833.4 | 111,794,601 |
| 10. ELECTRICAL ENERGY GENERATED (MWHe-Gross) | 789,091 | 8,110,747 | 37,179,288 |
| 11. ELECTRICAL ENERGY GENERATED (MWHe-Net) | 764,108 | 7,840,697 | 35,573,399 |
| 12. REACTOR SERVICE FACTOR (%) | 100.0 | 96.4 | 65.7 |
| 13. REACTOR AVAILABILITY FACTOR (%) | 100.0 | 96.4 | 68.4 |
| 14. UNIT SERVICE FACTOR (%) | 100.0 | 94.6 | 64.3 |
| 15. UNIT AVAILIBILITY FACTOR (%) | 100.0 | 94.6 | 64.3 |
| 16. UNIT CAPACITY FACTOR (USING MDC) (%) | 102.4 | 94.4 | 56.6 |
| 17. UNIT CAPACITY FACTOR (USING DESIGN MWe) (%) | 98.5 | 90.7 | 54.4 |
| 18. UNIT FORCED OUTAGE FACTOR (%) | 0.0 | 1.8 | 8.6 |

19. SHUTDOWNS SCHEDULED OVER THE NEXT 6 MONTHS (TYPE, DATE, AND DURATION OF EACH):
Refueling (L1R04) 02/23/91 10 Weeks

20. IF SHUTDOWN AT END OF REPORT PERIOD, ESTIMATED DATE OF STARTUP:
(N/A)

TABLE 4

F.2 AVERAGE DAILY UNIT POWER LEVEL (MWe-Net)

DOCKET NO. 050-373
UNIT LASALLE ONE
DATE DECEMBER 10,1990
COMPLETED BY M.J. CIALKOWSKI
TELEPHONE (815)-357-6761

REPORT PERIOD: NOVEMBER ,1990

| DAY | POWER | DAY | POWER |
|---|---|---|---|
| 1 | 1,09. | 17 | 1,093 |
| 2 | 1,093 | 18 | 1,093 |
| 3 | 1,093 | 19 | 1 083 |
| 4 | 1,091 | 20 | 1,043 |
| 5 | 1,091 | 21 | 1,011 |
| 6 | 1,087 | 22 | 848 |
| 7 | 1,095 | 23 | 1,010 |
| 8 | 1,094 | 24 | 1,094 |
| 9 | 1,093 | 25 | 1,093 |
| 10 | 1,094 | 26 | 1,085 |
| 11 | 1,097 | 27 | 1,072 |
| 12 | 1,093 | 28 | 1,057 |
| 13 | 1,085 | 29 | 922 |
| 14 | 996 | 30 | 1,037 |
| 15 | 1,096 | 31 | |
| 1. | 1,094 | | |

TABLE 5

F.3 UNIT SHUTDOWNS AND POWER REDUCTIONS > 20%
(Unit 1)

| YEARLY SEQUENTIAL NUMBER | DATE (YYMMDD) | TYPE F: FORCED S: SCHEDULED | DURATION (HOURS) | REASON | METHOD OF SHUTTING DOWN THE REACTOR OR REDUCING POWER | CORRECTIVE ACTIONS/COMMENTS (LER/DVR # if applicable) |
|---|---|---|---|---|---|---|
| (None) | | | | | | |

SUMMARY OF OPERATION:

The Unit remained on line at high power throughout the month. Several minor power reductions were required due to low grid demand, routine surveillances, Division 1 Battery Charger and CRD problems.

G. UNIQUE REPORTING REQUIREMENTS (Unit 1)

1. Safety/Relief valve operations

| DATE | VALVES ACTUATED | NATURE OF ACTUATION | PLANT CONDITION | DESCRIPTION OF EVENT |
|---|---|---|---|---|
| (None) | | | | |

2. ECCS System Outages
(See Table 6)

3. Changes to the Off-Site Dose Calculation Manual
(None)

4. Major changes to Radioactive Waste Treatment Systems.
(None)

5. Indications of Failed Fuel Elements.
(None)

(Unit 1)
Table 6

G.2 ECCS System Outages

Note: The year and unit data has been removed from the outage number.

| OUTAGE NO. | EQUIPMENT | PURPOSE |
|---|---|---|
| (U-0) | | |
| 377 | 0DG01K | Lubrication |
| (U-1) | | |
| 601 | 1E12-C300C<br>1E12-C300D | Lubricate pump coupling |
| 603 | 1E12-F093 | Administrative |
| 606 | 1E12-F026A | Perform Surveillance LES-EQ-112 |
| 608 | 1DG035 | Perform Surveillance LES-EQ-112 |
| 609 | 1E12-F026A | Valve inspection |
| 614 | 1E21-C001 | Change oil |
| 616 | 1DG035 | Valve inspection |
| 617 | 1E21-C001 | Megger Motor |
| 618 | 1E21-C001 | Perform LES-GM-129 |
| 623 | 1DG035 | Valve inspection |
| 625 | 1E12-F026A | Valve inspection |
| 632 | 1E12-C300B | Perform LES-GM-105 |
| 642 | 1E51-C004<br>1E51-C005 | Perform LES-DC-104 |
| 643 | 1E51-C003 | Lubricate coupling |
| 644 | 1E51-F360 | Remove turbine govenor |
| 660 | 1E22-D300 | Strainer inspection |

LASALLE NUCLEAR POWER STATION

UNIT 2

MONTHLY PERFORMANCE REPORT

NOVEMBER 1990

COMMONWEALTH EDISON COMPANY

NRC DOCKET NO. 050-374

LICENSE NO. NPF-18

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(Unit 2)

I. INTRODUCTION (Unit 2)

The LaSalle County Nuclear Power Station is a two-unit facility owned by Commonwealth Edison Company and located near Marseilles, Illinois. Each unit is a Boiling Water Reactor with a designed net electrical output of 1078 Megawatts. Waste heat is rejected to a man-made cooling pond using the Illinois River for make-up and blowdown. The architect-engineer was Sargent and Lundy and the primary construction contractor was Commonwealth Edison Company.

Unit Two was issued operating license number NPF-18 on December 16, 1983. Initial criticality was achieved on March 10, 1984 and commercial power operation was commenced on June 19, 1984.

This report was compiled by Michael J. Cialkowski, telephone number (815)357-6761 extension 2427.

II. MONTHLY REPORT

A. SUMMARY OF OPERATING EXPERIENCE (Unit 2)

| Day | Time | Event |
|---|---|---|
| 1 | 0000 | Reactor critical, generator on-line at 1130 MWE |
| 4 | 0100 | Reduced power to 850 MWE for heater bay work |
| | 1600 | Increased power to 1130 MWE |
| 13 | 0200 | Reduced power to 1070 MWE for condensate booster pump alignment |
| | 0800 | Increased power to 1130 MWE. |
| 17 | 0100 | Reduced load to 820 MWE for feedwater pump work and rod set. |
| | 0330 | Increased power to 980 MWE (holding power level due to feedwater pump problems) |
| 18 | 1600 | Increased power to 1130 MWE |
| 21 | 0030 | Reduced power to 1060 MWE for CRD exercising |
| | 1300 | Increased power to 1130 MWE |
| 28 | 0130 | Reduced power to 950 MWE for monthly surveillances |
| | 1100 | Increased power to 1130 MWE |
| 30 | 2400 | Reactor critical, generator on-line at 1130 MWE |

B. PLANT OR PROCEDURE CHANGES, TESTS OR EXPERIMENTS (50.59) (Unit 2)

1. Amendments to the Facility License or Technical Specification.
(None)

2. Changes to procedures which are described in the Safety Analysis Report.

(None.)

3. Changes to facility which are described in the Safety Analysis Report.
(None)

4. Tests and Experiments not described in the Safety Analysis Report.
(None)

C. MAJOR CORRECTIVE MAINTENANCE TO SAFETY RELATED EQUIPMENT (including SOR differential pressure switch failure reports).
(See Table 1)

D. COMPLETED SAFETY RELATED MODIFICATIONS.
(None)

E. LICENSEE EVENT REPORTS (Unit 2)
(None)

F. DATA TABULATIONS (Unit 2)

1. Operating Data Report.
(See Table 3)

2. Average Daily Unit Power Level.
(See Table 4)

3. Unit Shutdowns and Significant Power Reductions.
(See Table 5)

C TABLE 1 (Unit 2)

MAJOR CORRECTIVE MAINTENANCE TO
SAFETY-RELATED EQUIPMENT

| WORK REQUEST NUMBER | COMPONENT | CAUSE OF MALFUNCTION | RESULTS AND EFFECTS ON SAFE PLANT OPERATION | CORRECTIVE ACTION |
|---|---|---|---|---|
| L00694 | CRD HCU Accumulator | Pressure switch | Inaccurate accumulator pressure readings | Replaced pressure switch |
| L03897 | Div. 1 125V Battery Charger | Firing board | Power reduction per Tech Spec | Replace firing board |
| L91250 | RCIC Baro Vacuum Pump | Pump motor space heater | Motor oil improper temperature | Replaced space heater |

(See attached SOR failure report)

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SOR dp Switch Failure Data Sheet

Equipment Piece Number: 2E12-N010BA Model Number: 103-AS-B202-NX-JJTTX6

Serial Number: 85-6-3587R

Application: Residual Heat Removal Minimum Flow Switch

Date and Time of Discovery: 10/25/90 1430 hours

Reactor Mode: Run Power Level: 100%

Calibration Tolerance: 23.6-22.4"WC

Nominal Setpoint: 23.0"WC

Action Limits: <24.7 or >21.3"WC

Reject Limits: <17.7 or >28.3"WC

Technical Specification Limit: 5.90"WC

As Found Setpoint: ____"WC

Date and Time of Return to Service: 10/26/90 0450 hours

Model Number of Replacement Switch: 103AS-B202-NX-JJTTX6

Serial Number of Replacement Switch: 89-2-3874

DVR Number: 01-2-90-072

Cause: During routine calibration, pressure differential switch 2E12-N010BA setpoint was found to be erratic and not repeatable. During inspection of the switch, slight corrosion was found on the bearing and shaft. The cause of the failure was determined to be due to corrosion induced shaft binding.

Corrective Action: The switch was replaced, calibrated and returned to service. The failed switch was disassembled and inspected.

SOR dp Switch Failure Data Sheet

Equipment Piece Number: 2E31-N010B Model Number: 102-AS-B305-NX-JJTTX6

Serial Number: 85-1-2446

Application: Main Steam Line High Flow MSIV Isolation

Date and Time of Discovery: 11/20/90 1020 hours

Reactor Mode: Run Power Level: 100%

Calibration Tolerance: 102-104 PSID

Nominal Setpoint: 111 PSID

Action Limits: <98.2 or >107.8 PSID

Reject Limits: <94.8 or >111.2 PSID

Technical Specification Limit: 116 PSID

As Found Setpoint: 120 PSID

Date and Time of Return to Service: 11/27/90 2050 hours

Model Number of Replacement Switch: 102-AS-B305-NX-JJTTX6

Serial Number of Replacement Switch: 86-10-562

DVR Number: 01-2-90-073

Cause: Instrument setpoint drift.

Corrective Action: The switch was replaced, calibrated and returned to service. The failed switch was disassembled and inspected. Internal inspection of the switch was performed with no abnormalities found.

SOR dp Switch Failure Data Sheet

Equipment Piece Number: 2E31-N011B Model Number: 102-AS-403-NX-JJTTX6

Serial Number: 85-1-2450

Application: Main Steam Line High Flow MSIV isolation

Date and Time of Discovery: 11/28/90 1100 hours

Reactor Mode: Run Power Level: 100%

Calibration Tolerance: 102-104 PSID

Nominal Setpoint: 111 PSID

Action Limits: <98.2 or >107.8 PSID

Reject Limits: <94.8 or >111.2 PSID

Technical Specification Limit: 116 PSID

As Found Setpoint: 101 - 107 PSID

Date and Time of Return to Service: 11/30/90 1739 hours

Model Number of Replacement Switch: 102-AS-403-NX-JJTTX6

Serial Number of Replacement Switch: 86-10-563

DVR Number: 01-2-90-074

Cause: The switch was found to not maintain a constant setpoint. After disassembly and inspection, slight corrosion of the bearings and a scratch on the crossshaft was noted. The failure was due to corrosion induced shaft binding.

Corrective Action: The switch was replaced, calibrated and returned to service. The failed switch was disassembled and inspected.

TABLE 3

B.1 OPERATING DATA REPORT

DOCKET NO. 050-374
UNIT LASALLE TWO
DATE DECEMBER 10 1990
COMPLETED BY M.J. CIALKOWSKI
TELEPHONE (815)-357-6761

OPERATING STATUS

1. REPORTING PERIOD: NOVEMBER 1990 — GROSS HOURS IN REPORTING PERIOD: 720

2. CURRENTLY AUTHORIZED POWER LEVEL (MW): 3,323 — MAX DEPEND CAPACITY (MWe-Net): 1,036
DESIGN ELECTRICAL RATING (MWe-Net): 1,078

3. POWER LEVEL TO WHICH RESTRICTED (IF ANY) (MWe-Net): (NONE)

4. REASONS FOR RESTRICTION (IF ANY):

| | REPORTING PERIOD DATA | | |
|---|---|---|---|
| | THIS MONTH | YEAR-TO-DATE | CUMULATIVE |
| 5. REACTOR CRITICAL TIME (HOURS) | 720.0 | 5,695.1 | 35,042.2 |
| 6. REACTOR RESERVE SHUTDOWN TIME (HOURS) | 0.0 | 0.0 | 1,716.9 |
| 7. GENERATOR ON-LINE TIME (HOURS) | 720.0 | 5,556.6 | 35,239.7 |
| 8. UNIT RESERVE SHUTDOWN TIME (HOURS) | 0.0 | 0.0 | 0.0 |
| 9. THERMAL ENERGY GENERATED (MWH) | 2,361,312 | 17,350,967.2 | 103,679,015 |
| 10. ELECTRICAL ENERGY GENERATED (MWHe-Gross) | 806,000 | 5,837,672 | 34,280,598 |
| 11. ELECTRICAL ENERGY GENERATED (MWHe-Net) | 781,067 | 5,610,245 | 32,845,550 |
| 12. REACTOR SERVICE FACTOR (%) | 100.0 | 71.0 | 66.8 |
| 13. REACTOR AVAILABILITY FACTOR (%) | 100.0 | 71.0 | 70.0 |
| 14. UNIT SERVICE FACTOR (%) | 100.0 | 69.3 | 65.7 |
| 15. UNIT AVAILABILITY FACTOR (%) | 100.0 | 69.3 | 65.7 |
| 16. UNIT CAPACITY FACTOR (USING MDC) (%) | 104.7 | 67.6 | 59.1 |
| 17. UNIT CAPACITY FACTOR (USING DESIGN MWe) (%) | 100.6 | 64.9 | 56.8 |
| 18. UNIT FORCED OUTAGE FACTOR (%) | 0.0 | 5.8 | 14.8 |

19. SHUTDOWNS SCHEDULED OVER THE NEXT 6 MONTHS (TYPE, DATE, AND DURATION OF EACH):
(N/A)

20. IF SHUTDOWN AT END OF REPORT PERIOD, ESTIMATED DATE OF STARTUP:
(N/A)

DOCKET NO. 050-374
UNIT LASALLE TWO
DATE DECEMBER 10,1990
COMPLETED BY M.J. CIALKOWSKI
TELEPHONE (815)-357-6761

TABLE A
F.2 AVERAGE DAILY UNIT POWER LEVEL (MWe-Net)

REPORT PERIOD NOVEMBER ,1990

| DAY | POWER | DAY | POWER |
|---|---|---|---|
| 1 | 1,094 | 17 | 942 |
| 2 | 1,096 | 18 | 1,061 |
| 3 | 1,089 | 19 | 1,095 |
| 4 | 1,089 | 20 | 1,094 |
| 5 | 1,092 | 21 | 1,073 |
| 6 | 1,095 | 22 | 1,098 |
| 7 | 1,070 | 23 | 1,091 |
| 8 | 1,099 | 24 | 1,102 |
| 9 | 1,090 | 25 | 1,101 |
| 10 | 1,098 | 26 | 1,099 |
| 11 | 1,099 | 27 | 1,099 |
| 12 | 1,095 | 28 | 1,060 |
| 13 | 1,089 | 29 | 1,095 |
| 14 | 1,096 | 30 | 1,100 |
| 15 | 1,098 | 31 | |
| 16 | 1,083 | | |

TABLE 5

F.3 UNIT SHUTDOWNS AND POWER REDUCTIONS >20%
(UNIT 2)

| YEARLY SEQUENTIAL NUMBER | DATE (YYMMDD) | TYPE F: FORCED S: SCHEDULED | DURATION (HOURS) | REASON | METHOD OF SHUTTING DOWN THE REACTOR OR REDUCING POWER | CORRECTIVE ACTIONS/COMMENTS (LER/DVR # if applicable) |
|---|---|---|---|---|---|---|
| | | | | | | |

(None)

SUMMARY OF OPERATION:

The unit remained on line at high power throughout the month. Several minor power reductions were required due to low grid demand, routine surveillances, heater bay and feedwater pump work.

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C. UNIQUE REPORTING REQUIREMENTS (Unit 2)

1. Safety/Relief Valve Operations

| DATE | VALVES ACTUATED | NO & TYPE ACTUATIONS | PLANT CONDITION | DESCRIPTION OF EVENT |
|---|---|---|---|---|
| (None) | | | | |

2. ECCS System Outages
(See Table 6)

3. Changes to the Off-Site Dose Calculation Manual.
(None)

4. Major changes to Radioactive Waste Treatment Systems.
(None)

5. Indications of Failed Fuel Elements.
(None)

(Unit 2)
Table 6

G.2 ECCS System Outages

Note: The year and unit data has been removed from the outage number.

| OUTAGE NO. | EQUIPMENT | PURPOSE |
|---|---|---|
| 1764 | 2E51-F316 | Repack valve |
| 1767 | 2E12-C300A | Repair oil leak |
| 1781 | 2E22-C302A | Replaced motor |
| 1782 | 2E12-C002B | Change oil |
| 1787 | 2DG006 | Repack valve |
| 1792 | 2E22-D300 | Inspection |
| 1794 | 2E22-D300 | Replace gasket |

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