THC FORM 366 U. S. NUCLEAR REGULATORY COMMISSION FOLLOWUP REPORT 17.771 LICENSEE EVENT REPORT Previous report date 6/17/82 . CONTROL BLOCK: 10 (PLEASE PRINT OR TYPE ALL REQUIRED INFORMATION) 0 00000-00341111 17 (2)B İR 01 0 -LICENSEE CODE LICENSE NUMBER CON'T REPORT 0 0 0 2 5 9 7 0 3 2 7 8 2 3 1 0 1 3 8 2 0 1 L(6) 0 51 SOURCE DOCKET NUMBER EVENT DATE EVENT DESCRIPTION AND PROBABLE CONSEQUENCES (10) With unit 1 at 950 MW and increasing power at 10MW/hr, recirculation pump 1A tripped 0 2 due to spurious generator stator relay operation (T.S. 3.6.F.1). The relay was reset 03 after 20 minutes and the pump returned to service. There was no effect on public 0 4 health and safety. T.S. 3.6.F.1 allows operation for up to 24 hours with one 0 5 0 6 recirculation pump out of service. "B" pump was operable. 0 7 8 0 SYSTEM CAUSE CAUSE COMP VALVE CODE SUBCODE COMPON INT CODE SUBCODE SUBCODE 9 B Z (13) R Y X (14) A (15 ZI (16) 19 SEQUENTIAL OCCURRENCE REFORT REVISION LEARO EVENT YEAR REPORT NO. CODE TYPE NO. REPORT 8 2 0 2 10 13 2 3 XI NUMBER 28 3 ACTION FUTURE ACTION EFFEC ATTACHMENT SUBMITTED NPRD-4 PRIME COMP COMPONENT 22 ON PLANT METHOD HOURS FORM SUB SUPPLIER MANUFACTURER Z (21) Y 23 Х B 0 0 0 18) N (24) 0 N (25 G 10 18 10 47 CAUSE DESCRIPTION AND CORRECTIVE ACTIONS (27 10 Hi-temp relay (GE model 12IRT51C1A) tripped the generator. This is the only trip 1 1 since setpoints were raised in June 1981 after similar trips of pumps 2A, 2B, and 1 2 3A. Special tests of pumps 2A and 2B indicated no generic problem. Possible signal 1 3 shielding problems on generators for pumps 2A, 2B, 3A, and 3B were corrected. This 4 1A trip is considered a random failure. No other recurrence control planned. L 9 FACILITY 80 METHOD OF (30)% POWER OTHER STATUS DISCOVERY DESCRIPTION (32) F (28) 0 8 6 NA A (31) Operator observation ACTIVITY CONTENT 12 4.4 80 RELEASED OF RELEASE AMOUNT OF ACTIVITY (35 LOCATION OF HELEASE (36) Z (33) Z (34) 6 NA NA PERSONNEL EXPOSURES 80 NUMBER TYPE DESCRIPTION (39) 0 0 0 0 0 z 3 7 NA PERSONNEL INJURIES 13 80 NUMBER DESCRIPTION (41 0 0 (40) 0 NA LOSS OF OR DAMAGE TO FACILITY (43) 80 TYPE DESCRIPTION Z (42 2 NA PUBLICITY 8210290057 821013 80 LN 44 NRC USE ONLY PDR ADOCK 05000259 PDR S NA 68 69 80 Walter T. Christopher NAME OF PREPARER -(205) 729-0800 PHONE

Tennessee Valley Authority Browns Ferry Nuclear Plant

Form BF 17 BF 15.2

## LER SUPPLEMENTAL INFORMATION

BFRO-50- 259 / 82023 R2 Technical Specification Involved 3.6.F.1

Reported Under Technical Specification \_6.7.2.b.(2)\* Date Due NRC 09/01/82

## Event Narrative:

Unit 1 was operating at 950 MW and increasing power at 10 MW/hr; unit 2 was at 1100 MW; unit 3 was in a refueling outage. Units 2 and 3 were unaffected by this event. Recirculation pump 1A tripped due to generator stator temperature relay operation initiated by RTD high temperature. Surveillance Instructions 4.6.A.6 and 4.6.A.7 were performed and the pump returned to service. There was no effect on public health and safety. "1B" pump was operable and "1A" pump was returned to service within the time limits as specified by the Technical Specification 3.6.F.1. Six similar trips have occurred on pumps 2A, 2B, and 3A during the period from February 1980 to June 1981. Since unit 2 had the most trips, test was to monitor generator winding temperature under varying load conditions and identify possible generic problems. The resistance readings of RTD's on generators for pumps 1A, 1B, 2B, 3A, and 3B were obtained and each generator winding temperature recorded for reference. These measurements were taken in order to compare RTD's.

During the test, the ground wire connecting the RTD terminal strip ground bar to station ground on the generators for pumps 2A and 2B was found to be missing. These ground bars were connected to ground only by their mounting bolts. A ground wire was installed and generators for pumps 1A, 1B, 3A, and 3B were inspected for similar problems. It was discovered that generators for pumps 1A and 1B were grounded correctly but 3A and 3B were grounded only to the junction box frame. A ground wire was installed on 3A and 3B generators.

Test results and RTD resistance readings were evaluated by General Electric and TVA. Both concluded that there was no evidence of abnormal temperatures, localized hot spots, or generic problems. General Electric stated that this type \* Previous Similar Events: (See attached sheet)

BFR0-50-260/80007, 81009, 80018, 80023;

50-296/81028, 81031

RF

\*Revision:

Retention: Period - Lifetime; Responsibility - Document Control Supervisor

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of equipment has been used for years to monitor temperatures with no reported generic problems.

As a part of previous recurrence control, the generator stator temperature relay setpoints were raised from 110 degrees C to 120 degrees C in June 1981. This trip on generator 1A is the only trip to have occurred since that time.

Based on only this isolated occurrence since setpoints were raised, correction of possible shield grounding problems, and the results of the special test, it has been determined that no further recurrence control is required for this or the other listed previous similar events.

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