

ENCLOSURE 3

SEQUOYAH NUCLEAR PLANT

OFFSITE DOSE CALCULATION MANUAL

(REVISIONS 29, 30, AND 31)

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SEQUOYAH NUCLEAR PLANT

OFFSITE DOSE CALCULATION MANUAL

TENNESSEE VALLEY AUTHORITY

SEQUOYAH NUCLEAR PLANT
OFFSITE DOSE CALCULATION MANUAL
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Recommended by *Brian E. Lind-Lee* Date 10/13/93
RARC Chairman

Approved by *C. Hudson for WCM* Date 10/13/93
Manager, Technical Programs

- 1 Low Power license for Sequoyah Unit 1
- 2 RARC Meeting date
- 3 Date approved by RARC Chairman
- 4 Revision 23 implements the Nuclear Data Effluent Management Software. This ODCM revision and the software will be implemented concurrently on October 9, 1989. Releases made during the month of October prior to the software implementation will be backfitted to comply with this revision.
- 5 Revision 26 was recommended for approval by the SQN RARC at the October 2, 1991 meeting. The revision date is January 17, 1992. The final implementation date for Revision 26 will be March, 1992.
- 6 The implementation date for Revision 28 will be December 1, 1992.

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INTRODUCTION

The Sequoyah Nuclear Plant (SQN) Offsite Dose Calculation Manual (ODCM) is a supporting document of the SQN Technical Specifications. The ODCM is divided into two major parts. The first part of the ODCM contains:

- 1) Radioactive Effluent Controls required by Section 6.8.5.f of the SQN Technical Specifications;
- 2) Radiological Environmental Monitoring Controls required in Section 6.8.5.g of the SQN Technical Specifications;
- 3) descriptions of the information that should be included in the Annual Radiological Environmental Operating and Annual Radioactive Effluent Release Reports required by SQN Technical Specifications 6.9.1.6 and 6.9.1.8; and,
- 4) Administrative Controls for the ODCM requirements.

The second part of the ODCM contains the methodologies used to:

- 1) calculate offsite doses resulting from radioactive gaseous and liquid effluents;
- 2) calculate gaseous and liquid effluent monitor Alarm/Trip setpoints; and,
- 3) conduct the Environmental Radiological Monitoring Program.

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The SQN ODCM will be maintained for use as a reference guide on accepted methodologies and calculations. Changes in the calculation method, or parameters will be incorporated into the ODCM in order to assure that the ODCM represents the present methodology in all applicable areas. Any licensee initiated ODCM changes will be implemented in accordance with SQN Technical Specification 6.14 and ODCM Administrative Control 5.3.

RADIOACTIVE EFFLUENT/RADIOLOGICAL ENVIRONMENTAL MONITORING CONTROLS

SECTIONS 1.0 AND 2.0
CONTROLS AND
SURVEILLANCE REQUIREMENTS

1/2 CONTROLS AND SURVEILLANCE REQUIREMENTS

1/2.0 APPLICABILITY

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CONTROLS

- 1.0.1 Compliance with the Controls contained in the succeeding controls is required during the OPERATIONAL MODES or other conditions specified therein; except that upon failure to meet the Control, the associated ACTION requirements shall be met.
- 1.0.2 Noncompliance with a Control shall exist when the requirements of the Control and associated ACTION requirements are not met within the specified time intervals. If the Control is restored prior to the expiration of the specified intervals, completion of the ACTION requirements is not required.
- 1.0.3 When a Control is not met, except as provided in the associated ACTION requirements, within 1 hour action shall be initiated to place the unit in a MODE in which the control does not apply by placing it, as applicable, in:
- a. At least HOT STANDBY within the next 6 hours,
 - b. At least HOT SHUTDOWN within the following 6 hours, and
 - c. At least COLD SHUTDOWN within the subsequent 24 hours.
- When corrective measures are completed that permit operation under the ACTION requirements, the action may be taken in accordance with the specified time limits as measured from the time of failure to meet the control. Exceptions to these requirements are stated in the individual controls.
- 1.0.4 Entry into an OPERATIONAL MODE or other specified condition shall not be made when the conditions for the Control are not met and the associated ACTION requires a shutdown if they are not met within a specified time interval. Entry into an OPERATIONAL MODE or specified condition may be made in accordance with ACTION requirements when conformance to them permits continued operation of the facility for an unlimited period of time. This provision shall not prevent passage through or to OPERATIONAL MODES as required to comply with the ACTION requirements. Exceptions to these requirements are stated in the individual controls.

1/2 CONTROLS AND SURVEILLANCE REQUIREMENTS

1/2.0 APPLICABILITY

SURVEILLANCE REQUIREMENTS

- 2.0.1 Surveillance Requirements shall be met during the OPERATIONAL MODES or other conditions specified for individual Controls unless otherwise stated in the individual Surveillance Requirement.
- 2.0.2 Each Surveillance Requirement shall be performed within the specified time interval with a maximum allowable extension not to exceed 25% of the specified surveillance interval. R28
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- 2.0.3 Failure to perform a Surveillance Requirement within the specified time interval shall constitute a failure to meet the OPERABILITY requirements for a Control. The time limits of the ACTION requirements are applicable at the time it is identified that a Surveillance Requirement has not been performed. The ACTION requirements may be delayed for up to 24 hours to permit the completion of the surveillance when the allowable outage time limits of the ACTION requirements are less than 24 hours. Surveillance Requirements do not have to be performed on inoperable equipment. R31
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R31
- 2.0.4 Entry into an OPERATIONAL MODE or other specified applicable condition shall not be made unless the Surveillance Requirement(s) associated with the Control has been performed within the applicable surveillance interval or as otherwise specified. This provision shall not prevent passage through or to OPERATIONAL MODES as required to comply with ACTION requirements. Exceptions to these requirements are stated in the individual controls. R31
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1/2 CONTROLS AND SURVEILLANCE REQUIREMENTS

1/2.1 INSTRUMENTATION

1/2.1.1 RADIOACTIVE LIQUID EFFLUENT MONITORING INSTRUMENTATION

CONTROLS

1.1.1 In accordance with SQN Technical Specification 6.8.5.f.1, the radioactive liquid effluent monitoring instrumentation channels shown in Table 1.1-1 shall be OPERABLE with their alarm/trip setpoints set to ensure that the limits of ODCM Control 1.2.1.1 are not exceeded. The alarm/trip setpoints of these channels shall be determined in accordance with the methodology and parameters in ODCM Section 6.2.

APPLICABILITY: This requirement is applicable during all releases via these pathways.

ACTION:

- a. With a radioactive liquid effluent monitoring instrumentation channel alarm/trip setpoint less conservative than required above, without delay suspend the release of radioactive liquid effluents monitored by the affected channel or declare the channel inoperable, or change the setpoint so that it is acceptably conservative.
- b. With less than the minimum number of radioactive liquid effluent monitoring instrumentation channels OPERABLE, take the action shown in Table 1.1-1. Exert best effort to return the instruments to OPERABLE status within 30 days and, if unsuccessful, explain in the next Annual Effluent Release Report why the inoperability could not be corrected within 30 days. R30
- c. The provisions of Controls 1.0.3 and 1.0.4 are not applicable. R31
Report all deviations in the Annual Effluent Release Report. R31

SURVEILLANCE REQUIREMENTS

2.1.1 Each radioactive liquid effluent monitoring channel shall be demonstrated OPERABLE by performance of the CHANNEL CHECK, CHANNEL CALIBRATION, and CHANNEL FUNCTIONAL TEST operations at the frequencies shown in Table 2.1-1.

RADIOACTIVE EFFLUENT/RADIOLOGICAL ENVIRONMENTAL MONITORING CONTROLS

Table 1.1-1 (Page 1 of 3)
 RADIOACTIVE LIQUID EFFLUENT MONITORING INSTRUMENTATION

<u>Instrument</u>	<u>Minimum Channels OPERABLE</u>	<u>Action</u>	
1. GROSS RADIOACTIVITY MONITORS PROVIDING AUTOMATIC TERMINATION OF RELEASE			
a. Liquid Radwaste Effluent Line (0-RM-90-122)	1	30	
b. Steam Generator Blowdown Effluent Line (1,2-RM-90-120,121)	1	31	
c. Condensate Demineralizer Effluent Line (0-RM-90-225)	1	30	
2. GROSS RADIOACTIVITY MONITORS NOT PROVIDING AUTOMATIC TERMINATION OF RELEASE			
a. Essential Raw Cooling Water Effluent Header** (0-RM-90-133,-134,-140,-141)	1	32	
b. Turbine Building Sump Effluent Line (0-RM-90-212)	1	32	
3. FLOW RATE MEASUREMENT DEVICES			
a. Liquid Radwaste Effluent Line (0-F-77-42)	1	33	R29
b. Steam Generator Blowdown Effluent Line (0-FR-14-456, 0-F-14-185, 0-F-14-192)	1	33	R29
c. Condensate Demineralizer Effluent Line (1,2-FI-15-44, 1,2-FR-15-25, 1,2-F-15-43)	1	33	R29
d. Cooling Tower Blowdown Effluent Line (0-FT-27-175 or 0-LS-27-225))	1	33	R29
4. TANK LEVEL INDICATING DEVICES			
a. Condensate Storage Tank (0-L-2-230, 0-L-2-233)	1	34	R29
b. Steam Generator Layup Tank* (LOCAL FLOAT)	1	34	
5. CONTINUOUS COMPOSITE SAMPLER AND SAMPLE FLOW MONITOR			
a. Condensate Demineralizer Regenerant Effluent Line (0-FI-14-466)	1	35	

*Required when connected to the secondary system
 ** Requires minimum of 1 Channel/Header to be OPERABLE.

RADIOACTIVE EFFLUENT/RADIOLOGICAL ENVIRONMENTAL MONITORING CONTROLS

Table 1.1-1 (Page 2 of 3)
RADIOACTIVE LIQUID EFFLUENT MONITORING INSTRUMENTATION
TABLE NOTATION

- ACTION 30 - With the number of channels OPERABLE less than required by the Minimum Channels OPERABLE requirement, effluent releases may continue provided that prior to initiating a release:
- a. At least two independent samples are analyzed in accordance with ODCM Control 2.2.1.1, and
 - b. At least two technically qualified members of the Facility Staff independently verify the release rate calculations and discharge line valving;
- Otherwise, suspend release of radioactive effluents via this pathway.
- ACTION 31 - With the number of channels OPERABLE less than required by the Minimum Channels OPERABLE requirement, effluent releases via this pathway may continue provided grab samples are analyzed for gross radioactivity gamma at a limit of detection of at least 10^{-7} microcuries/gram:
- a. At least once per 12 hours when the specific activity of the secondary coolant is greater than or equal to 0.01 microcuries/gram DOSE EQUIVALENT I-131.
 - b. At least once per 24 hours when the specific activity of the secondary coolant is less than or equal to 0.01 microcuries/gram DOSE EQUIVALENT I-131.
- ACTION 32 - With the number of channels OPERABLE less than required by the Minimum Channels OPERABLE requirement, effluent releases via this pathway may continue provided that, at least once per 12 hours, grab samples are collected and analyzed for gross radioactivity gamma at a limit of detection of at least 10^{-7} microcuries/ml.
- ACTION 33 - With the number of channels OPERABLE less than required by the Minimum Channels OPERABLE requirement, effluent releases via this pathway may continued provided the flow rate is estimated at least once per 4 hours during actual releases. Pump curves may be used to estimate flow.

RADIOACTIVE EFFLUENT/RADIOLOGICAL ENVIRONMENTAL MONITORING CONTROLS

Table 1.1-1 (Page 3 of 3)
RADIOACTIVE LIQUID EFFLUENT MONITORING INSTRUMENTATION
TABLE NOTATION

- ACTION 34 - With the number of channels OPERABLE less than required by the Minimum Channels OPERABLE requirement, liquid additions to this tank may continued provided the tank liquid level is estimated during all liquid additions to the tank.
- ACTION 35 - With the number of channels OPERABLE less than required by the Minimum Channels OPERABLE requirement, effluent releases via this pathway may continue provided representative batch samples of each tank to be released are taken prior to release and composited for analysis according to Table 2.2-1, footnote g.

RADIOACTIVE EFFLUENT/RADIOLOGICAL ENVIRONMENTAL MONITORING CONTROLS

Table 2.1-1 (Page 1 of 2)
 RADIOACTIVE LIQUID EFFLUENT MONITORING INSTRUMENTATION
 SURVEILLANCE REQUIREMENTS

<u>Instrument</u>	<u>CHANNEL CHECK</u>	<u>SOURCE/ SENSOR CHECK</u>	<u>CHANNEL CALIBRATION</u>	<u>CHANNEL FUNCTIONAL TEST</u>	
1. GROSS BETA OR GAMMA RADIOACTIVITY MONITORS PROVIDING ALARM AND AUTOMATIC TERMINATION OF RELEASE					
a. Liquid Radwaste Effluent Line (0-RM-90-122)	D	P	R(3)	Q(1)	
b. Steam Generator Blowdown Effluent Line (1,2-RM-90-120,121)	D	M	R(3)	Q(5)	
c. Condensate Demineralizer Effluent Line (0-RM-90-225)	D	M	R(3)	Q(5)	
2. GROSS BETA OR GAMMA RADIOACTIVITY MONITORS PROVIDING ALARM BUT NOT PROVIDING AUTOMATIC TERMINATION OF RELEASE					
a. Essential Raw Cooling Water Effluent Line (0-RM-90-133,134,140,141)	D	M	R(3)	Q(2)	
b. Turbine Building Sump Effluent Line (0-RM-90-212)	D	M	R(3)	Q(2)	
3. FLOW RATE MEASUREMENT DEVICES					
a. Liquid Radwaste Effluent Line (0-F-77-42)	D(4)	N.A.	R	Q	R29
b. Steam Generator Blowdown Effluent Line (0-FR-14-456, 0-F-14-192, 0-F-14-185)	D(4)	N.A.	R	Q	R29
c. Condensate Demineralizer Effluent Line (1,2-FI-15-44, 1,2-FR-15-25, 1,2-F-15-43)	D(4)	N.A.	R	Q	R29
d. Cooling Tower Blowdown Effluent Line (0-FT-27-175, 0-L-27-225)	D(4)	N.A.	R	Q	R29
4. TANK LEVEL INDICATING DEVICES					
a. Condensate Storage Tank (0-LI-2-230, 0-L-2-233)	D*	N.A.	R	Q	R29
b. Steam Generator Layup Tank (LOCAL FLOAT)	D*	N.A.	R	N.A.	
5. CONTINUOUS COMPOSITE SAMPLER AND SAMPLE FLOW MONITOR					
a. Condensate Demineralizer Regenerant Effluent Line (0-FI-14-466)	P	N.A.	R	N.A.	

RADIOACTIVE EFFLUENT/RADIOLOGICAL ENVIRONMENTAL MONITORING CONTROLS

Table 2.1-1 (Page 2 of 2)
RADIOACTIVE LIQUID EFFLUENT MONITORING INSTRUMENTATION
SURVEILLANCE REQUIREMENTS
TABLE NOTATION

* During liquid additions to the tank.

- (1) The CHANNEL FUNCTIONAL TEST shall also demonstrate that automatic isolation of this pathway and control room alarm annunciation occurs if any of the following conditions exists:
 1. Instrument indicates measured levels above the alarm/trip setpoint.
 2. Circuit failure.
 3. Downscale failure.
- (2) The CHANNEL FUNCTIONAL TEST shall also demonstrate that control room alarm annunciation occurs if any of the following conditions exists:
 1. Instrument indicates measured levels above the alarm setpoint.
 2. Circuit failure.
 3. Downscale failure.
- (3) The initial CHANNEL CALIBRATION shall be performed using one or more of the reference standards certified by the National Bureau of Standards or using standards that have been obtained from suppliers that participate in measurement assurance activities with NBS. These standards shall permit calibrating the system over its intended range of energy and measurement range. For subsequent CHANNEL CALIBRATION, sources that have been related to the initial calibration shall be used.
- (4) CHANNEL CHECK shall consist of verifying indication of flow during periods of release. CHANNEL CHECK shall be made at least once per 24 hours on days on which continuous periodic, or batch releases are made.
- (5) The CHANNEL FUNCTIONAL TEST shall also demonstrate that automatic isolation of this pathway and control room alarm annunciation occurs if any of the following conditions occur:
 1. Instrument indicates measured levels above the alarm/trip setpoint.
 2. Circuit failure.

The CHANNEL FUNCTIONAL TEST shall also demonstrate that control room annunciation occurs if the following condition occurs:

1. Downscale failure.

1/2 CONTROLS AND SURVEILLANCE REQUIREMENTS

1/2.1 INSTRUMENTATION

1/2.1.2 RADIOACTIVE GASEOUS EFFLUENT MONITORING INSTRUMENTATION

CONTROLS

1.1.2 In accordance with SQN Technical Specification 6.8.5.f.1, the radioactive gaseous effluent monitoring instrumentation channels shown in Table 1.1-2 shall be OPERABLE with their alarm/trip setpoints set to ensure that the limits of ODCM Control 1.2.2.1 are not exceeded. The alarm/trip setpoints of these channels shall be determined in accordance with the methodology and parameters in ODCM Section 7.1.

APPLICABILITY: As shown in Table 1.1-2.

ACTION:

- a. With a radioactive gaseous effluent monitoring instrumentation channel alarm/trip setpoint less conservative than required above, without delay suspend the release of radioactive gaseous effluents monitored by the affected channel, declare the channel inoperable, or change the setpoint so it is acceptably conservative.
- b. With less than the minimum number of radioactive gaseous effluent monitoring instrumentation channels OPERABLE take the action shown in Table 1.1-2. Exert best efforts to return the instruments to OPERABLE status within 30 days and, if unsuccessful, explain in the next Annual Effluent Report why the inoperability could not be corrected within 30 days. R30
- c. The provisions of Controls 1.0.3 and 1.0.4 are not applicable. R31
Report all deviations in the Annual Effluent Release Report. R31

SURVEILLANCE REQUIREMENTS

2.1.2 Each radioactive gaseous effluent monitoring instrumentation channel shall be demonstrated OPERABLE by performance of the CHANNEL CHECK, SOURCE CHECK, CHANNEL CALIBRATION, and CHANNEL FUNCTIONAL TEST operations at the frequencies shown in Table 2.1-2.

Table 1.1-2 (Page 1 of 2)
 RADIOACTIVE GASEOUS EFFLUENT MONITORING INSTRUMENTATION

<u>Instrument</u>	<u>Minimum Channels OPERABLE</u>	<u>Applic- ability</u>	<u>Action</u>
1. WASTE GAS DISPOSAL SYSTEM			
a. Noble Gas Activity Monitor (0-RM-90-118)	1	*	40
b. Effluent System Flow Rate Measuring Device (0-FI-77-230)	1	*	41 R29
2. CONDENSER VACUUM EXHAUST SYSTEM			
a. Noble Gas Activity Monitor (1,2-RM-90-119)	1	*	42 R30
b. Flow Rate Monitor (1,2-F-2-256,257) (1,2-FI-2-101,148,191)	1	*	41 R29
3. SHIELD BUILDING EXHAUST SYSTEM			
a. Noble Gas Activity Monitor (1,2-RE-90-400A (Low Range))	1	***	42 R30
b. Iodine Sampler (1,2-RE-90-402)	1	***	44
c. Particulate Sampler (1,2-RE-90-402)	1	***	44
d. Flow Rate Monitor (1,2-FI-90-400) (1,2-FT-90-400)	1	***	41
e. Sampler Flow Rate Monitor (1,2-RI-90-400)	1	***	41
4. AUXILIARY BUILDING VENTILATION SYSTEM			
a. Noble Gas Activity Monitor (0-RM-90-101)	1	*	42
b. Iodine Sampler (0-RM-90-101)	1	*	44
c. Particulate Sampler (0-RM-90-101)	1	*	44
d. Flow Rate Monitor (0-FT-30-174)	1	*	41
e. Sampler Flow Rate Monitor (0-FS,FIS-90-101)	1	*	41 R30 R30
5. SERVICE BUILDING VENTILATION SYSTEM			
a. Noble Gas Activity Monitor (0-RM-90-132B)	1	*	42 R30
b. Flow Rate Monitor (0-FT-90-132, 0-FI-90-132)	1	*	41 R30 R30

RADIOACTIVE EFFLUENT/RADIOLOGICAL ENVIRONMENTAL MONITORING CONTROLS

Table 1.1-2 (Page 2 of 2)
RADIOACTIVE GASEOUS EFFLUENT MONITORING INSTRUMENTATION
TABLE NOTATION

* At all times.

*** During shield building exhaust system operation.

ACTION 40 - With the number of channels OPERABLE less than required by the Minimum Channels OPERABLE requirement, the contents of the tank(s) may be released to the environment provided that prior to initiating the release:

- a. At least two independent samples of the tank's contents are analyzed, and
- b. At least two technically qualified members of the Facility Staff independently verify the release rate calculations and discharge valve lineup;

Otherwise, suspend release of radioactive effluents via this pathway.

ACTION 41 - With the number of channels OPERABLE less than required by the Minimum Channels OPERABLE requirement, effluent releases via this pathway may continue provided the flow rate is estimated at least once per 4 hours.

ACTION 42 - With the number of channels OPERABLE less than required by the Minimum Channels OPERABLE requirement, effluent releases via this pathway may continue provided grab samples are taken at least once per 12 hours and these samples are analyzed for noble gas gross activity within 24 hours.

ACTION 44 - With the number of channels OPERABLE less than required by the Minimum Channels OPERABLE requirement, effluent releases via the affected pathway may continue provided that within 4 hours after the channel has been declared inoperable samples are continuously collected with auxiliary sampling equipment as required in Table 2.2-2.

Table 2.1-2 (Page 1 of 2)
 RADIOACTIVE GASEOUS EFFLUENT MONITORING INSTRUMENTATION
 SURVEILLANCE REQUIREMENTS

<u>Instrument</u>	<u>CHANNEL CHECK</u>	<u>SOURCE/ SENSOR CHECK</u>	<u>CHANNEL CALIBRATION</u>	<u>FUNCTIONAL TEST</u>	<u>MODES</u>	
					<u>CHANNEL in Which</u>	<u>Surveillance Required</u>
1. WASTE GAS DISPOSAL SYSTEM						
a. Noble Gas Activity Monitor (0-RM-90-118)	P	P	R(3)	Q(1)	*	
b. Flow Rate Monitor (0-FI-77-230)	D	N.A.	R	Q	****	R29
2. CONDENSER VACUUM EXHAUST SYSTEM						
a. Noble Gas Activity Monitor (1,2-RM-90-119)	D	M(4)	R(3)	Q(2)	*	R30
b. Flow Rate Monitor (1,2-F-2-256,257, 1,2-FI-2-101,148,191)	D	N.A.	R	Q	*	R29
3. SHIELD BUILDING EXHAUST SYSTEM						
a. Noble Gas Activity Monitor (1,2-RE-90-400A(Low Range))	D	M	R(3)	Q(2)	***	R30
b. Iodine Sampler (1,2-RE-90-402)	W	N.A.	N.A.	N.A.	***	
c. Particulate Sampler (1,2-RE-90-402)	W	N.A.	N.A.	N.A.	***	
d. Flow Rate Monitor (1,2-FI-90-400, 1,2-FT-90-400)	D	N.A.	R	Q	***	
e. Sampler Flow Rate Monitor (1,2-RI-90-400)	D	N.A.	R	Q	***	
4. AUXILIARY BUILDING VENTILATION SYSTEM						
a. Noble Gas Activity Monitor (0-RM-90-101)	D	M	R(3)	Q(2)	*	
b. Iodine Sampler (0-RM-90-101)	W	N.A.	N.A.	N.A.	*	
c. Particulate Sampler (0-RM-90-101)	W	N.A.	N.A.	N.A.	*	
d. Flow Rate Monitor (0-FT-30-174)	D	N.A.	R	Q	*	
e. Sampler Flow Rate Monitor (0-FIS,FS-90-101)	D	N.A.	R	Q	*	R30
5. SERVICE BUILDING VENTILATION SYSTEM						
a. Noble Gas Activity Monitor (0-RM-90-132)	D	M	R(3)	Q(2)	*	
b. Flow Rate Monitor (0-FT-90-132,0-FI-90-132)	D	N.A.	R	Q	*	R30

RADIOACTIVE EFFLUENT/RADIOLOGICAL ENVIRONMENTAL MONITORING CONTROLS

Table 2.1-2 (Page 2 of 2)
RADIOACTIVE GASEOUS EFFLUENT MONITORING INSTRUMENTATION
SURVEILLANCE REQUIREMENTS
TABLE NOTATION

- * At all times.
- *** During shield building exhaust system operation.
- **** During waste gas releases.

- (1) The CHANNEL FUNCTIONAL TEST shall also demonstrate that automatic isolation of this pathway and control room alarm annunciation occurs if any of the following conditions exists:
 - 1. Instrument indicates measured levels above the alarm/trip setpoint.
 - 2. Circuit failure.
 - 3. Downscale failure.
- (2) The CHANNEL FUNCTION TEST shall also demonstrate that control room alarm annunciation occurs if any of the following conditions exists:
 - 1. Instrument indicates measured levels above the alarm setpoint.
 - 2. Circuit failure.
 - 3. Downscale failure.

For the auxiliary building ventilation system, at least once every 18 months, the CHANNEL FUNCTIONAL TEST shall also demonstrate automatic isolation of this pathway if the following condition exists:

Instrument indicates measured levels above the alarm/trip setpoint.

- (3) The initial CHANNEL CALIBRATION shall be performed using one or more of the reference standards certified by the National Bureau of Standards or using standards that have been obtained from suppliers that participate in measurement assurance activities with NBS. These standards shall permit calibrating the system over its intended range of energy and measurement range. For subsequent CHANNEL CALIBRATION, sources that have been related to the initial calibration shall be used.
- (4) The SOURCE/SENSOR CHECK for the Condenser Vacuum Exhaust Monitor will be accomplished using an LED sensor check source in lieu of a radioactive source. R27
R27
R27

1/2 CONTROLS AND SURVEILLANCE REQUIREMENTS

1/2.2 RADIOACTIVE EFFLUENTS

1/2.2.1 LIQUID EFFLUENTS

1/2.2.1.1 CONCENTRATION

CONTROLS

1.2.1.1 In accordance with SN Technical Specifications 6.5.5.f.2 and 3, the concentration of radioactive material released to UNRESTRICTED AREAS (see Figure 3.1) shall be limited to the concentrations specified in 10 CFR Part 20, Appendix B, Table II, Column 2 for radionuclides other than dissolved or entrained noble gases. For dissolved or entrained noble gases, the concentration shall be limited to 2×10^{-4} microcuries/ml total activity.

APPLICABILITY: At all times.

ACTION:

- a. With the concentration of radioactive material released to UNRESTRICTED AREAS exceeding the above limits, without delay, restore the concentration to within the above limits.
- b. If samples or analyses required by Table 2.2-1 are not performed, report these in the next Annual Radioactive Effluent Release Report with an explanation why they were missed and what actions were taken to prevent reoccurrence. R29 R30 R29
- c. The provisions of Controls 1.0.3 and 1.0.4 are not applicable. R31

SURVEILLANCE REQUIREMENTS

- 2.2.1.1.1 Radioactive liquid wastes shall be sampled and analyzed according to the sampling and analysis program of Table 2.2-1.
- 2.2.1.1.2 The results of the radioactivity analysis shall be used in accordance with the methods in ODCM Section 6.1 to assure that the concentration at the point of release is maintained within the limits stated above.

RADIOACTIVE EFFLUENT/RADIOLOGICAL ENVIRONMENTAL MONITORING CONTROLS

Table 2.2-1 (Page 1 of 4)
 RADIOACTIVE LIQUID WASTE SAMPLING AND ANALYSIS PROGRAM

Liquid Release Type	Sampling Frequency	Minimum Analysis Frequency	Type of Activity Analysis	Lower Limit of Detection (LLD) ($\mu\text{Ci/ml}$) ^a			
A. Batch Waste Release Tanks ^d	P Each Batch	P Each Batch	Principal Gamma Emitters ^f	5×10^{-7}			
			I-131	1×10^{-6}			
	P One Batch/M	M	Dissolved/ Entrained Gases (Gamma Emitters)		1×10^{-5}		
				P Each Batch	M Composite ^b	H-3	1×10^{-5}
						Gross Alpha	1×10^{-7}
	P Each Batch	Q Composite ^b	Sr-89, Sr-90 Fe-55		5×10^{-8}		
					1×10^{-6}		
	B. Continuous Releases ^e	D Grab Sample	W Composite ^c	Principal Gamma Emitters ^f	5×10^{-7}		
				I-131	1×10^{-6}		
		M Grab Sample	M	Dissolved/ Entrained Gases (Gamma Emitters)		1×10^{-5}	
D Grab Sample					M Composite ^c	H-3	1×10^{-5}
		Gross Alpha	1×10^{-7}				
D Grab Sample		Q Composite ^c	Sr-89, Sr-90 Fe-55		5×10^{-8}		
					1×10^{-6}		

RADIOACTIVE EFFLUENT/RADIOLOGICAL ENVIRONMENTAL MONITORING CONTROLS

Table 2.2-1 (Page 2 of 4)
 RADIOACTIVE LIQUID WASTE SAMPLING AND ANALYSIS PROGRAM

Liquid Release Type	Sampling Frequency	Minimum Analysis Frequency	Type of Activity Analysis	Lower Limit of Detection (LLD) ($\mu\text{Ci/ml}$) ^a
C. Periodic Continuous Releases ^{e,h}	Continuous ^g	W Composite ^c	Principal Gamma Emitters ^f	5×10^{-7}
			I-131	1×10^{-6}
1. Non-Reclaimable Waste Tank	M ^g Grab Sample	M	Dissolved and entrained Gases (Gamma Emitters)	1×10^{-5}
2. High Crud Tanks (2)				
3. Neutralizer Tank	Continuous ^g	M Composite ^c	H-3	1×10^{-5}
			Gross Alpha	1×10^{-7}
	Continuous ^g	Q Composite ^c	Sr-89, Sr-90	5×10^{-8}
			Fe-55	1×10^{-6}

R27

RADIOACTIVE EFFLUENT/RADIOLOGICAL ENVIRONMENTAL MONITORING CONTROLS

Table 2.2-1 (Page 3 of 4)
RADIOACTIVE LIQUID WASTE SAMPLING AND ANALYSIS PROGRAM
TABLE NOTATION

- a The LLD is defined for the purpose of these specifications as the smallest concentration of radioactive material in a sample that will yield a net count above system background that will be detected with 95% probability with only a 5% probability of falsely concluding that a blank observation represents a "real" signal.

For a particular measurement system (which may include radiochemical separation):

$$LLD = \frac{4.66s_b}{E V 2.22 \times 10^6 Y \exp(-\lambda \Delta t)}$$

Where:

LLD is the "a priori" lower limit of detection as defined above in microcurie per unit mass or volume,

s_b is the standard deviation of the background counting rate or of the counting rate of a blank sample as appropriate (as counts per minute),

E is the counting efficiency as counts per disintegration,

V is the sample size in units of mass or volume,

2.22×10^6 is the number of disintegrations per minute per microcurie,

Y is the fractional radiochemical yield (when applicable),

λ is the radioactive decay constant for the particular radionuclide, and

Δt for plant effluents is the elapsed time between midpoint of sample collection and time of counting (midpoint).

It should be recognized that the LLD is defined as an a priori (before the fact) limit representing the capability of a measurement system and not an a posteriori (after the fact) limit for a particulate measurement.

- b A composite sample is one in which the quantity of liquid sampled is proportional to the quantity of liquid waste discharged and in which the method of sampling employed results in a specimen which is representative of the liquids released.
- c Prior to analyses, all samples taken for the composite shall be thoroughly mixed in order for the composite sample to be representative of the effluent release.

RADIOACTIVE EFFLUENT/RADIOLOGICAL ENVIRONMENTAL MONITORING CONTROLS

Table 2.2-1 (Page 4 of 4)
RADIOACTIVE LIQUID WASTE SAMPLING AND ANALYSIS PROGRAM
TABLE NOTATION

- d A batch release is the discharge of liquid wastes of a discrete volume. Prior to sampling for analyses, each batch shall be isolated, and then thoroughly mixed, by the method described in ODCM Section 6.1.1, to assure representative sampling.
- e A continuous release is the discharge of liquid wastes of a nondiscrete volume; e.g., from a volume or system that has an input flow during the continuous release.
- f The principal gamma emitters for which the LLD specification applies exclusively are the following radionuclides: Mn-54, Fe-59, Co-58, Co-60, Zn-65, Mo-99, Cs-134, Cs-137, Ce-141. Ce-144 shall also be measured with an LLD of 5×10^{-6} . This list does not mean that only these nuclides are to be detected and reported. Other peaks which are measurable and identifiable, together with the above nuclides, shall also be identified and reported.
- g Releases from these tanks are continuously composited during releases. With the composite sampler or the sampler flow monitor inoperable, the sampling frequency shall be changed to require representative batch samples from each tank to be released to be taken prior to release and manually composite for these analyses.
- h Applicable only during periods of primary to secondary leakage or the release of radioactivity as detected by the effluent radiation monitor provided the radiation monitor setpoint is at a LLD of 1×10^{-6} $\mu\text{Ci/ml}$ and allowing for background radiation during periods when primary to secondary leakage is not occurring.

1/2 CONTROLS AND SURVEILLANCE REQUIREMENTS

1/2.2 RADIOACTIVE EFFLUENTS

1/2.2.1 LIQUID EFFLUENTS

1/2.2.1.2 DOSE

CONTROLS

1.7.1.2 In accordance with SN Technical Specification 6.8.5.f.4 and 5, the dose or dose commitment to a MEMBER OF THE PUBLIC from radioactive materials in liquid effluents released to UNRESTRICTED AREAS shall be limited from each reactor unit:

- a. During any calendar quarter to less than or equal to 1.5 mrem to the total body and to less than or equal to 5 mrem to any organ, and
- b. During any calendar year to less than or equal to 3 mrem to the total body and to less than or equal to 10 mrem to any organ.

APPLICABILITY: At all times.

ACTION:

- a. With the calculated dose from the release of radioactive materials in liquid effluents exceeding any of the above limits, prepare and submit to the Commission within 30 days, pursuant to ODCM Administrative Control 5.4, a Special Report which identifies the cause(s) for exceeding the limit(s) and defines the corrective actions that have been taken to reduce the releases and the proposed corrective actions to be taken to assure that subsequent releases will be in compliance with the above limits. This Special Report shall also include (1) the results of radiological analyses of the drinking water source and (2) the radiological impact on finished drinking water supplies with regard to the requirements of 40 CFR 141 (applicable only if drinking water supply is taken from the receiving water body within three miles downstream of the plant discharge). R31
- b. The provisions of Controls 1.0.3 and 1.0.4 are not applicable. R31

SURVEILLANCE REQUIREMENTS

2.2.1.2 Cumulative dose contributions from liquid effluents for the current calendar quarter and current calendar year shall be determined in accordance with the methodology and parameters in ODCM Section 6.3 at least once per 31 days.

1/2 CONTROLS AND SURVEILLANCE REQUIREMENTS

1/2.2 RADIOACTIVE EFFLUENTS

1/2.2.1 LIQUID EFFLUENTS

1/2.2.1.3 LIQUID RADWASTE TREATMENT SYSTEM

CONTROLS

1.2.1.3 In accordance with SQN Technical Specification 6.8.5.f.6, the liquid radwaste treatment system shall be used to reduce the radioactive materials in liquid wastes prior to their discharge when the projected doses due to the liquid effluent to UNRESTRICTED AREAS (see Figure 3.1) would exceed 0.06 mrem per reactor unit to the total body or 0.2 mrem per reactor unit to any organ in a 31-day period.

APPLICABILITY: At all times.

ACTION:

a. With radioactive liquid waste being discharged without treatment and in excess of the above limits, prepare and submit to the Nuclear Regulatory Commission within 30 days pursuant to ODCM Administrative Control 5.4, a Special Report which includes the following information:

1. Identification of the inoperable equipment or subsystems and the reason for inoperability,
2. Action(s) taken to restore the inoperable equipment to OPERABLE status, and
3. Summary description of action(s) taken to prevent a recurrence.

b. The provisions of Controls 1.0.3 and 1.0.4 are not applicable.

R31

SURVEILLANCE REQUIREMENTS

2.2.1.3 Doses due to liquid releases from each unit to UNRESTRICTED AREAS shall be projected at least once per 31 days, in accordance with the methodology and parameters in ODCM Section 6.5.

1/2 CONTROLS AND SURVEILLANCE REQUIREMENTS

1/2.2 RADIOACTIVE EFFLUENTS

1/2.2.2 GASEOUS EFFLUENTS

1/2.2.2.1 DOSE RATE

CONTROLS

1.2.2.1 In accordance with SNQ Technical Specification 6.8.5.f.7, the dose rate due to radioactive materials released in gaseous effluents to areas at or beyond the SITE BOUNDARY (see Figure 3.1) shall be limited to the following:

- a. For noble gases: Less than or equal to 500 mrem/yr to the total body and less than or equal to 3000 mrem/yr to the skin, and
- b. For Iodine-131, Iodine-133, tritium, and for all radionuclides in particulate form with half-lives greater than 8 days: Less than or equal to 1500 mrem/yr to any organ.

APPLICABILITY: At all times.

ACTION:

- a. With dose rate(s) exceeding the above limits, without delay restore the release rate to within the above limit(s).
- b. If samples or analyses required by Table 2.2-1 are not performed, report these in the next Annual Radioactive Effluent Release Report with an explanation why they were missed and what actions were taken to prevent reoccurrence. R29 R30 R29
- c. The provisions of Controls 1.0.3 and 1.0.4 are not applicable. R31

SURVEILLANCE REQUIREMENTS

2.2.2.1.1 The dose rate due to noble gases in gaseous effluents shall be determined to be within the above limits in accordance with the methodology and parameters in ODCM Section 7.2.3, and

2.2.2.1.2 The dose rate due to I-131, I-133, tritium, and for all radionuclides in particulate form with half-lives greater than 8 days in gaseous effluents shall be determined to be within the above limits in accordance with the methodology and parameters in ODCM Section 7.2.4 and by obtaining representative samples and performing analyses in accordance with the sampling and analysis program specified in Table 2.2-2.

RADIOACTIVE EFFLUENT/RADIOLOGICAL ENVIRONMENTAL MONITORING CONTROLS

Table 2.2-2 (Page 1 of 4)
 RADIOACTIVE GASEOUS WASTE MONITORING SAMPLING AND ANALYSIS PROGRAM

Gaseous Release Type	Sampling Frequency	Minimum Analysis Frequency	Type of Activity Analysis	Lower Limit of Detection (LLD) ($\mu\text{Ci/ml}$) ^a
A. Waste Gas Storage Tank	P Each Tank Grab	P Each Tank	Principal Gamma Emitters ^g	1×10^{-4}
B. Containment 1. PURGE	p _i Each PURGE Grab Sample	p _i Each PURGE	Principal Gamma Emitters ^g H-3	1×10^{-4} 1×10^{-6}
2. Vent	d _j Each Day Grab Sample	d _j Each Day	Principal Gamma Emitters ^g H-3	1×10^{-4} 1×10^{-6}
C. Noble Gases and Tritium	M Grab Sample	M	Principal Gamma Emitters ^g	1×10^{-4}
1. Condenser Vacuum Exhaust ^h			H-3	1×10^{-6}
2. Auxiliary Building Exhaust ^{b,e}				
3. Service Bldg. Exhaust				
4. Shield Bldg. Exhaust ^{b,c,h}				

RADIOACTIVE EFFLUENT/RADIOLOGICAL ENVIRONMENTAL MONITORING CONTROLS

Table 2.2-2 (Page 2 of 4)
 RADIOACTIVE GASEOUS WASTE MONITORING SAMPLING AND ANALYSIS PROGRAM

Gaseous Release Type	Sampling Frequency	Minimum Analysis Frequency	Type of Activity Analysis	Lower Limit of Detection -LLD ($\mu\text{Ci/ml}$) ^a
D. Iodine and Particulates	Continuous ^f Sampler	W ^d Charcoal Sample	I-131	1×10^{-12}
1. Auxiliary Building Exhaust	Continuous ^f Sampler	W ^d Particulate Sample	Principal Gamma Emitters ^g (I-131, Others)	1×10^{-11}
2. Shield Building Exhaust	Continuous ^f Sampler	M Composite Particulate Sample	Gross Alpha	1×10^{-11}
	Continuous ^f Sampler	Q Composite Particulate Sample	Sr-89, Sr-90	1×10^{-11}
E. Noble Gases all Release types as listed in C	Continuous ^f Monitor	Noble Gas Monitor	Noble Gases Gross Beta or Gamma	1×10^{-6}

RADIOACTIVE EFFLUENT/RADIOLOGICAL ENVIRONMENTAL MONITORING CONTROLS

Table 2.2-2 (Page 3 of 4)
RADIOACTIVE GASEOUS WASTE MONITORING SAMPLING AND ANALYSIS PROGRAM
TABLE NOTATION

- a The LLD is defined, for the purpose of these Controls, as the smallest concentration of radioactive material in a sample that will yield a net count above system background that will be detected with 95% probability with only a 5% probability of falsely concluding that a blank observation represents a "real" signal.

For a particular measurement system (which may include radiochemical separation):

$$LLD = \frac{4.66s_b}{E V 2.22 \times 10^6 Y \exp(-\lambda \Delta t)}$$

Where:

LLD is the "a priori" lower limit of detection as defined above in microcurie per unit mass or volume,

s_b is the standard deviation of the background counting rate or of the counting rate of a blank sample as appropriate (as counts per minute),

E is the counting efficiency as counts per disintegration,

V is the sample size in units of mass or volume,

2.22×10^6 is the number of disintegrations per minute per microcurie,

Y is the fractional radiochemical yield (when applicable),

λ is the radioactive decay constant for the particular radionuclide, and

Δt is the elapsed time between midpoint of sample collection and time of counting (midpoint).

It should be noted that the LLD is defined as an a priori (before the fact) limit representing the capability of a measurement system and not an a posteriori (after the fact) limit for a particular measurement.

- b Sampling and analysis shall also be performed following shutdown, startup, or a thermal power change exceeding 15% of RATED THERMAL POWER within 1 hour unless (1) analysis shows that the DOSE EQUIVALENT I-131 concentration in the primary coolant has not increased more than a factor of 3 and (2) the containment noble gas activity monitor (RE-90-106 or RE-90-112) shows that the radioactivity has not increased by more than a factor of 3.
- c Tritium grab samples shall be taken at least once per 24 hours when the refueling canal is flooded.

Table 2.2-2 (Page 4 of 4)
RADIOACTIVE GASEOUS WASTE MONITORING SAMPLING AND ANALYSIS PROGRAM
TABLE NOTATION

- d Samples shall be changed at least once per 7 days and analyses shall be completed within 48 hours after changing (or after removal from sampler). Sampling shall also be performed at least once per 24 hours for at least 2 days following each shutdown from $\geq 15\%$ RATED THERMAL POWER, startup of $\geq 15\%$ RATED THERMAL POWER or THERMAL POWER change exceeding 15% of RATED THERMAL POWER in one hour and analyses shall be completed within 48 hours of changing. When samples collected for 24 hours are analyzed, the corresponding LLD's may be increased by a factor of 10.
- e Tritium grab samples shall be taken at least once per 7 days from the ventilation exhaust from the spent fuel pool area, whenever spent fuel is in the spent fuel pool.
- f The ratio of the sample flow rate to the sampled stream flow rate shall be known for the time period covered by each dose or dose rate calculation made in accordance with ODCM Sections 7.2, 7.3, and 7.4.
- g The principal gamma emitters for which the LLD specification applies exclusively are the following radionuclides: Kr-87, Kr-88, Xe-133, Xe-133m, Xe-135, and Xe-138 for noble gases and Mn-54, Fe-59, I-131, Co-58, Co-60, Zn-65, Mo-99, Cs-134, Cs-137, Ce-141 and Ce-144 for particulate principal gamma emitters. This list does not mean that only these nuclides are to be detected and reported. Other gamma peaks which are measurable and identifiable, together with the above nuclides, shall also be analyzed and reported in the Annual Radioactive Effluent Release Report pursuant to ODCM Administrative Control 5.2. R30
- h During releases via this exhaust system.
- i PURGING - Applicable in MODES 1, 2, 3 and 4, the upper and lower compartments of the containment shall be sampled prior to PURGING. Prior to breaking containment integrity in MODE 5 or 6, the upper and lower compartments of the containment shall be sampled. The incore instrument room purge sample shall be obtained at the shield building exhaust between 20 and 25 minutes following initiation of the incore instrument room PURGE.
- j VENTING - Applicable in MODES 1, 2, 3, and 4; the containment will be VENTED to the containment annulus and then to the auxiliary building via containment annulus fans. The lower containment compartment shall be sampled weekly when VENTING is to occur to account for the radioactivity being discharged from the VENTING process. R29

1/2 CONTROLS AND SURVEILLANCE REQUIREMENTS

1/2.2 RADIOACTIVE EFFLUENTS

1/2.2.2 GASEOUS EFFLUENTS

1/2.2.2.2 DOSE - NOBLE GASES

CONTROLS

1.2.2.2 In accordance with SQN Technical Specification 6.8.5.f.8, the air dose due to noble gases released in gaseous effluents from each reactor unit to areas at or beyond the SITE BOUNDARY (see Figure 3.1) shall be limited to the following:

- a. During any calendar quarter: Less than or equal to 5 mrad for gamma radiation and less than or equal to 10 mrad for beta radiation and
- b. During any calendar year: Less than or equal to 10 mrad for gamma radiation and less than or equal to 20 mrad for beta radiation.

APPLICABILITY: At all times.

ACTION:

- a. With the calculated air dose from radioactive noble gases in gaseous effluents exceeding any of the above limits, prepare and submit to the Nuclear Regulatory Commission within 30 days, pursuant to ODCM Administrative Control 5.4, a Special Report which identifies the cause(s) for exceeding the limit(s) and defines the corrective actions that have been taken to reduce the releases and the proposed corrective actions to be taken to assure that subsequent releases will be in compliance with the above limits. R31
- b. The provisions of Controls 1.0.3 and 1.0.4 are not applicable. R31

SURVEILLANCE REQUIREMENTS

2.2.2.2 Cumulative dose contributions for the current calendar quarter and current calendar year for noble gases shall be determined in accordance with the methodology and parameters in ODCM Section 7.3 at least once per 31 days.

1/2 CONTROLS AND SURVEILLANCE REQUIREMENTS

1/2.2 RADIOACTIVE EFFLUENTS

1/2.2.2 GASEOUS EFFLUENTS

1/2.2.2.3 DOSE - I-131, I-133, TRITIUM AND RADIONUCLIDES IN PARTICULATE FORM WITH HALF-LIVES GREATER THAN EIGHT DAYS

CONTROLS

1.2.2.3 In accordance with SQN Technical Specification 6.8.5.f.9, the dose to a MEMBER OF THE PUBLIC from I-131, I-133, tritium and all radionuclides in particulate form with half-lives greater than 8 days in gaseous effluents released to areas at or beyond the SITE BOUNDARY (see Figure 3.1) shall be limited to the following from each reactor unit:

- a. During any calendar quarter: Less than or equal to 7.5 mrem to any organ and.
- b. During any calendar year: Less than or equal to 15 mrem to any organ.

APPLICABILITY: At all times.

ACTION:

- a. With the calculated dose from the release of I-131, I-133, tritium and all radionuclides in particulate form with half-lives greater than 8 days, in gaseous effluents exceeding any of the above limits, prepare and submit to the Nuclear Regulatory Commission within 30 days, pursuant to ODCM Administrative Control 5.4, a Special Report which identifies the cause(s) for exceeding the limit(s) and defines the corrective actions that have been taken to reduce the releases and the proposed corrective actions to be taken to assure that subsequent releases will be in compliance with the above limits. R31
- b. The provisions of Controls 1.0.3 and 1.0.4 are not applicable. R31

SURVEILLANCE REQUIREMENTS

2.2.2.3 Cumulative dose contributions for the current calendar quarter and current calendar year for I-131, I-133, tritium and all radionuclides in particulate form with half-lives greater than 8 days shall be determined in accordance with the methodology and parameters in ODCM Section 7.4 at least once per 31 days.

1/2 CONTROLS AND SURVEILLANCE REQUIREMENTS

1/2.2 RADIOACTIVE EFFLUENTS

1/2.2.2 GASEOUS EFFLUENTS

1/2.2.2.4 GASEOUS RADWASTE TREATMENT

CONTROLS

1.2.2.4 In accordance with SQN Technical Specification 6.8.5.f.5 and 6, the GASEOUS RADWASTE TREATMENT SYSTEM and the VENTILATION EXHAUST TREATMENT SYSTEM shall be used to reduce radioactive materials in gaseous waste prior to their discharge when the projected gaseous effluent doses due to gaseous effluent releases to areas at or beyond The SITE BOUNDARY (see Figure 3.1), when averaged over 31 days, would exceed 0.2 mrad per unit for gamma radiation, and 0.4 mrad per unit for beta radiation. The appropriate portions of the VENTILATION EXHAUST TREATMENT SYSTEM shall be used to reduce radioactive materials in gaseous waste prior to their discharge when the projected doses due to gaseous effluents to areas at or beyond the SITE BOUNDARY (See Figure 3.1) when averaged over 31 days would exceed 0.3 mrem per unit to any organ.

ACTION:

- a. With the radioactive gaseous waste being discharged without treatment R31 for more than 31 days and in excess of the above limits, prepare and submit to the Commission within 30 days, pursuant to ODCM Administrative Control 5.4, a Special Report which includes the following information:
1. Identification of the inoperable equipment or subsystems and the reason for inoperability.
 2. Action(s) taken to restore the inoperable equipment to OPERABLE status, and
 3. Summary description of action(s) taken to prevent a recurrence.
- b. The provisions of Controls 1.0.3 and 1.0.4 are not applicable. R31

SURVEILLANCE REQUIREMENTS

2.2.2.4 Doses due to gaseous releases from the site shall be projected at least once per 31 days, in accordance with the methodology and parameters in ODCM Section 7.5.

1/2 CONTROLS AND SURVEILLANCE REQUIREMENTS

1/2.2 RADIOACTIVE EFFLUENTS

1/2.2.3 TOTAL DOSE

CONTROLS

1.2.3 In accordance with SQN Technical Specification 6.8.5.f.10, the annual (calendar year) dose or dose commitment to any MEMBER OF THE PUBLIC, due to releases of radioactivity from uranium fuel cycle sources, shall be limited to less than or equal to 25 mrem to the total body or any organ (except the thyroid, which shall be limited to less than or equal to 75 mrem).

APPLICABILITY: At all times.

ACTION:

- a. With the calculated doses from the release of radioactive materials in R31 liquid or gaseous effluents exceeding twice the limits of ODCM Control 1.2.1.2, 1.2.2.2, or 1.2.2.3, calculations should be made to determine whether the above limits have been violated. If such is the case, prepare and submit a Special Report to the Director, Nuclear Reactor Regulation, U.S. Nuclear Regulatory Commission, Washington D.C. 20555, within 30 days, which defines the corrective action to be taken to reduce subsequent releases to prevent recurrence of exceeding the limits above. This Special Report, as defined in 10 CFR Part 20.405c, shall include an analysis which estimates the radiation exposure (dose) to a MEMBER OF THE PUBLIC from uranium fuel cycle sources (including all effluent pathways and direct radiation) for a calendar year that includes the release(s) covered by this report. If the estimated dose(s) exceeds the above limits, and if the release condition resulting in violation of 40 CFR Part 190 has not already been corrected, the Special Report shall include a request for a variance in accordance with the provisions of 40 CFR Part 190 and include the specified information of Section 190.11(b). Submittal of the report is considered a timely request, and a variance is granted until the staff action on the request is complete.
- b. The provisions of Controls 1.0.3 and 1.0.4 are not applicable. R31

SURVEILLANCE REQUIREMENTS

2.2.3 Cumulative dose contributions from liquid and gaseous effluents shall be determined in accordance with the methodology and parameters in ODCM Sections 6.3, 7.3, and 7.4.

RADIOACTIVE EFFLUENT/RADIOLOGICAL ENVIRONMENTAL MONITORING CONTROLS

1/2 CONTROLS AND SURVEILLANCE REQUIREMENTS

1/2.3 RADIOLOGICAL ENVIRONMENTAL MONITORING

1/2.3.1 MONITORING PROGRAM

CONTROLS

1.3.1 In accordance with SQN Technical Specification 6.8.5.g.1, the radiological environmental monitoring program shall be conducted as specified in Table 2.3-1.

APPLICABILITY: At all times.

ACTION:

- a. With the radiological environmental monitoring program not being conducted as specified in Table 2.3-1, prepare and submit to the Commission, in the Annual Radiological Environmental Operating Report, a description of the reasons for not conducting the program as required and the plans for preventing a recurrence.
- b. With the level of radioactivity in an environmental sampling medium exceeding the reporting levels of Table 2.3-2 when averaged over any calendar quarter, prepare and submit to the Commission within 30 days from the end of the affected quarter, pursuant to ODCM Administrative Control 5.4, a Special Report that identifies the cause(s) for exceeding the limit(s) and defines the corrective actions to be taken to reduce radioactive effluents so that the potential annual dose to a MEMBER OF THE PUBLIC is less than the calendar year limits of ODCM Controls 1.2.1.2, 1.2.2.2 and 1.2.2.3. When one or more of the radionuclides in Table 2.3-2 is detected in the sampling medium, this report shall be submitted if:

$$\frac{\text{concentration}(1)}{\text{limit level}(1)} + \frac{\text{concentration}(2)}{\text{limit level}(2)} + \dots \geq 1.0$$

When radionuclides other than those in Table 2.3-2 are detected and are the result of plant effluents, this report shall be submitted if the potential annual dose to a MEMBER OF THE PUBLIC from all radionuclides is equal to or greater than the calendar year limits of ODCM Controls 1.2.1.2, 1.2.2.2, and 1.2.2.3. This report is not required if the measured level of radioactivity was not the result of plant effluents; however, in such an event, the condition shall be reported and described in the Annual Radiological Environmental Operating Report

1/2 CONTROLS AND SURVEILLANCE REQUIREMENTS

1/2.3 RADIOLOGICAL ENVIRONMENTAL MONITORING

1/2.3.1 MONITORING PROGRAM

ACTION (CONTINUED):

- c. With milk or fresh leafy vegetable samples unavailable from one or more of the sample locations required by Table 2.3-1, identify locations for obtaining replacement samples and add them to the radiological environmental monitoring program within 30 days. The specified locations from which samples were unavailable may then be deleted from the monitoring program. Pursuant to ODCM Administrative Control 5.1, identify the new locations for obtaining replacement samples in the Annual Radiological Environmental Operating Report. A revised figure(s) and table(s) for the ODCM reflecting the new location(s) shall be included in the next Annual Effluent Release Report pursuant to ODCM Administrative Control 5.2. R30
- d. The provisions of Controls 1.0.3 and 1.0.4 are not applicable. R31

SURVEILLANCE REQUIREMENTS

2.3.1 The radiological environmental monitoring samples shall be collected pursuant to Table 2.3-1 from the locations given in the tables and figures given in ODCM Section 9.0 and shall be analyzed pursuant to the requirements of Table 2.3-1 and the detection capabilities required by Table 2.3-3.

RADIOACTIVE EFFLUENT/RADIOLOGICAL ENVIRONMENTAL MONITORING CONTROLS

Table 2.3-1 (Page 1 of 3)
 MINIMUM REQUIRED RADIOLOGICAL ENVIRONMENTAL MONITORING PROGRAM

Exposure Pathway and/or Sample	Number and Location of Samples*	Sampling and Collection Frequency	Type and Frequency of Analysis
AIRBORNE Radioiodine and Particulates	Minimum of 5 locations	Continuous sampler** W	Radioiodine canister: W I-131. Particulate sampler: Analyze for gross beta radioactivity \geq 24 hours following filter change. Perform gamma isotopic analysis on each sample when gross beta activity is $>$ 10 times the yearly mean of control samples. Q Perform gamma isotopic analysis on composite (by location) sample.
DIRECT RADIATION	35 to 40 locations with \geq 2 dosimeters for continuously measuring and recording dose rate at each location.	Q	Q Gamma Dose

* Sample locations are given in Table 9.1.

** Continuous sampling with sample collection as required by dust loading, but at least once per 7 days.

RADIOACTIVE EFFLUENT/RADIOLOGICAL ENVIRONMENTAL MONITORING CONTROLS

Table 2.3-1 (Page 2 of 3)
 MINIMUM REQUIRED RADIOLOGICAL ENVIRONMENTAL MONITORING PROGRAM

Exposure Pathway and/or Sample	Number and Location of Samples*	Sampling and Collection Frequency	Type and Frequency of Analysis
WATERBORNE Surface	3 locations	M Composite** sample	Gamma isotopic Each composite sample
			Tritium analysis Q
Ground	2 locations	Q	Gamma isotopic and tritium analyses of each sample.
Drinking	Minimum of 1 location	M Composite** sample	Gross beta and gamma isotopic analysis
			Q Tritium analysis
	2 locations	M Grab sample	Gross beta and gamma isotopic analysis
Sediment from Shoreline locations	Minimum of 2 locations.	S	Gamma isotopic analysis of each sample.

* Sample locations are given in Table 9.1.
 ** Composite samples shall be collected by collecting an aliquot at intervals not exceeding 2 hours.

RADIOACTIVE EFFLUENT/RADIOLOGICAL ENVIRONMENTAL MONITORING CONTROLS

Table 2.3-1 (Page 3 of 3)
 MINIMUM REQUIRED RADIOLOGICAL ENVIRONMENTAL MONITORING PROGRAM

Exposure Pathway and/or Sample	Number and Location of Samples*	Sampling and Collection Frequency	Type and Frequency of Analysis	
INGESTION				
Milk	Milk from 3 locations. Samples of broad leaf vegetation at offsite location of highest D/Q if no milk samples are available.	B**	Gamma isotopic and I-131 analysis of each sample.	R28
Fish and Invertebrates	2 locations	One sample in season, or at least once per 184 days if not seasonal. One sample of each of the following species: Channel Catfish Crappie Smallmouth Buffalo	Gamma isotopic analysis on edible portions.	R28 R28
Food Products	Minimum of 2 locations	At time of harvest One sample of each of the following or similar classes of food products, as available 1. Lettuce and/or cabbage 2. Corn 3. Beans 4. Tomatoes	Gamma isotopic analysis on edible portion.	R28 R28

* Sample locations are given in Table 9.1.

** When animals are on pasture, at least once per 31 days at other times.

RADIOACTIVE EFFLUENT/RADIOLOGICAL ENVIRONMENTAL MONITORING CONTROLS

Table 2.3-2
 REPORTING LEVELS FOR RADIOACTIVITY CONCENTRATIONS IN ENVIRONMENTAL SAMPLES

<u>Analysis</u>	<u>Water</u> (pCi/L)	<u>Airborne</u> <u>Particulate</u> <u>or gases</u> (pCi/m ³)	<u>Fish</u> (pCi/kg, wet)	<u>Milk</u> (pCi/L)	<u>Food Products</u> (pCi/kg, wet)
H-3	2 x 10 ⁴ (a)	N.A.	N.A.	N.A.	N.A.
Mn-54	1 x 10 ³	N.A.	3 x 10 ⁴	N.A.	N.A.
Fe-59	4 x 10 ²	N.A.	1 x 10 ⁴	N.A.	N.A.
Co-58	1 x 10 ³	N.A.	3 x 10 ⁴	N.A.	N.A.
Co-60	3 x 10 ²	N.A.	1 x 10 ⁴	N.A.	N.A.
Zn-65	3 x 10 ²	N.A.	2 x 10 ⁴	N.A.	N.A.
Zr-Nb-95	4 x 10 ²	N.A.	N.A.	N.A.	N.A.
I-131	2 ^(b)	0.9	N.A.	3	1 x 10 ²
Cs-134	30	10	1 x 10 ³	60	1 x 10 ³
Cs-137	50	20	2 x 10 ³	70	2 x 10 ³
Ba-La-140	2 x 10 ²	N.A.	N.A.	3 x 10 ²	N.A.

(a) For drinking water samples. This is 40 CFR Part 141 value. If no drinking water pathway exists, a value of 30,000 pCi/L may be used.

(b) If no drinking water pathway exists, a value of 20 pCi/L may be used.

RADIOACTIVE EFFLUENT/RADIOLOGICAL ENVIRONMENTAL MONITORING CONTROLS

Table 2.3-3 (Page 1 of 2)
 MAXIMUM VALUES FOR THE LOWER LIMITS OF DETECTION (LLD)^{a,b}

Analysis	Water (pCi/L)	Airborne Particulate or Gases (pCi/m ³)	Fish (pCi/kg, wet)	Milk (pCi/L)	Food Products (pCi/kg, wet)	Sediment (pCi/kg, dry)
gross beta	4	1x10 ⁻²	N.A.	N.A.	N.A.	N.A.
H-3	2000*	N.A.	N.A.	N.A.	N.A.	N.A.
Mn-54	15	N.A.	130	N.A.	N.A.	N.A.
Fe-59	30	N.A.	260	N.A.	N.A.	N.A.
Co-58,60	15	N.A.	130	N.A.	N.A.	N.A.
Zn-65	30	N.A.	260	N.A.	N.A.	N.A.
Zr-95	30	N.A.	N.A.	N.A.	N.A.	N.A.
Nb-95	15	N.A.	N.A.	N.A.	N.A.	N.A.
I-131	1**	7x10 ⁻²	N.A.	1	60	N.A.
Cs-134	15	5x10 ⁻²	130	15	60	150
Cs-137	18	6x10 ⁻²	150	18	80	180
Ba-140	60	N.A.	N.A.	60	N.A.	N.A.
La-140	15	N.A.	N.A.	15	N.A.	N.A.

* If no drinking water pathway exists, a value of 3000 pCi/L may be used.

** If no drinking water pathway exists, a value of 15 pCi/L may be used.

RADIOACTIVE EFFLUENT/RADIOLOGICAL ENVIRONMENTAL MONITORING CONTROLS

Table 2.3-3 (Page 2 of 2)
MAXIMUM VALUES FOR THE LOWER LIMITS OF DETECTION (LLD)^{a,b}
TABLE NOTATION

a The LLD is defined, for the purpose of these Controls, as the smallest concentration of radioactive material in a sample that will yield a net count above system background that will be detected with 95% probability with only a 5% probability of falsely concluding that a blank observation represents a "real" signal.

For a particular measurement system (which may include radiochemical separation):

$$LLD = \frac{4.66s_b}{E \quad V \quad 2.22 \quad Y \quad \exp(-\lambda\Delta t)}$$

Where:

LLD is the "a priori" lower limit of detection as defined above in picocurie per unit mass or volume,
 s_b is the standard deviation of the background counting rate or of the counting rate of a blank sample as appropriate (as counts per minute),
E is the counting efficiency as counts per disintegration,
V is the sample size in units of mass or volume,
2.22 is the number of disintegrations per minute per picocurie,
Y is the fractional radiochemical yield (when applicable),
 λ is the radioactive decay constant for the particular radionuclide, and
 Δt for environmental samples is the elapsed time between sample collection (or end of the sample collection period) and time of counting.

It should be recognized that the LLD is defined as an a priori (before the fact) limit representing the capability of a measurement system and not an a posteriori (after the fact) limit for a particular measurement. Analysis will be performed in such a manner that the stated LLDs will be achieved under routine conditions.

b Other peaks which are measurable and identifiable, together with the radionuclides above, shall be identified and reported.

1/2 CONTROLS AND SURVEILLANCE REQUIREMENTS

1/2.3 RADIOLOGICAL ENVIRONMENTAL MONITORING

1/2.3.2 LAND USE CENSUS

CONTROLS

2.3.2 In accordance with SQN Technical Specification 6.8.5.g.2, a Land Use Census shall be conducted and shall identify within a distance of 8 km (5 miles) the location in each of the 16 meteorological sectors of the nearest milk animal, the nearest residence, and the nearest garden^a of greater than 50 m² (500 ft²) producing fresh leafy vegetables.

^aBroad leaf vegetation sampling of at least three different kinds of vegetation may be performed at the SITE BOUNDARY in each of two different direction sectors with the highest predicted D/Qs in lieu of the garden census. Specifications for broad leaf vegetation sampling in Table 2.3-1 shall be followed, including analysis of control samples.

APPLICABILITY: At all times.

ACTION:

- a. With a Land Use Census identifying a location(s) that yields a calculated dose or dose commitment 20% greater than at a location from which doses are currently being calculated in ODCM Section 7.3 and 7.4 identify the new location(s) in the next Annual Effluent Release Report pursuant to ODCM Administrative Control 5.2. R30
- b. With a Land Use Census identifying a location(s) that yields a calculated dose or dose commitment (via the same pathway) 20% greater than at a location from which samples are currently being obtained in accordance with the requirements of ODCM Control 1.3.1, add the new location(s) within 30 days to the radiological environmental monitoring program given in ODCM Section 9.0, if samples are available. The sampling location(s), excluding the control station location, having the lowest calculated dose or dose commitment(s), via the same exposure pathway, may be deleted from this monitoring program after October 31 of the year in which this Land Use Census was conducted. Pursuant to ODCM Administrative Controls 5.2 and 5.3, submit in the next Annual Effluent Release Report documentation for a change in the ODCM including a revised figure(s) and table(s) for the ODCM reflecting the new location(s) with the information supporting the change in sampling locations. R30
- c. The provisions of Controls 1.0.3 and 1.0.4 are not applicable. R31

RADIOACTIVE EFFLUENT/RADIOLOGICAL ENVIRONMENTAL MONITORING CONTROLS

1/2 CONTROLS AND SURVEILLANCE REQUIREMENTS

1/2.3 RADIOLOGICAL ENVIRONMENTAL MONITORING

1/2.3.2 LAND USE CENSUS

SURVEILLANCE REQUIREMENTS

- 2.3.2 The Land Use Census shall be conducted during the growing season at least once per 12 months using that information that will provide the best results, such as by a door-to-door survey, mail survey, telephone survey, aerial survey, or by consulting local agricultural authorities. The results of the Land Use Census shall be included in the Annual Radiological Environmental Operating Report pursuant to ODCM Administrative Control 5.1.

1/2 CONTROLS AND SURVEILLANCE REQUIREMENTS

1/2.3 RADIOLOGICAL ENVIRONMENTAL MONITORING

1/2.3.3 INTERLABORATORY COMPARISON PROGRAM

CONTROLS

1.3.3 In accordance with SQN Technical Specification 6.8.5.g.3, analyses shall be performed on radioactive materials supplied as part of an Interlaboratory Comparison Program which has been approved by the Commission.

APPLICABILITY: At all times.

ACTION:

- a. With analyses not being performed as required above, report the corrective actions being taken to prevent a recurrence to the Commission in the Annual Radiological Environmental Operating Report pursuant to ODCM Administrative Control 5.1. R31
- b. The provisions of Controls 1.0.3 and 1.0.4 are not applicable. R31

SURVEILLANCE REQUIREMENTS

2.3.3 A summary of the results obtained as a part of the above required Interlaboratory Comparison Program and in accordance with the guidance below shall be included in the Annual Radiological Environmental Operating Report pursuant to ODCM Administrative Control 5.1.

RADIOACTIVE EFFLUENT/RADIOLOGICAL ENVIRONMENTAL MONITORING CONTROLS

BASES FOR
SECTIONS 1.0 AND 2.0
CONTROLS
AND
SURVEILLANCE REQUIREMENTS

NOTE

The BASES contained in succeeding pages summarize the reasons for the Controls in Sections 1.0 and 2.0, but are not part of these Controls.

RADIOACTIVE EFFLUENT/RADIOLOGICAL ENVIRONMENTAL MONITORING CONTROLS

BASES

INSTRUMENTATION

1/2.1.1 RADIOACTIVE LIQUID EFFLUENT MONITORING INSTRUMENTATION

The radioactive liquid effluent instrumentation is provided to monitor and control, as applicable, the releases of radioactive materials in liquid effluents during actual or potential releases of liquid effluents. The alarm/trip setpoints for these instruments shall be calculated in accordance with the procedures in ODCM Section 6.2 to ensure that the alarm/trip will occur prior to exceeding the limits of 10 CFR Part 20. The OPERABILITY and use of this instrumentation is consistent with the requirements of General Design Criteria 60, 63, and 64 of Appendix A to 10 CFR Part 50.

1/2.1.2 RADIOACTIVE GASEOUS EFFLUENT MONITORING INSTRUMENTATION

The radioactive gaseous effluent instrumentation is provided to monitor and control, as applicable, the releases of radioactive materials in gaseous effluents during actual or potential releases of gaseous effluents. The alarm/trip setpoints for these instruments shall be calculated in accordance with the procedures in ODCM Section 7.1 to ensure that the alarm/trip will occur prior to exceeding the limits of 10 CFR Part 20. The OPERABILITY and use of this instrumentation is consistent with the requirements of General Design Criteria 60, 63, and 64 of Appendix A to 10 CFR Part 50.

1/2.2 RADIOACTIVE EFFLUENTS

1/2.2.1.1 CONCENTRATION

This Control is provided to ensure that the concentration of radioactive materials released in liquid waste effluents to UNRESTRICTED AREAS will be less than the concentration levels specified in 10 CFR Part 20, Appendix B, Table II, Column 2. This limitation provides additional assurance that the levels of radioactive materials in bodies of water in UNRESTRICTED AREAS will result in exposures within (1) the Section II.A design objectives of Appendix I, 10 CFR 50, to a MEMBER OF THE PUBLIC and (2) the limits of 10 CFR 20.106(e) to the population. The concentration limit for dissolved or entrained noble gases is based upon the assumption that Xe-135 is the controlling radioisotope and its MPC in air (submersion) was converted to an equivalent concentration in water using the methods described in International Commission of Radiological Protection (ICRP) Publication 2.

1/2.2.1.2 DOSE

This Control is provided to implement the requirements of Sections II.A, III.A, and IV.A of Appendix I, 10 CFR Part 50. The requirement implements the guide set forth in Section II.A of Appendix I. The action statements provide the required operating flexibility and at the

RADIOACTIVE EFFLUENT/RADIOLOGICAL ENVIRONMENTAL MONITORING CONTROLS

BASES

1/2.2 RADIOACTIVE EFFLUENTS

1/2.2.1.2 DOSE (continued)

same time implement the guides set forth in Section IV.A of Appendix I to assure that the releases of radioactive materials in liquid effluents will be kept "as low as reasonable achievable." Also, for fresh water sites with drinking water supplies which can be potentially affected by plant operations, there is reasonable assurance that the operation of the facility will not result in radionuclide concentrations in the finished drinking water that are in excess of the requirements of 40 CFR 141. The dose calculations in ODCM Section 6.3 implement the requirements in Section III.A of Appendix I that conformance with the guides of Appendix I be shown by calculational procedures based on models and data, such that the actual exposure of a MEMBER OF THE PUBLIC through appropriately modeled pathways is unlikely to substantially underestimated. The equations specified in Section 6.3 for calculating the doses due to the actual release rates of radioactive materials in liquid effluents are consistent with the methodology provided in Regulatory Guide 1.109, "Calculation of Annual Doses to Man from Routine Releases of Reactor Effluents for the Purposes of Evaluating Compliance with 10 CFR Part 50, Appendix I," Revision 1, October 1977 and Regulatory Guide 1.113, "Estimating Aquatic Dispersion of Effluents from Accidental and Routine Reactor Releases for the Purpose of Implementing Appendix I," April 1977.

This Control applies to the release of liquid effluents from each reactor at the site. For units with shared radwaste treatment systems, the liquid effluents from the shared systems are proportioned among the units sharing that system.

1/2.2.1.3 LIQUID RADWASTE TREATMENT SYSTEM

The Control that the appropriate portions of this system be used when specified provides assurance that the releases of radioactive materials in liquid effluents will be kept "as low as reasonable achievable." This requirement implements the requirements of 10 CFR Part 50.36a, General Design Criteria 60 of Appendix A to 10 CFR Part 50 and the design objective given in Section II.D of Appendix I to 10 CFR Part 50. The specified limits governing the use of appropriate portions of the liquid radwaste system were specified as a suitable fraction of the dose design objectives set forth in Section II.A of Appendix I, 10 CFR Part 50, for liquid effluents.

1/2.2.2.1 DOSE RATE

This Control is provided to ensure that the dose at any time at the SITE BOUNDARY from gaseous effluents from all units on the site will be within the annual dose limits of 10 CFR Part 20. The annual dose

RADIOACTIVE EFFLUENT/RADIOLOGICAL ENVIRONMENTAL MONITORING CONTROLS

BASES

1/2.2 RADIOACTIVE EFFLUENTS

1/2.2.2.1 DOSE RATE (continued)

limits are the doses associated with the concentrations of 10 CFR Part 20, Appendix B, Table II, Column 1. These limits provide reasonable assurance that radioactive material discharged in gaseous effluents will not result in the exposure of a MEMBER OF THE PUBLIC, either within or outside the SITE BOUNDARY, to annual average concentrations exceeding the limits specified in Appendix B, Table II of 10 CFR Part 20 (10 CFR Part 20.106(b)). For MEMBERS OF THE PUBLIC who may at times be within the SITE BOUNDARY, the occupancy of the individual will be sufficiently low to compensate for any increase in the atmospheric diffusion factor above that for the SITE BOUNDARY. The specified release rate limits restrict, at all times, the corresponding gamma and beta dose rates above background to an individual at or beyond the SITE BOUNDARY to less than or equal to 500 mrem/yr to the total body or to less than or equal to 3000 mrem/yr to the skin. These release rate limits also restrict, at all times, the corresponding thyroid dose rate above background to an infant via the cow-milk-infant pathway to less than or equal to 1500 mrem/yr for the nearest cow to the plant. This requirement applies to the release of gaseous effluents from all reactors at the site. For units with shared radwaste treatment systems, the gaseous effluents from the shared systems are proportioned among the units sharing that system.

1/2.2.2.2 DOSE - NOBLE GASES

This requirement is provided to implement the requirements of Sections II.B, III.A and IV.A of Appendix I, 10 CFR Part 50. The requirement implements the guides set forth in Section II.B of Appendix I. The action to be taken provide the required operating flexibility and at the same time implements the guides set forth in Section IV.A of Appendix I to assure that the releases of radioactive material in gaseous effluents will be kept "as low a reasonably achievable." The surveillance implements the requirements in Section III.A of Appendix I that conformance with the guides of Appendix I be shown by calculational procedures based on models and data such that the actual exposure of a MEMBER OF THE PUBLIC through appropriately modeled pathways is unlikely to be substantially underestimated. The dose calculations established in ODCM Section 7.3 for calculating the doses due to the actual release rates of radioactive noble gases in gaseous effluents are consistent with the methodology provided in Regulatory Guide 1.109, "Calculation of Annual Doses to Man from Routine Releases of Reactor Effluents for the Purposes of Evaluating Compliance with 10 CFR Part 50, Appendix I," Revision 1, October 1977 and Regulatory Guide 1.111, "Methods for Estimating Atmospheric Transport and Dispersion of Gaseous Effluents in Routine Releases from Light-Water

RADIOACTIVE EFFLUENT/RADIOLOGICAL ENVIRONMENTAL MONITORING CONTROLS

BASES

1/2.3 RADIOACTIVE EFFLUENTS

1/2.2.2.2 DOSE - NOBLE GASES (continued)

Cooled Reactors," Revision 1, July 1977. The ODCM equations provided for determining the air doses at the SITE BOUNDARY are based upon the historical average atmospheric conditions.

1/2.2.2.3 DOSE - IODINE-131, IODINE-133, TRITIUM, AND RADIOACTIVE MATERIAL IN PARTICULATE FORM WITH HALF-LIFE GREATER THAN EIGHT DAYS

This Control is provided to implement the requirements of Sections II.C, III.A and IV.A of Appendix I, 10 CFR Part 50. The requirement implements the guides set forth in Section II.C of Appendix I. The action to be taken provides the required operating flexibility and at the same time implements the guides set forth in Section IV.A of Appendix I to assure that the releases of radioactive material in gaseous effluents will be kept "as low as reasonably achievable." Section 7.4 calculational methods implement the requirements in Section III.A of Appendix I that conformance with the guides of Appendix I be shown by calculational procedures based on models and data such that the actual exposure of a MEMBER OF THE PUBLIC through appropriately modeled pathways is unlikely to be substantially underestimated. Section 7.4 calculational methods for calculating the doses due to the actual release rates of the subject materials are consistent with the methodologies provided in NUREG/CR-1004, "A Statistical Analysis of Selected Parameters for Predicting Food Chain Transport and Internal Dose of Radionuclides," October 1979 and Regulatory Guide 1.109, "Calculation of Annual Doses to Man from Routine Releases of Reactor Effluents for the Purposes of Evaluating Compliance with 10 CFR Part 50, Appendix I," Revision 1, October 1977 and Regulatory Guide 1.111, "Methods for Estimating Atmospheric Transport and Dispersion of Gaseous Effluents in Routine Releases from Light-Water Cooled Reactors," Revision 1, July 1977. These equations also provide for determining the actual doses based upon the historical average atmospheric conditions. The release rate specifications for I-131, I-133 tritium and all radionuclides in particulate form with half-lives greater than 8 days are dependent on the existing radionuclide pathways to man, beyond the SITE BOUNDARY. The pathways which were examined in the development of these calculations were: 1) individual inhalation of airborne radionuclides, 2) deposition of radionuclides onto green leafy vegetation with subsequent consumption by man, 3) deposition onto grassy areas where milk animals and meat producing animals graze with consumption of the milk and meat by man, and 4) deposition on the ground with subsequent exposure of man.

RADIOACTIVE EFFLUENT/RADIOLOGICAL ENVIRONMENTAL MONITORING CONTROLS

BASES

1/2.3 RADIOACTIVE EFFLUENTS

1/2.3.2.4 GASEOUS RADWASTE TREATMENT SYSTEM

This Control that the appropriate portions of these systems be used, when specified, provides reasonable assurance that the releases of radioactive materials in gaseous effluents will be kept "as low as reasonably achievable." This Control implements the requirements of 10 CFR Part 50.36a, General Design Criteria 60 of Appendix A to 10 CFR Part 50, and the design objectives given in Section II.D of Appendix I to 10 CFR Part 50. The specified limits governing the use of appropriate portions of the systems were specified as a suitable fraction of the dose design objectives set forth in Section II.B and II.C of Appendix I, 10 CFR Part 50, for gaseous effluents.

1/2.2.3 TOTAL DOSE

This Control is provided to meet the dose limitations of 40 CFR Part 190 that have been incorporated into 10 CFR Part 20 by 46 FR 18525. The ACTION requires the preparation and submittal of a Special Report whenever the calculated doses due to releases of radioactivity and to radiation from uranium fuel cycle sources exceed 25 mrem to the total body or any other organ except thyroid, which shall be limited to less than or equal to 75 mrem. For sites containing up to 4 reactors, it is highly unlikely that the resultant dose to a MEMBER OF THE PUBLIC will exceed the dose limits of 40 CFR 190 if the individual reactors remain within twice the dose design objectives of Appendix I and if direct radiation doses from the units and from outside storage tanks are kept small. The Special Report will describe a course of action that should result in the limitation of annual dose to a MEMBER OF THE PUBLIC to within the 40 CFR Part 190 limits. For the purposes of the Special Report, it may be assumed that the dose commitment to the MEMBER OF THE PUBLIC from other uranium fuel cycle sources is negligible, with the exception that dose contributions from other nuclear fuel cycle facilities at the same site or within a radius of 8 km must be considered.

If the dose to any MEMBER OF THE PUBLIC is estimated to exceed the requirements of 40 CFR Part 190, the Special Report with a request for a variance (provide the release conditions resulting in violation of 40 CFR Part 190 have not already been corrected), in accordance with the provisions of 40 CFR 190.11 and 10 CFR 20.405c, is considered to be a timely request and fulfills the requirements of 40 CFR Part 190 until NRC staff action is completed. The variance only relates to the limits of 40 CFR Part 190, and does not apply in any way to the other requirements for dose limitation of 10 CFR Part 20, as addressed in ODCM Controls 1.2.1.1 and 1.2.2.1. An individual is not considered a MEMBER OF THE PUBLIC during any period in which he/she is engaged in carrying out any operation that is a part of the nuclear fuel cycle.

RADIOACTIVE EFFLUENT/RADIOLOGICAL ENVIRONMENTAL MONITORING CONTROLS

BASES

1/2.3 RADIOLOGICAL ENVIRONMENTAL MONITORING

1/2.3.1 MONITORING PROGRAM

The radiological environmental monitoring program required by this Control provides representative measurements of radiation and of radioactive materials in those exposure pathways and for those radionuclides that lead to the highest potential radiation exposures of MEMBERS OF THE PUBLIC resulting from the station operation. This monitoring program implements Section IV.B.2 of Appendix I to 10 CFR Part 50 and thereby supplements the radiological effluent monitoring program by verifying that the measurable concentration of radioactive materials and levels of radiation are not higher than expected on the basis of the effluent measurements and modeling of the environmental exposure pathways. Guidance for this monitoring program is provided by the Radiological Assessment Branch Technical Position on Environmental Monitoring.

The required detection capabilities for environmental sample analyses are tabulated in terms of the lower limits of detection (LLDs). The LLDs required by Table 2.3-3 are considered optimum for routine environmental measurements in industrial laboratories. It should be recognized that the LLD is defined as an a priori (before the fact) limit representing the capability of a measurement system and not as a posteriori (after the fact) limit for a particular measurement.

Detailed description of the LLD, and other detection limits can be found in HASL Procedures Manual, HASL-300 (revised annually), Curie, L. A., "Limits for Qualitative Detection and Quantitative Determination - Application to Radiochemistry," Anal. Chem. 40, 586-93 (1968), and Hartwell, J. K., "Detection Limits for Radioanalytical Counting Techniques," Atlantic Richfield Hanford Company Report ARH-SA-215 (June 1975).

1/2.3.2 LAND USE CENSUS

This Control is provided to ensure that changes in the use of unrestricted areas are identified and that modifications to the monitoring program are made if required by the results of that census. The best survey information from the door-to-door, aerial, or consulting with local agricultural authorities shall be used. This census satisfies the requirements of Section IV.B.3 of Appendix I to 10 CFR Part 50. Restricting the census to gardens of greater than 500 ft² provides assurance that significant exposure pathways via leafy vegetables will be identified and monitored since a garden of this size is the minimum required to provide the quantity (26 kg/yr) of leafy vegetables assumed in Regulatory Guide 1.109 for consumption by a child. To determine this minimum garden size, the following assumptions were used, 1) that 20% of the garden was used for growing broad leaf vegetation (i.e., similar to lettuce and cabbage), and 2) a vegetation yield of 2 kg/m².

RADIOACTIVE EFFLUENT/RADIOLOGICAL ENVIRONMENTAL MONITORING CONTROLS

BASES

1/2.3 RADIOLOGICAL ENVIRONMENTAL MONITORING

1/2.3.3 INTERLABORATORY COMPARISON

The requirement for participation in an Interlaboratory Comparison Program is provided to ensure that independent checks on the precision and accuracy of the measurements of radioactive material in environmental sample matrices are performed as part of the quality assurance program for environmental monitoring in order to demonstrate that the results are reasonably valid.

RADIOACTIVE EFFLUENT/RADIOLOGICAL ENVIRONMENTAL MONITORING CONTROLS

SECTION 3.0

DEFINITIONS

RADIOACTIVE EFFLUENT/RADIOLOGICAL ENVIRONMENTAL MONITORING CONTROLS

3.0 DEFINITIONS

The defined terms in this section appear in capitalized type in the text and are applicable throughout this ODCM.

3.1 CHANNEL CALIBRATION

A CHANNEL CALIBRATION shall be the adjustment, as necessary, of the channel output such that it responds with the necessary range and accuracy to known values of the parameter which the channel monitors. The CHANNEL CALIBRATION shall encompass the entire channel including the sensor and alarm and/or trip functions, and shall include the CHANNEL FUNCTIONAL TEST. The CHANNEL CALIBRATION may be performed by any series of sequential, overlapping, or total channel steps such that the entire channel is calibrated.

3.2 CHANNEL CHECK

A channel check shall be the qualitative assessment of channel behavior during operation by observation. This determination shall include, where possible, comparison of the channel indication and/or status with other indications and/or status derived from independent instrument channels measuring the same parameter.

R27

3.3 CHANNEL FUNCTIONAL TEST

A CHANNEL FUNCTIONAL TEST shall be:

- a. Analog channels - the injection of a simulated signal into the channel as close to the sensor as practicable to verify OPERABILITY including alarm and/or trip functions.
- b. Bistable channel - the injection of a simulated signal into the sensor to verify OPERABILITY including alarm and/or trip function.

3.4 DOSE EQUIVALENT I-131

DOSE EQUIVALENT I-131 shall be that concentration of I-131 ($\mu\text{Ci}/\text{gram}$) which alone would produce the same thyroid dose as the quantity and isotopic mixture of I-131, I-132, I-133, I-134, and I-135 actually present. The thyroid dose conversion factors used for this calculation shall be those listed in Table III of TID-14844, "Calculation of Distance Factors for Power and Test Reactor Sites."

Reformatting/Renumbering Changes only

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RADIOACTIVE EFFLUENT/RADIOLOGICAL ENVIRONMENTAL MONITORING CONTROLS

3.0 DEFINITIONS (continued)

3.5 GASEOUS RADWASTE TREATMENT SYSTEM

A GASEOUS RADWASTE TREATMENT SYSTEM is any system designed and installed to reduce radioactive gaseous effluents by collecting primary coolant system offgases from the primary system and providing for delay or holdup for the purpose of reducing the radioactivity prior to release to the environment.

3.6 MEMBER(S) OF THE PUBLIC

MEMBER(S) OF THE PUBLIC shall include all individuals who are not occupationally associated with the plant. This category shall include non-employees of the licensee who are permitted to use portions of the site for recreational, occupational, or other purposes not associated with plant functions. This category does not include non-employees such as vending machine servicemen or postmen who, as part of their formal job function, occasionally enter an area that is controlled by the licensee for purposes of protection of individuals from exposure to radiation and radioactive materials.

3.7 OPERABLE - OPERABILITY

A system, subsystem, train, component, or device shall be OPERABLE or have OPERABILITY when it is capable of performing its specified function(s), and when all necessary attendant instrumentation, controls, a normal and an emergency electrical power source, cooling or seal water, lubrication or other auxiliary equipment that are required for the system, subsystem, train, component, or device to perform its function(s) are also capable of performing their related support function.

R27

3.8 MODE

A MODE shall correspond to any one inclusive combination of core reactivity condition, power level, and average reactor coolant temperature specified in Table 1.1 of the SQN Technical Specifications.

3.9 PURGE - PURGING

PURGE or PURGING is the controlled process of discharging air or gas from a confinement to maintain temperature, pressure, humidity, concentration or other operating condition, in such a manner that replacement air or gas is required to purify the confinement.

R27

Reformatting/Renumbering Changes only

R26

RADIOACTIVE EFFLUENT/RADIOLOGICAL ENVIRONMENTAL MONITORING CONTROLS

3.0 DEFINITIONS (continued)

3.10 RATED THERMAL POWER

RATED THERMAL POWER shall be a total reactor core heat transfer rate to the reactor coolant of 3411 MWt.

3.11 SITE BOUNDARY

The SITE BOUNDARY shall be that line beyond which the land is not owned, leased, or otherwise controlled by the licensee (see Figure 3.1)

3.12 SOURCE/SENSOR CHECK

A SOURCE/SENSOR CHECK shall be the qualitative assessment of channel response when the channel sensor is exposed to a radioactive source or other channel sensor internal test circuit.

R27

R27

R27

3.13 UNRESTRICTED AREA

An UNRESTRICTED AREA shall be any area, at or beyond the SITE BOUNDARY to which access is not controlled by the licensee for purposes of protection of individuals from exposure to radiation and radioactive materials or any area within the SITE BOUNDARY used for residential quarters or industrial, commercial, institutional, and/or recreational purposes.

3.14 VENTILATION EXHAUST TREATMENT SYSTEM

A VENTILATION EXHAUST TREATMENT SYSTEM is any system designed and installed to reduce gaseous radioiodine or radioactive material in particulate form in effluents by passing ventilation or vent exhaust gases through charcoal adsorbers and/or HEPA filters for the purpose of removing iodines or particulates from the gaseous exhaust stream prior to the release to the environment (such a system is not considered to have any effect on noble gas effluents). Engineered Safety Feature (ESF) atmospheric cleanup systems are not considered to be VENTILATION EXHAUST TREATMENT SYSTEM components.

3.15 VENTING

VENTING is the controlled process of discharging air or gas from a confinement to maintain temperature, pressure, humidity, concentration, or other operating condition, in such a manner that replacement air gas is not provided or required during VENTING. Vent, used in system names, does not imply a VENTING process.

RADIOACTIVE EFFLUENT/RADIOLOGICAL ENVIRONMENTAL MONITORING CONTROLS

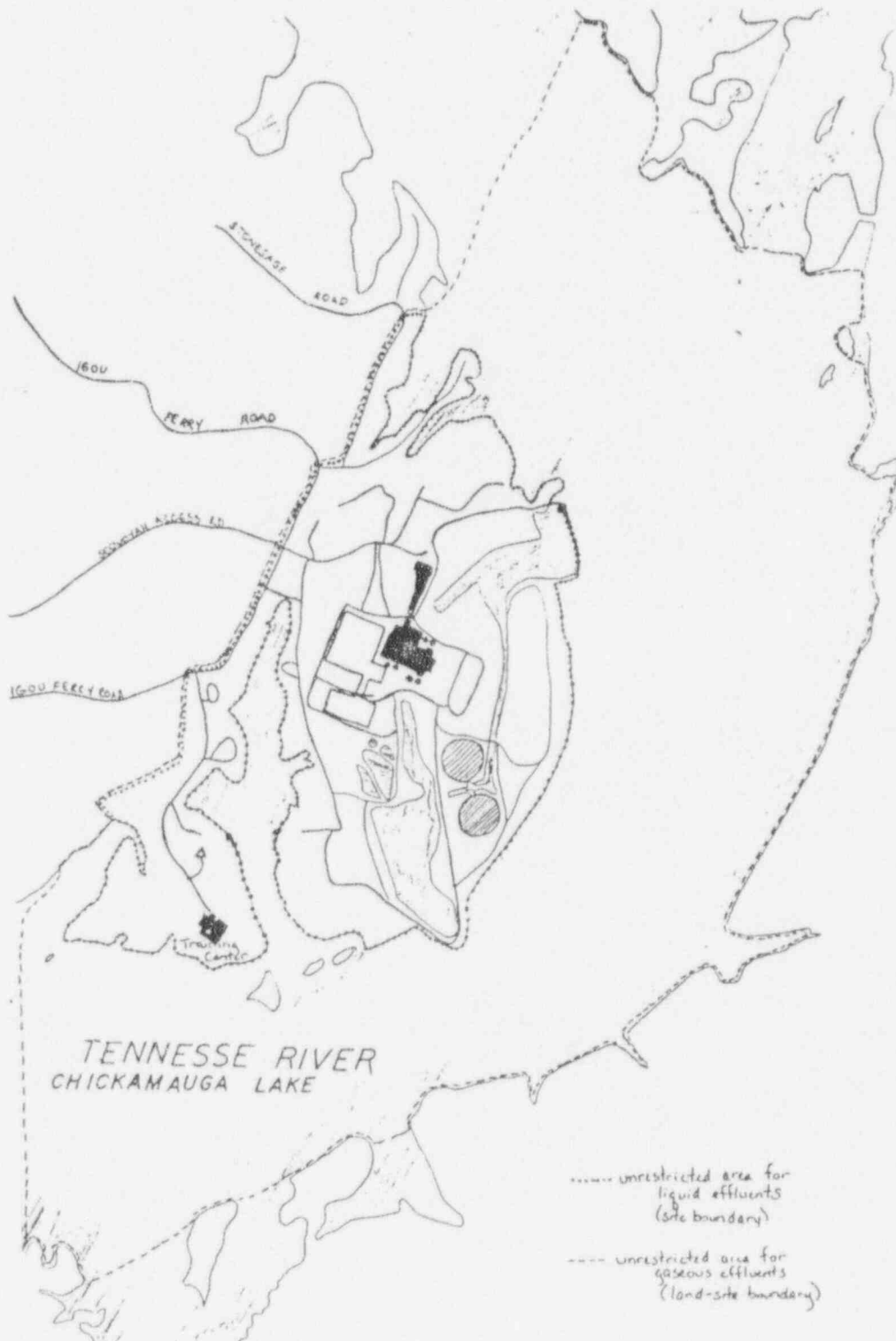
Table 3.1
FREQUENCY NOTATION

- P = Completed prior to each release
- D = At least once per 24 hours
- B = At least once per 15 days
- M = At least once per 31 days
- Q = At least once per 92 days
- R = At least once per 18 months
- N.A. = Not Applicable

R27

RADIOACTIVE EFFLUENT/RADIOLOGICAL ENVIRONMENTAL MONITORING CONTROLS

Figure 3.1
SITE BOUNDARY/UNRESTRICTED AREA



RADIOACTIVE EFFLUENT/RADIOLOGICAL ENVIRONMENTAL MONITORING CONTROLS

SECTION 4.0

(NOT USED)

RADIOACTIVE EFFLUENT/RADIOLOGICAL ENVIRONMENTAL MONITORING CONTROLS

SECTION 5.0

ADMINISTRATIVE CONTROLS

5.0 ADMINISTRATIVE CONTROLS

5.1 ANNUAL RADIOLOGICAL ENVIRONMENTAL OPERATING REPORT

As required by SQN Technical Specification 6.9.1.6, Routine Annual Radiological Environmental Operating Reports covering the operation of the unit during the previous calendar year shall be submitted prior to May 1 of each year.

The annual radiological environmental operating reports shall include summaries, interpretations, and an analysis of trends of the results of the radiological environmental surveillance activities for the report period, including a comparison with preoperational studies, operational controls (as appropriate), and previous environmental surveillance reports and an assessment of the observed impacts of the plant operation on the environment. The reports shall also include the results of land use censuses required by ODCM Control 1.3.2 and a listing of the new locations for dose calculations and/or environmental monitoring identified by the land use census. If harmful effects or evidence of irreversible damage are detected by the monitoring, the report shall provide an analysis of the problems and a planned course of action to alleviate the problem.

The annual radiological environmental operating reports shall include summarized and tabulated results in the format of Regulatory Guide 4.8, December 1975 of all radiological environmental samples taken during the report period. In the event that some results are not available for inclusion with the report, the report shall be submitted noting and explaining the reasons for the missing results. The missing data shall be submitted as soon as possible in a supplementary report.

The reports shall also include the following: a summary description of the radiological environmental monitoring program; a map of all sampling locations keyed to a table giving distances and directions from one reactor; and the results of licensee participation in the Interlaboratory Comparison Program required by ODCM Control 1.3.3.

5.2 ANNUAL RADIOACTIVE EFFLUENT RELEASE REPORT

R30

As required by SQN Technical Specification 6.9.1.8, an Annual Radioactive Effluent Release Report covering the operation of the unit during the previous 12 months of operation shall be submitted prior to May 1 of each year.

R30

R30

R30

5.0 ADMINISTRATIVE CONTROLS

5.2 ANNUAL RADIOACTIVE EFFLUENT RELEASE REPORT (continued)

R30

Annual radioactive release reports shall include a summary of the quantities of radioactive liquid and gaseous effluents and solid waste released from the unit as outlined in Regulatory Guide 1.21, "Measuring, Evaluating, and Reporting Radioactivity in Solid Wastes and Releases of Radioactive Materials in Liquid and Gaseous Effluents from Light-Water-Cooled Nuclear Power Plants," Revision 1, June 1974, with data summarized on a quarterly basis following the format of Appendix B thereof.

R30

The annual radioactive release report shall include unplanned releases from the site to unrestricted areas on a quarterly basis and shall also include any changes made to the ODCM pursuant to ODCM Administrative Control 5.3.

R30

The annual radioactive release report shall include information for solid waste as outlined in the Process Control Program, and shall also include any changes made to the PCP during the reporting period.

R30

The annual radioactive release report shall include a discussion of any licensee initiated major changes to the radioactive waste systems as required by SQN Technical Specification 6.15.1.1.

R30

The annual radioactive effluent release report (Radiological Impact) shall include an annual summary of hourly meteorological data collected over the previous year. This annual summary may be either in the form of an hour-by-hour listing of wind speed, wind direction, atmospheric stability, and precipitation (if measured) on magnetic tape, or in the form of joint frequency distributions of wind speed, wind direction, and atmospheric stability. In lieu of submission with the annual radioactive effluent release report, this summary of required meteorological data may be retained on site in a file that shall be provided to NRC upon request). This same report shall include an assessment of the radiation doses due to radioactive liquid and gaseous effluents released from the unit or station during the previous calendar year. This same report shall also include an assessment of the radiation doses from radioactive liquid and gaseous effluents to MEMBERS OF THE PUBLIC due to their activities inside the SITE BOUNDARY (Figure 3.1) during the report period. All assumptions used in making these assessments (i.e., specific activity, exposure time, and location) shall be included in these reports.

R30

5.0 ADMINISTRATIVE CONTROLS

5.2 ANNUAL RADIOACTIVE EFFLUENT RELEASE REPORT (continued)

R30

The meteorological conditions concurrent with the time of release of radioactive materials in gaseous effluents (as determined by sampling frequency and measurement) shall be used for determining the gaseous pathway doses. The assessment of radiation doses shall be performed in accordance with Sections 6.6 and 7.6.

The annual radioactive effluent release report to be submitted after January 1 of each year shall also include an assessment of radiation doses to the likely most exposed MEMBERS OF THE PUBLIC from reactor releases and other nearby uranium fuel cycle sources (including doses from primary effluent pathways and direct radiation) for the previous calendar year to show conformance with 40 CFR 190, Environmental Radiation Protection Standards for Nuclear Power Operation, in accordance with ODCM Section 8.0. Acceptable methods for calculating the dose contribution from liquid and gaseous effluents are given in Regulatory Guide 1.109, Revision 1.

5.3 OFFSITE DOSE CALCULATION MANUAL CHANGES

As required by SQN Technical Specification 6.14, changes to the ODCM:

1. Shall be documented and records of reviews performed shall be retained as required by SQN Technical Specification 6.10.2.p. This documentation shall contain:
 - a. Sufficient information to support the change together with the appropriate analyses or evaluations justifying the change(s) and
 - b. A determination that the change will maintain the level of radioactive effluent control required by 10 CFR 20.106, 40 CFR 190, 10 CFR 50.36a, and Appendix I to 10 CFR 50 and not adversely impact the accuracy or reliability of effluent, dose, or setpoint calculations.
2. Shall become effective after review and acceptance by the SQN RARC.

5.0 ADMINISTRATIVE CONTROLS

5.3 OFFSITE DOSE CALCULATION MANUAL CHANGES (continued)

3. Shall be submitted to the Commission in the form of a complete, legible copy of the entire ODCM as a part of or concurrent with the Annual Radioactive Effluent Report for the period of the report in which any change to the ODCM was made. Each change shall be identified by markings in the margin of the affected pages, clearly indicating the area of the page that was changed, and shall indicate the date (i.e., month/year) the change was implemented. R30

5.4 SPECIAL REPORTS

Special Reports shall be submitted within the time period specified for each report, in accordance with 10 CFR 50.4.

OFFSITE DOSE CALCULATION/ENVIRONMENTAL MONITORING METHODOLOGIES

SECTION 6.0

LIQUID EFFLUENTS

OFFSITE DOSE CALCULATION/ENVIRONMENTAL MONITORING METHODOLOGIES

RELEASE POINTS

There are four systems from which liquid effluents are released to the environment. These are the Liquid Radwaste System, the Condensate Demineralizer System, the Turbine Building Sump, and the Units 1 and 2 Steam Generator Blowdown. Figure 6.1 provides an outline of the liquid release paths and discharge points with associated flow rates and radiation monitors.

All liquid effluents are ultimately discharged to the Diffuser Pond which releases to the Tennessee River. The Essential Raw Cooling Water (ERCW) routinely provides dilution for liquid effluents at a minimum flow rate of 15,000 gpm. ERCW flow is monitored by radiation monitors 0-RM-133, -134, -140, -141.

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Liquid Radwaste System

The Liquid Radwaste System processes liquid from the Reactor Building and Auxiliary Building Floor Drains and the laundry/hot shower and chemical drain tanks. Figure 6.2 provides a schematic of the Liquid Radwaste System, showing the liquid pathways, flow rate and radiation monitors. The normal release points for liquid radwaste are the Monitor Tank and the Cask Decontamination Collector Tank (CDCT). The Monitor Tank has a capacity of 22,000 gal and is released routinely at a flow rate of 125 gpm. The CDCT has a capacity of 15,000 gal and is also released routinely at a flow rate of 125 gpm. The Monitor Tank and CDCT discharge to the Cooling Tower Blowdown (CTBD) line as a batch release and are monitored by radiation monitor 0-RM-90-122.

Condensate Demineralizer System

The Condensate Demineralizer System processes liquid wastes coming from the High Crud Tanks (HCT-1 and -2), the Neutralization Tank, and the Non-Reclaimable Waste Tank (NRWT). The HCTs have a capacity of 20,000 gal and a maximum discharge flow rate of 245 gpm. The Neutralization Tank has a capacity of 19,000 gal and a maximum discharge flow rate of 245 gpm. The NRWT has a capacity of 11,000 gal and a maximum discharge flow rate of 245 gpm. The Condensate Demineralizer System is routinely released to the CTBD line and is monitored by radiation monitor 0-RM-90-225.

Turbine Building Sump

The Turbine Building Sump (TBS) normally releases to the Low Volume Waste Treatment Pond (LVWTP) but can be released to the Yard Pond. The TBS has a capacity of 30,000 gal and a design discharge release rate of 1,750 gpm per pump. TBS releases are monitored by radiation monitor 0-RM-90-212.

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Steam Generator Blowdown

The Steam Generator Blowdown (SGBD) is processed in the Steam Generator Draindown Flash Tanks or SGBD Heat Exchangers. The SGBD discharge has a maximum flow rate of 80 gpm per steam generator. SGBD discharges to the CTBD line are continuous and are monitored by radiation monitors (1) (2)-RM-90-120, -121.

OFFSITE DOSE CALCULATION/ENVIRONMENTAL MONITORING METHODOLOGIES

6.1 LIQUID RELEASES

6.1.1 Pre-Release Analysis

Radwaste tanks will be recirculated through two volume changes prior to sampling to ensure that a representative sample is obtained. The condensate demineralizer waste evaporator blowdown tanks cannot be recirculated. However, the contents will be transferred to the waste distillate tanks prior to release.

Condensate demineralizer tanks are routinely continuously released^a and utilize a composite sampler to obtain a representative sample while being discharged. In the event of an inoperable effluent radiation monitor or composite sampler, a two volume recirculation and two independent samples and analyses will be performed. Releases from the steam generator blowdown and turbine building sump^a are considered continuous and grab sampled daily.

Prior to a batch release, a grab sample will be taken and analyzed to determine the concentration, $\mu\text{Ci/ml}$, of each gamma-emitting nuclide. For continuous releases, daily grab or composite samples will be taken and analyzed to determine the concentration, $\mu\text{Ci/ml}$, of each gamma-emitting nuclide. Composite samples are maintained (as required by Table 2.2-1) to determine the concentration of certain nuclides (H-3, Fe-55, Sr-89, Sr-90, and alpha emitters).

R27

For those nuclides whose activities are determined from composite samples (i.e. Sr-89, Sr-90, Fe-55 and H-3) the concentrations for the previous composite period will be assumed as the concentration for the next period to perform the calculations in Sections 6.3 and 6.5. The actual measured concentrations will be used for the dose calculations described in Section 6.6.

^a Sampling requirements for these release points are applicable only during periods of primary to secondary leakage or the release of radioactivity as detected by the effluent radiation monitor provided the radiation monitor setpoint is at a LLD of $\leq 1\text{E-}06$ $\mu\text{Ci/ml}$ and allowing for background radiation during periods when primary to secondary leakage is occurring.

OFFSITE DOSE CALCULATION/ENVIRONMENTAL MONITORING METHODOLOGIES

6.1.2 MPC-Sum of the Ratios

The sum of the ratios (R_j) for each release point will be calculated by the following relationship.

$$R_j = \sum_i \frac{C_i}{MPC_i} \quad (6.1)$$

where:

- R_j = the sum of the ratios for release point j.
- MPC_i = the MPC of radionuclide i, $\mu\text{Ci/mL}$.
- C_i = concentration of radionuclide i, $\mu\text{Ci/mL}$.

The sum of the MPC ratios must be ≤ 1 due to the releases from any or all of the release points described above.

The following relationship is used to ensure that this criterion is met:

$$R_{TBS} + \frac{f_1 R_1 + f_2 R_2 + f_3 R_3 + f_4 R_4}{F} \leq 1.0 \quad (6.2)$$

where

- R_{TBS} = sum of the ratios of the turbine building sump as determined by equation 6.1.
- f_1, f_2, f_3, f_4 = effluent flow rate for radwaste, condensate demineralizer system and each of the steam generators, respectively, gpm.
- R_1, R_2, R_3, R_4 = sum of ratios for radwaste, condensate demineralizer system and each of the steam generators, respectively, as determined by equation 6.1.
- F = dilution flow rate for CTBD, routinely 15,000 gpm. R30

6.1.3 Post-Release Analysis

A post-release analysis will be done using actual release data to ensure that the limits specified in ODCM Control 1.2.1.1 were not exceeded.

A composite list of concentrations (C_i), by isotope, will be used with the actual waste (f) and dilution (F) flow rates (or volumes) during the release. The data will be substituted into Equation 6.2 to demonstrate compliance with the limits in ODCM Control 1.2.1.1. This data and setpoints will be recorded in auditable records by plant personnel.

OFFSITE DOSE CALCULATION/ENVIRONMENTAL MONITORING METHODOLOGIES

6.2 INSTRUMENT SETPOINTS

Liquid effluent monitor setpoints are determined to ensure that the concentration of radioactive material released at any time from the site to UNRESTRICTED AREAS does not exceed the MPC limits referenced in ODCM Control 1.2.1.1 and to identify any unexpected releases.

6.2.1 Discharge Point Monitor Setpoints (0-RE-90-211,
0-RE-90-133,134,140,141)

R27

The setpoints for the discharge point monitor, RE-90-211, and for ERCW monitors (RE-90-133,134,140,141) are set to ensure that the concentration of radioactive materials released at any time from the site do not exceed the limits given in ODCM Control 1.2.1.1. The setpoints for these monitors will be set to alarm if the activity in the stream exceeds a value of $1E-06$ μ Ci/ml.

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6.2.2 Release Point Monitor Setpoints (0-RM-90-122; 0-RM-90-225;
0-RM-90-212; 1,2-RM-90-120,121)

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There are five release point effluent monitors: the Liquid Radwaste System radiation monitor 0-RM-90-122; the Condensate Demineralizer System radiation monitor 0-RM-90-225; the Turbine Building Sump radiation monitor 0-RM-90-212; and the Steam Generator Blowdown (SGBD) radiation monitors 1,2-RM-90-120,121.

R27

The batch release points, the Liquid Radwaste System and the Condensate Demineralizer System (if being released in a batch mode), are looking at an undiluted waste stream as it comes out of a tank. The purpose of the monitor setpoints for these batch releases is to identify any release that is larger than expected and would have the potential to exceed the 10 CFR 20 limits after dilution. Setpoints are calculated as described in Section 6.2.3.

The continuous release points, the Condensate Demineralizer System, the Steam Generator Blowdowns and Turbine Building Sump, will not be releasing radioactivity unless there is a primary to secondary leak. When there is no identified primary to secondary leakage, these release points are monitored to indicate the presence of elevated activity levels in these systems. In accordance with the requirements of ODCM Table 2-2.1 footnote h, the setpoints for these monitors will be set to alarm if the activity in the stream exceeds a value of 1×10^{-6} μ Ci/ml.

When there is identified primary to secondary leakage, the continuous release points are monitored to identify any release that is larger than expected and would have the potential to exceed the 10 CFR 20 limits after dilution. The monitor setpoints are calculated in the

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same manner as the batch release point monitor setpoints, described in Section 6.2.3, when this is the case. When these release points are being treated in this manner, a single batch release is defined as all effluent released through this point on a continuous bases for a period of time (usually one week).

6.2.3 Batch Release Point Monitor Setpoint

For each release from a release point, two setpoints are calculated: one based on the monitor response to the contents of the effluent stream; and another based on the predicted response of the monitor to the activity in the release stream if it were large enough to exceed the 10 CFR 20 limits after dilution. The expected monitor response, R in cpm, is calculated using equation 6.3 below. The maximum calculated setpoint, S_{max}, is calculated using Equation 6.4 below. A comparison is made between these two calculated setpoints to determine which is used. The actual monitor setpoint for the release is set equal to X times the expected monitor response, or to the maximum calculated setpoint, whichever is less. X is an administrative factor designed to account for expected variations in monitor response (it will be defined in approved plant instructions). The X times expected response setpoint allows for the identification of any release of radioactivity above the expected amount. The maximum calculated setpoint ensures that the release will be stopped if it exceeds the 10 CFR 20 concentration limits after dilution.

Expected response

$$R = B + \sum_i \text{Eff}_i * C_i \quad (6.3)$$

where

- B = monitor background, cpm.
- Eff_i = monitor efficiency for nuclide i, cpm per μCi/ml. R27
- C_i = tank concentration of nuclide i, μCi/ml. R27

Calculated Maximum Monitor Setpoint

$$S_{\text{max}} = \frac{SF (F_w + (A * F_{\text{dil}}))}{F_w R_j} (R - B) + B \quad (6.4)$$

where

- SF = safety factor for the monitor.
- F_w = flow of waste stream, gpm.
- F_{dil} = flow of the dilution stream, gpm.
- A = fraction of dilution flow allocated to this release point.
 For the TBS, this fraction is zero. The fractions for the remaining 4 release points are defined as the ratio of the

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allocated CTBD flow for that release point to the total CTBD		R30
flow. The CTBD flow allocation fractions for these release		R30
points are routinely:		R30
Radwaste	0.60	R30
Condensate demineralizer	0.20	R30
Steam Generator Blowdown (U1)	0.10	R30
Steam Generator Blowdown (U2)	0.10	R30

NOTE: These allocation factors may be adjusted for a particular release if it is known that there are no releases being made through other release points into the CTBD. For example, if there are no releases being made through the Condensate Demineralizer or either Steam Generator Blowdown, the allocation factor for the Radwaste System may be set equal to one.

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R30

- R_j = sum of the MPC ratios for release point j as calculated in Section 6.1.2.
- R = expected monitor response, cpm, as calculated above.
- B = background, cpm.

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6.3 CUMULATIVE LIQUID EFFLUENT DOSE CALCULATIONS

Doses due to liquid effluents are calculated for each release for all age groups (adult, teen, child and infant) and organs (bone, liver, total body, thyroid, skin, kidney, lung and GI tract). Pathways considered are ingestion of drinking water, fish consumption and recreation-shoreline. The maximum individual dose from drinking water is assumed to be that calculated at the location immediately downstream from the diffuser. The maximum individual dose from fish ingestion is assumed to be that calculated for the consumption of fish caught anywhere between the plant and the first downstream dam (Chickamauga Dam). The maximum potential recreation dose is calculated for a location immediately downstream of the plant outfall. Dose factors for these age groups and pathways are calculated as described in Section 6.7. For pathways with no age or organ specific dose factors (i.e. shoreline recreation), the total body dose will be added to the internal organ doses for all age groups.

The general equation for the dose calculations is:

$$\text{Dose} = \sum_i A_{it} T C_i D \quad (6.5)$$

where:

- A_{it} = the total dose factor to the total body or any organ t for nuclide i , mrem/hr per $\mu\text{Ci/ml}$. The total dose factor is the sum of the dose factors for water ingestion, fish ingestion, and shoreline recreation, as defined in Section 6.7.
- T = the length of time period over which the concentrations and the flows are averaged for the liquid release, hours.
- C_i = the average concentration of radionuclide i , in undiluted liquid effluent during the time period T from any liquid release, $\mu\text{Ci/ml}$.
- D = the near field average dilution factor for C_i during any effluent release. D is calculated by the following equation:

$$D = \frac{\text{FLOW}_w}{0.60 \text{ RF}}$$

where:

- FLOW_w = maximum undiluted liquid waste flow during the release, cfs. For TBS releases, this term is the diluted waste flow into the pond.
- 0.60 = mixing factor of effluent in river, defined as the percentage of the riverflow which is available for dilution of the release.
- RF = default riverflow, cfs. For each release, this value is set to 7900 cfs (the lowest average quarterly riverflow recorded from the period 1978-1988).

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From the four age groups considered, the maximum is determined by comparing all organ doses for all age groups. The age group with the highest single organ dose is selected as the critical age group. The total body and maximum organ doses for the critical age group are used in the calculation of the monthly dose described in Section 6.3.1.

6.3.1 Monthly Dose Calculations

At the end of each month, the actual average riverflow for the month is used to recalculate the liquid doses. The monthly cumulative dose is defined as the sum of the doses for the critical age group for each release during the month. Thus, the monthly cumulative dose will be a conservative value, consisting of doses belonging to various age groups depending on the mix of radionuclides. These doses are multiplied by the ratio of the default riverflow (7900 cfs) to the actual monthly average riverflow to obtain the monthly dose. The total body and maximum organ doses determined in this manner are then used to determine the cumulative quarterly and annual doses described in Section 6.3.2, and for the dose projections described in Section 6.5.

6.3.2 Cumulative Doses

Quarterly and annual sums of all doses are determined at the end of each month to compare to the limits given in ODCM Control 1.2.1.2. These quarterly and annual sums will be the sum of the monthly cumulative doses described in Section 6.3.1 for the appropriate months in the quarter or year. These doses will be used in the comparison to the limits.

6.3.3 Comparison to Limits

The cumulative calendar quarter and calendar year doses are compared to the limits in ODCM Control 1.2.1.2 once per 31 days to determine compliance.

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6.4 LIQUID RADWASTE TREATMENT SYSTEM

The liquid radwaste treatment system described below shall be maintained and operated to keep releases ALARA.

A flow diagram for the LRTS is given in Figure 6.2. The system consists of one reactor coolant drain tank with two pumps and a floor and equipment drain sump inside the containment of each unit and the following shared equipment inside the auxiliary building: one sump tank and pumps, one tritiated drain collector tank with two pumps and one filter, one floor drain collector tank with two pumps and one filter, a waste condensate tank filter, three waste condensate tanks and two pumps, a chemical drain tank and pump, two laundry and hot shower tanks and pump, a spent resin storage tank, a cask decontamination tank with two pumps and two filters. Auxiliary Building floor end equipment drain sump and pumps, and evaporator with two distillate tanks, a Mobile Waste Demineralizer System (if needed) and the associated piping, valves and instrumentation.

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6.5 DOSE PROJECTIONS

In accordance with ODCM Surveillance Requirement 2.2.1.3, dose projections will be performed by averaging the two previous month's doses as determined in Section 6.3.1. To determine compliance with the limits, these averages are assigned as the dose projections for the upcoming month.

The projected doses are compared to the limits of ODCM Control 1.2.1.3. If the projected doses exceed either of these limits, the liquid radwaste treatment system shall be used to reduce the radioactive materials in liquid wastes prior to their discharge to UNRESTRICTED AREAS.

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6.6 QUARTERLY DOSE CALCULATIONS

A complete dose analysis utilizing the total estimated liquid releases for each calendar quarter will be performed and reported as required in ODCM Administrative Control 5.2. Methodology for this analysis is that which is described in this section using the quarterly release values reported by the plant personnel. The releases are assumed, for this calculation, to be continuous over the 90 day period.

The average dilution factor, D, used for the quarterly calculations is:

$$D = \frac{1}{RF * 0.60} \quad \text{(for receptors upstream of Chickamauga Dam)} \quad (6.6)$$

and

$$D = \frac{1}{RF} \quad \text{(for receptors downstream of Chickamauga Dam)} \quad (6.7)$$

where:

- RF = the average actual riverflow for the location at which the dose is being determined, cfs.
- 0.60 = the fraction of the riverflow available for dilution in the near field, dimensionless.

6.6.1 WATER INGESTION

Water ingestion doses are calculated for each water supply identified within a 50 mile radius downstream of SQN (Table 6.1). Water ingestion doses are calculated for the total body and each internal organ as described below:

$$D_{org} = 10^6 \cdot 9.80E-09 \cdot A_{wit} \cdot Q_i \cdot D \cdot \exp(-8.64E+04 \lambda_i t_d) \quad (6.8)$$

where

- 10^6 = conversion factor, $\mu\text{Ci}/\text{Ci}$.
- $9.80E-09$ = conversion factor, cfs per ml/hour.
- A_{wit} = Dose factor for water ingestion for nuclide i, age group t, mrem/hour per $\mu\text{Ci}/\text{ml}$, as calculated in Section 6.7.1.
- Q_i = Quantity of nuclide i released during the quarter, Curies.
- D = dilution factor, as described above, cfs^{-1} .
- λ_i = radiological decay constant of nuclide i, seconds^{-1} (Table 6.2).

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- t_d = decay time for water ingestion, equal to the travel time from the plant to the water supply plus one day to account for the time of processing at the water supply (per Regulatory Guide 1.109), days.
 $8.64E+04$ = conversion factor, seconds per day.

6.6.2 FISH INGESTION

Fish ingestion doses are calculated for each identified reach within a 50 mile radius downstream of SQN (Table 6.1). Individual fish ingestion doses are calculated for the total body and each internal organ as described below:

$$D_{org} = 10^6 \cdot 9.80E-09 \cdot 0.25 \cdot A_{fit} \cdot Q_i \cdot D \cdot \exp(-8.64E+04 \lambda_i t_d) \quad (6.9)$$

where

- 10^6 = conversion factor, $\mu\text{Ci}/\text{Ci}$.
 $9.80E-09$ = conversion factor, cfs per ml/hour.
 0.25 = fraction of the yearly fish consumption eaten in one quarter, dimensionless.
 A_{fit} = Dose factor for fish ingestion for nuclide i , age group t , mrem/hour per $\mu\text{Ci}/\text{ml}$, as calculated in Section 6.7.2.
 Q_i = Quantity of nuclide i released during the quarter, Curies.
 D = dilution factor, as described above, cfs^{-1} .
 λ_i = radiological decay constant of nuclide i , seconds^{-1} (Table 6.2).
 t_d = decay time for fish ingestion, equal to the travel time from the plant to the center of the reach plus one day to account for transit through the food chain and food preparation time (per Regulatory Guide 1.109), days.
 $8.64E+04$ = conversion factor, seconds per day.

6.6.3 SHORELINE RECREATION

Recreation doses are calculated for each identified reach within a 50 mile radius downstream of SQN (Table 6.1). It is assumed that the maximum exposed individual spends 500 hours per year on the shoreline at a location immediately downstream from the diffusers. Individual recreation shoreline doses are calculated for the total body and skin as described below:

$$D_{org} = 10^6 \cdot 9.80E-09 \cdot r_f \cdot A_{rit} \cdot Q_i \cdot D \cdot \exp(-8.64E+04 \lambda_i t_d) \quad (6.10)$$

where

- 10^6 = conversion factor, $\mu\text{Ci}/\text{Ci}$.
 $9.80E-09$ = conversion factor, cfs per ml/hour.

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- rf = recreation factor, used to account for the fact that the same amount of time will not be spent at a recreation site during each quarter. Recreation factors used are:
 1st quarter - 0.1
 2nd quarter - 0.3
 3rd quarter - 0.4
 4th quarter - 0.2.
- A_{rit} = Dose factor for shoreline recreation for nuclide i, age group t, mrem/hour per $\mu\text{Ci/ml}$, as calculated in Section 6.7.3.
- Q_i = quantity of nuclide i released during the quarter, Curies.
- D = dilution factor, as described above, cfs^{-1} .
- λ_i = radiological decay constant of nuclide i, seconds^{-1} (Table 6.2).
- t_d = decay time for recreation, equal to the travel time from the plant to the center of the reach, days.
- $8.64\text{E}+04$ = conversion factor, seconds per day.

6.6.4 TOTAL MAXIMUM INDIVIDUAL DOSE

The total maximum individual total body dose is obtained by summing the following for each age group: the highest total body water ingestion dose from among all the public water supplies; the highest total body fish ingestion dose from among all the reaches; and the total body maximum shoreline recreation dose. The total maximum individual organ dose is obtained by summing the following for each organ and each age group: that organ's highest water ingestion dose from among all the public water supplies; that organ's highest fish ingestion dose from among all the reaches; and the total body maximum shoreline recreation dose. The total maximum individual skin dose is that skin dose calculated for the maximum shoreline dose.

6.6.5 POPULATION DOSES

For determining population doses to the 50-mile population around the plant, an average dose is calculated for each age group and each pathway and then multiplied by the population.

For water ingestion, the general equation used for calculating the population doses, POPWTR, in man-rem for a given PWS is:

$$\text{POPWTR}_t = 10^{-3} \sum_{m=1}^5 \text{POP}_m \sum_{a=1}^4 \text{POP}_a \text{ATNW}_a \text{TWDOS}_{amt} \quad (6.11)$$

where:

POPWTR_t = water ingestion population dose to organ t, man-rem.

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- POP_a = fraction of population in each age group a (from NUREG CR-1004, table 3.39).
 Adult = 0.665
 Child = 0.168
 Infant = 0.015
 Teen = 0.153
- POP_m = population at PWS m. The 4 PWSs and their populations are listed in Table 6.1.
- ATMW_a = ratio of average to maximum water ingestion rates for each age group a. Maximum water ingestion rates are given in Table 6.3. Average water ingestion rates, in L/year, (from R.G. 1.109 Table E-4) are:
 Adult = 370
 Child = 260
 Infant = 260
 Teen = 260
- TWDOS_{amt} = total individual water ingestion dose to organ t at PWS m, to the age group a, as described in Section 6.6.1, mrem.
- 10⁻³ = conversion factor for rem/mrem.

For population doses resulting from fish ingestion the calculation assumes that all fish caught within a 50-mile radius downstream of SQN are consumed by local population. An additional 7-day decay term is added due to distribution time of sport fish. The general equation for calculating population doses, POPF, in man-rem from fish ingestion of all fish caught within a 50-mile radius downstream is:

$$POPF_t = \frac{453.6 \text{ HVST APR}}{10^3 10^3} \sum_{r=1}^4 \sum_{a=1}^3 \frac{\text{TFDOS}_{art} \text{ POP}_a}{\text{FISH}_a \text{ POP}_a} \quad (6.12)$$

where:

- POPF_t = total fish ingestion population dose to organ t, man-rem.
 HVST = fish harvest for the Tennessee River, 3.04 lbs/acre/year.
 APR = size of reach, acres (Table 6.1).
 TFDOS_{art} = total fish ingestion dose to organ t for reach r, for the age group a, as described in Section 6.6.2, mrem.
 POP_a = fraction of population in each age group a, as given above.
 FISH_a = amount of fish ingested by each age group a, kg/year.
 Average fish ingestion rates (R.G. 1.109 Table E-4) are:
 Adult = 6.9
 Child = 2.2
 Teen = 5.2
- 453.6 = conversion factor, g/lb.
 10³ = conversion factor, mrem/rem.
 10³ = conversion factor, g/kg.

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For recreation shoreline, the general equation used for calculating the population doses, POPR, in man-rem is:

$$\text{POPR}_t = \frac{\text{REQFRA}}{10^3 \cdot 8760} \sum_{r=1}^4 \text{TSHDOS}_{rt} \text{SHVIS}_r \text{HRSVIS}_r \quad (6.13)$$

where:

- POPR_t = total recreation population dose for all reaches to organ t, man-rem.
- REQFRA = fraction of yearly recreation which occurs in that quarter, as given in Section 6.6.3.
- TSHDOS_{rt} = total shoreline dose rate for organ t, in reach r, mrem/h.
- SHVIS_r = shoreline visits per year at each reach r, (Table 6.1).
- HRSVIS_r = length of shoreline recreation visit at reach r, 5 hours.
- 10³ = conversion factor, mrem/rem.
- 8760 = conversion factor, hours/year.

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6.7 LIQUID DOSE FACTOR EQUATIONS

6.7.1 WATER INGESTION - A_{wit} (mrem/hr per $\mu\text{Ci/ml}$)

$$A_{wit} = \frac{DF_{Liat} U_{wa} 10^6 10^3}{8760}$$

where:

- DF_{Liat} = ingestion dose conversion factor for nuclide i, age group a, organ t, mrem/pCi, (Table 6.4).
- U_{wa} = water consumption rate for age group a, L/year, (Table 6.3).
- 10^6 = conversion factor, pCi/ μCi .
- 10^3 = conversion factor, ml/L.
- 8760 = conversion factor, hours per year.

6.7.2 FISH INGESTION - A_{fit} (mrem/hr per $\mu\text{Ci/ml}$)

$$A_{fit} = \frac{DF_{Liat} U_{fa} B_i 10^6 10^3}{8760}$$

where:

- DF_{Liat} = ingestion dose conversion factor for nuclide i, age group a, organ t, mrem/pCi, (Table 6.4).
- U_{fa} = fish consumption rate for age group a, kg/year, (Table 6.3).
- B_i = bioaccumulation factor for nuclide i, pCi/kg per pCi/L, (Table 6.5).
- 10^6 = conversion factor, pCi/ μCi .
- 10^3 = conversion factor, ml/L.
- 8760 = conversion factor, hours per year.

6.7.3 SHORELINE RECREATION - A_{rit} (mrem/hr per $\mu\text{Ci/ml}$).

$$A_{rit} = \frac{DF_{Git} K_c M W 10^3 10^6 U}{8760 * 3600 \lambda_i} [1 - \exp(-\lambda_i t_b)]$$

where:

- DF_{Git} = dose conversion factor for standing on contaminated ground for nuclide i and organ t (total body and skin), mrem/hr per pCi/m², (Table 6.6).
- K_c = transfer coefficient from water to shoreline sediment, L/kg-hr, (Table 6.3).
- M = mass density of sediment, kg/m², (Table 6.3).

OFFSITE DOSE CALCULATION/ENVIRONMENTAL MONITORING METHODOLOGIES

W = shoreline width factor, dimensionless, (Table 6.3).
10³ = conversion factor, ml/L.
10⁶ = conversion factor, pCi/μCi.
3600 = conversion factor, seconds/hour.
λ_i = decay constant for nuclide i, seconds⁻¹, (Table 6.2).
t_b = time shoreline is exposed to the concentration on the water,
seconds, (Table 6.3).
U = usage factor, 500 hours/year.
8760 = conversion factor, hours/year.

OFFSITE DOSE CALCULATION/ENVIRONMENTAL MONITORING METHODOLOGIES

Table 6.1
 RECEPTORS FOR LIQUID DOSE CALCULATIONS

Tennessee River Reaches Within 50 Mile Radius Downstream of SQN

<u>Name</u>	<u>Beginning TRM</u>	<u>Ending TRM</u>	<u>Size (acres)</u>	<u>Recreation visits/year</u>
Chickamauga Lake below SQN	484.0	471.0	9939	5,226,700
Nickajack Lake (Part 1)	471.0	435.0	5604	240,700
Nickajack Lake (Part 2)	435.0	425.0	5326	607,600
Guntersville Lake	425.0	400.0	6766	104,000

Public Water Supplies Within 50 Mile Radius Downstream of SQN

<u>Name</u>	<u>TRM</u>	<u>Population</u>
E. I. DuPont	469.9	1,400
Chattanooga, TN	465.3	224,000
South Pittsburg, TN	418.0	4,898
Bridgeport, AL	413.6	4,650

OFFSITE DOSE CALCULATION/ENVIRONMENTAL MONITORING METHODOLOGIES

Table 6.2 (1 of 3)
 RADIONUCLIDE DECAY AND STABLE ELEMENT TRANSFER DATA

	Half-Life (minutes)	λ (1/s)	B_{iv}	F_{mi} (cow)	F_{mi} (goat)	F_{fi} (beef)
H-3	6.46E+06	1.79E-09	4.80E+00	1.00E-02	1.70E-01	1.20E-02
C-14	3.01E+09	3.84E-12	5.50E+00	1.20E-02	1.00E-01	3.10E-02
Na-24	9.00E+02	1.28E-05	5.20E-02	4.00E-02	4.00E-02	3.00E-02
P-32	2.06E+04	5.61E-07	1.10E+00	2.50E-02	2.50E-01	4.60E-02
Cr-51	3.99E+04	2.90E-07	2.50E-04	2.20E-03	2.20E-03	2.40E-03
Mn-54	4.50E+05	2.57E-08	2.90E-02	2.50E-04	2.50E-04	8.00E-04
Mn-56	1.55E+02	7.45E-05	2.90E-02	2.50E-04	2.50E-04	8.00E-04
Fe-55	1.42E+06	8.13E-09	6.60E-04	1.20E-03	1.30E-04	1.20E-02
Fe-59	6.43E+04	1.80E-07	6.60E-04	1.20E-03	1.30E-04	1.20E-02
Co-57	3.90E+05	2.96E-08	9.40E-03	1.00E-03	1.00E-03	1.30E-02
Co-58	1.02E+05	1.13E-07	9.40E-03	1.00E-03	1.00E-03	1.30E-02
Co-60	2.77E+06	4.17E-09	9.40E-03	1.00E-03	1.00E-03	1.30E-02
Ni-63	5.27E+07	2.19E-10	1.90E-02	6.70E-03	6.70E-03	5.30E-02
Ni-65	1.51E+02	7.65E-05	1.90E-02	6.70E-03	6.70E-03	5.30E-02
Cu-64	7.62E+02	1.52E-05	1.20E-01	1.40E-02	1.30E-02	9.70E-04
Zn-65	3.52E+05	3.28E-08	4.00E-01	3.90E-02	3.90E-02	3.00E-02
Zn-69m	8.26E+02	1.40E-05	4.00E-01	3.90E-02	3.90E-02	3.00E-02
Zn-69	5.56E+01	2.08E-04	4.00E-01	3.90E-02	3.90E-02	3.00E-02
Br-82	2.12E+03	5.45E-06	7.60E-01	5.00E-02	5.00E-02	2.60E-02
Br-83	1.43E+02	8.08E-05	7.60E-01	5.00E-02	5.00E-02	2.60E-02
Br-84	3.18E+01	3.63E-04	7.60E-01	5.00E-02	5.00E-02	2.60E-02
Br-85	2.87E+00	4.02E-03	7.60E-01	5.00E-02	5.00E-02	2.60E-02
Rb-86	2.69E+04	4.29E-07	1.30E-01	3.00E-02	3.00E-02	3.10E-02
Rb-88	1.78E+01	6.49E-04	1.30E-01	3.00E-02	3.00E-02	3.10E-02
Rb-89	1.54E+01	7.50E-04	1.30E-01	3.00E-02	3.00E-02	3.10E-02
Sr-89	7.28E+04	1.59E-07	1.70E-02	1.40E-03	1.40E-02	6.00E-04
Sr-90	1.50E+07	7.70E-10	1.70E-02	1.40E-03	1.40E-02	6.00E-04
Sr-91	5.70E+02	2.03E-05	1.70E-02	1.40E-03	1.40E-02	6.00E-04
Sr-92	1.63E+02	7.09E-05	1.70E-02	1.40E-03	1.40E-02	6.00E-04
Y-90	3.85E+03	3.00E-06	2.60E-03	1.00E-05	1.00E-05	4.60E-03
Y-91m	4.97E+01	2.32E-04	2.60E-03	1.00E-05	1.00E-05	4.60E-03
Y-91	8.43E+04	1.37E-07	2.60E-03	1.00E-05	1.00E-05	4.60E-03
Y-92	2.12E+02	5.45E-05	2.60E-03	1.00E-05	1.00E-05	4.60E-03
Y-93	6.06E+02	1.91E-05	2.60E-03	1.00E-05	1.00E-05	4.60E-03
Zr-95	9.22E+04	1.25E-07	1.70E-04	5.00E-06	5.00E-06	3.40E-02
Zr-97	1.01E+03	1.14E-05	1.70E-04	5.00E-06	5.00E-06	3.40E-02
Nb-95	5.05E+04	2.29E-07	9.40E-03	2.50E-03	2.50E-03	2.80E-01
Nb-97	7.21E+01	1.60E-04	9.40E-03	2.50E-03	2.50E-03	2.80E-01
Mo-99	3.96E+03	2.92E-06	1.20E-01	7.50E-03	7.50E-03	1.10E-03
Tc-99m	3.61E+02	3.20E-05	2.50E-01	2.50E-02	2.50E-02	4.00E-01
Tc-101	1.42E+01	8.13E-04	2.50E-01	2.50E-02	2.50E-02	4.00E-01
Ru-103	5.67E+04	2.04E-07	5.00E-02	1.00E-06	1.00E-06	4.00E-01
Ru-105	2.66E+02	4.34E-05	5.00E-02	1.00E-06	1.00E-06	4.00E-01
Ru-106	5.30E+05	2.18E-08	5.00E-02	1.00E-06	1.00E-06	4.00E-01

OFFSITE DOSE CALCULATION/ENVIRONMENTAL MONITORING METHODOLOGIES

Table 6.2 (2 of 3)
 RADIONUCLIDE DECAY AND STABLE ELEMENT TRANSFER DATA

	Half-Life (minutes)	λ (1/s)	B_{iv}	F_{mi} (cow)	F_{mi} (goat)	F_{fi} (beef)
Ag-110m	3.60E+05	3.21E-08	1.50E-01	5.00E-02	5.00E-02	1.70E-02
Sb-124	8.67E+04	1.33E-07	N/A	1.50E-03	1.50E-03	N/A
Sb-125	1.46E+06	7.91E-09	N/A	1.50E-03	1.50E-03	N/A
Sn-125	1.39E+04	8.32E-07	N/A	N/A	N/A	N/A
Te-125m	8.35E+04	1.38E-07	1.30E+00	1.00E-03	1.00E-03	7.70E-02
Te-127m	1.57E+05	7.36E-08	1.30E+00	1.00E-03	1.00E-03	7.70E-02
Te-127	5.61E+02	2.06E-05	1.30E+00	1.00E-03	1.00E-03	7.70E-02
Te-129m	4.84E+04	2.39E-07	1.30E+00	1.00E-03	1.00E-03	7.70E-02
Te-129	6.96E+01	1.66E-04	1.30E+00	1.00E-03	1.00E-03	7.70E-02
Te-131m	1.80E+03	6.42E-06	1.30E+00	1.00E-03	1.00E-03	7.70E-02
Te-131	2.50E+01	4.62E-04	1.30E+00	1.00E-03	1.00E-03	7.70E-02
Te-132	4.69E+03	2.46E-06	1.30E+00	1.00E-03	1.00E-03	7.70E-02
I-130	7.42E+02	1.56E-05	2.00E-02	1.20E-02	4.30E-01	2.90E-03
I-131	1.16E+04	9.96E-07	2.00E-02	1.20E-02	4.30E-01	2.90E-03
I-132	1.38E+02	8.37E-05	2.00E-02	1.20E-02	4.30E-01	2.90E-03
I-133	1.25E+03	9.24E-06	2.00E-02	1.20E-02	4.30E-01	2.90E-03
I-134	5.26E+01	2.20E-04	2.00E-02	1.20E-02	4.30E-01	2.90E-03
I-135	3.97E+02	2.91E-05	2.00E-02	1.20E-02	4.30E-01	2.90E-03
Cs-134	1.08E+06	1.06E-08	1.00E-02	8.00E-03	3.00E-01	1.50E-02
Cs-136	1.90E+04	6.08E-07	1.00E-02	8.00E-03	3.00E-01	1.50E-02
Cs-137	1.59E+07	7.26E-10	1.00E-02	8.00E-03	3.00E-01	1.50E-02
Cs-138	3.22E+01	3.59E-04	1.00E-02	8.00E-03	3.00E-01	1.50E-02
Ba-139	8.31E+01	1.39E-04	5.00E-03	4.00E-04	4.00E-04	3.20E-03
Ba-140	1.84E+04	6.28E-07	5.00E-03	4.00E-04	4.00E-04	3.20E-03
Ba-141	1.83E+01	6.31E-04	5.00E-03	4.00E-04	4.00E-04	3.20E-03
Ba-142	1.07E+01	1.08E-03	5.00E-03	4.00E-04	4.00E-04	3.20E-03
La-140	2.41E+03	4.79E-06	2.50E-03	5.00E-06	5.00E-06	2.00E-04
La-142	9.54E+01	1.21E-04	2.50E-03	5.00E-06	5.00E-06	2.00E-04
Ce-141	4.68E+04	2.47E-07	2.50E-03	1.00E-04	1.00E-04	1.20E-03
Ce-143	1.98E+03	5.83E-06	2.50E-03	1.00E-04	1.00E-04	1.20E-03
Ce-144	4.09E+05	2.82E-08	2.50E-03	1.00E-04	1.00E-04	1.20E-03
Pr-143	1.95E+04	5.92E-07	2.50E-03	5.00E-06	5.00E-06	4.70E-03
Pr-144	1.73E+01	6.68E-04	2.50E-03	5.00E-06	5.00E-06	4.70E-03
Nd-147	1.58E+04	7.31E-07	2.40E-03	5.00E-06	5.00E-06	3.30E-03
W-187	1.43E+03	8.08E-06	1.80E-02	5.00E-04	5.00E-04	1.30E-03
Np-239	3.39E+03	3.41E-06	2.50E-03	5.00E-06	5.00E-06	2.00E-04
Ar-41	1.10E+02	1.05E-04	N/A	N/A	N/A	N/A
Kr-83m	1.10E+02	1.05E-04	N/A	N/A	N/A	N/A
Kr-85m	2.69E+02	4.29E-05	N/A	N/A	N/A	N/A
Kr-85	5.64E+06	2.05E-09	N/A	N/A	N/A	N/A
Kr-87	7.63E+01	1.51E-04	N/A	N/A	N/A	N/A
Kr-88	1.70E+02	6.79E-05	N/A	N/A	N/A	N/A
Kr-89	3.16E+00	3.66E-03	N/A	N/A	N/A	N/A
Kr-90	5.39E-01	2.14E-02	N/A	N/A	N/A	N/A
Xe-131m	1.70E+04	6.79E-07	N/A	N/A	N/A	N/A

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OFFSITE DOSE CALCULATION/ENVIRONMENTAL MONITORING METHODOLOGIES

Table 6.2 (3 of 3)
 RADIONUCLIDE DECAY AND STABLE ELEMENT TRANSFER DATA

	Half-Life (minutes)	λ (1/s)	B_{iv}	F_{mi} (cow)	F_{mi} (goat)	F_{fi} (beef)
Xe-133m	3.15E+03	3.67E-06	N/A	N/A	N/A	N/A
Xe-133	7.55E+03	1.53E-06	N/A	N/A	N/A	N/A
Xe-135m	1.54E+01	7.50E-04	N/A	N/A	N/A	N/A
Xe-135	5.47E+02	2.11E-05	N/A	N/A	N/A	N/A
Xe-137	3.83E+00	3.02E-03	N/A	N/A	N/A	N/A
Xe-138	1.41E+01	8.19E-04	N/A	N/A	N/A	N/A

References:

Half lives for all nuclides: DOE-TIC-11026, "Radioactive Decay Data Tables - A handbook of Decay Data for Application to Radiation Dosimetry and Radiological Assessment," D. C. Kocher, 1981.

Transfer factors for Sb- isotopes are from ORNL 4992, "Methodology for Calculating Radiation Doses from Radioactivity Released to the Environment," March 1976, Table 2-7.

Cow-milk transfer factors for Iodine, Strontium, and Cesium nuclides are from NUREG/CR-1004, Table 3.17.

Goat-milk transfer factors for Iodine nuclides are from NUREG/CR-1004, Table 3.17.

Beef transfer factors for Iron, Copper, Molybdenum, and Cesium nuclides are from NUREG/CR-1004, Table 3.18.

All other nuclides' transfer factors are from Regulatory Guide 1.109, Tables E-1 and E-2.

OFFSITE DOSE CALCULATION/ENVIRONMENTAL MONITORING METHODOLOGIES

Table 6.3 (1 of 2)
 DOSE CALCULATION FACTORS

<u>Factor</u>	<u>Value</u>	<u>Units</u>	<u>Reference</u>
BR _a (infant)	1400	m ³ /year	ICRP 23
BR _a (child)	5500	m ³ /year	ICRP 23
BR _a (teen)	8000	m ³ /year	ICRP 23
BR _a (adult)	8100	m ³ /year	ICRP 23
f _g	1		TVA Assumption
f _L	1		R. G. 1.109 (Table E-15)
f _p	1		TVA Assumption
f _s	0		TVA Assumption
H	9	g/m ³	TVA Value
K _c	0.072	L/kg-hr	R. G. 1.109 (Section 2.C.)
M	40	kg/m ²	R. G. 1.109 (Section 2.C.)
P	240	kg/m ²	R. G. 1.109 (Table E-15)
Q _f (cow)	64	kg/day	NUREG/CR-1004 (Sect. 3.4)
Q _f (goat)	08	kg/day	NUREG/CR-1004 (Sect. 3.4)
r	0.47		NUREG/CR-1004 (Sect. 3.2)
t _b	4.73E+08	seconds (15 years)	R. G. 1.109 (Table E-15)
t _{cb}	7.78E+06	seconds (90 days)	SQN FSAR Section 11.3.9.1
t _{csf}	1.56E+07	seconds (180 days)	SQN FSAR Section 11.3.9.1
t _e	5.18E+06	seconds (60 days)	R. G. 1.109 (Table E-15)
t _{ep}	2.59E+06	seconds (30 days)	R. G. 1.109 (Table E-15)
t _{esf}	7.78E+06	seconds (90 days)	R. G. 1.109 (Table E-15)
t _{fm}	8.64E+04	seconds (1 day)	SQN FSAR Section 11.3.9.1
t _{hc}	8.64E+04	seconds (1 day)	NUREG/CR-1004, Table 3.40
t _s	1.12E+06	seconds (13 days)	NUREG/CR-1004, Table 3.40
t _{sv}	2.38E+07	seconds (275 days)	SQN FSAR Section 11.3.9.1
U _{am} (infant)	0	kg/year	R. G. 1.109 (Table E-5)
U _{am} (child)	41	kg/year	R. G. 1.109 (Table E-5)
U _{am} (teen)	65	kg/year	R. G. 1.109 (Table E-5)
U _{am} (adult)	110	kg/year	R. G. 1.109 (Table E-5)
U _{ap} (infant)	330	L/year	R. G. 1.109 (Table E-5)
U _{ap} (child)	330	L/year	R. G. 1.109 (Table E-5)
U _{ap} (teen)	400	L/year	R. G. 1.109 (Table E-5)
U _{ap} (adult)	310	L/year	R. G. 1.109 (Table E-5)
U _{fa} (infant)	0	kg/year	R. G. 1.109 (Table E-5)
U _{fa} (child)	6.9	kg/year	R. G. 1.109 (Table E-5)
U _{fa} (teen)	16	kg/year	R. G. 1.109 (Table E-5)

OFFSITE DOSE CALCULATION/ENVIRONMENTAL MONITORING METHODOLOGIES

Table 6.3 (2 of 2)
 DOSE CALCULATION FACTORS

<u>Factor</u>	<u>Value</u>	<u>Units</u>	<u>Reference</u>
U _{fa} (adult)	21	kg/year	R. G. 1.109 (Table E-5)
U _{FLa} (infant)	0	kg/year	R. G. 1.109 (Table E-5)
U _{FLa} (child)	26	kg/year	R. G. 1.109 (Table E-5)
U _{FLa} (teen)	42	kg/year	R. G. 1.109 (Table E-5)
U _{FLa} (adult)	64	kg/year	R. G. 1.109 (Table E-5)
U _{Sa} (infant)	0	kg/year	R. G. 1.109 (Table E-5)
U _{Sa} (child)	520	kg/year	R. G. 1.109 (Table E-5)
U _{Sa} (teen)	630	kg/year	R. G. 1.109 (Table E-5)
U _{Sa} (adult)	520	kg/year	R. G. 1.109 (Table E-5)
U _{wa} (infant)	330	L/year	R. G. 1.109 (Table E-5)
U _{wa} (child)	510	L/year	R. G. 1.109 (Table E-5)
U _{wa} (teen)	510	L/year	R. G. 1.109 (Table E-5)
U _{wa} (adult)	730	L/year	R. G. 1.109 (Table E-5)
W	0.3	none	R. G. 1.109 (Table A-2)
Y _f	1.85	kg/m ²	NUREG/CR-1004 (Table 3.4)
Y _p	1.18	kg/m ²	NUREG/CR-1004 (Table 3.3)
Y _{sf}	0.64	kg/m ²	NUREG/CR-1004 (Table 3.3)
Y _{sv}	0.57	kg/m ²	NUREG/CR-1004 (Table 3.4)
			(value selected is for non-leafy vegetables)
λ _w (iodines)	7.71E-07	sec ⁻¹	NUREG/CR-1004 (Table 3.10)
	(10.4 d half-life)		
λ _w (particulates)	5.21E-07	sec ⁻¹	NUREG/CR-1004 (Table 3.10)
	(15.4 d half-life)		

OFFSITE DOSE CALCULATION/ENVIRONMENTAL MONITORING METHODOLOGIES

Table 6.4 (1 of 8)
 INGESTION DOSE FACTORS
 (mrem/pCi ingested)

	ADULT						
	bone	liver	t body	thyroid	kidney	lung	gi-lli
H-3	1.05E-07	1.05E-07	1.05E-07	1.05E-07	1.05E-07	1.05E-07	1.05E-07
C-14	2.84E-06	5.68E-07	5.68E-07	5.68E-07	5.68E-07	5.68E-07	5.68E-07
Na-24	1.70E-06	1.70E-06	1.70E-06	1.70E-06	1.70E-06	1.70E-06	1.70E-06
P-32	1.93E-04	1.20E-05	7.46E-06	0.00E+00	0.00E+00	0.00E+00	2.17E-05
Cr-51	0.00E+00	0.00E+00	2.66E-09	1.59E-09	5.86E-10	3.53E-09	6.69E-07
Mn-54	0.00E+00	4.57E-06	8.72E-07	0.00E+00	1.36E-06	0.00E+00	1.40E-05
Mn-56	0.00E+00	1.15E-07	2.04E-08	0.00E+00	1.46E-07	0.00E+00	3.67E-06
Fe-55	2.75E-06	1.90E-06	4.43E-07	0.00E+00	0.00E+00	1.06E-06	1.09E-06
Fe-59	4.34E-06	1.02E-05	3.91E-06	0.00E+00	0.00E+00	2.85E-06	3.40E-05
Co-57	0.00E+00	1.75E-07	2.91E-07	0.00E+00	0.00E+00	0.00E+00	4.44E-06
Co-58	0.00E+00	7.45E-07	1.67E-06	0.00E+00	0.00E+00	0.00E+00	1.51E-05
Co-60	0.00E+00	2.14E-06	4.72E-06	0.00E+00	0.00E+00	0.00E+00	4.02E-05
Ni-63	1.30E-04	9.01E-06	4.36E-06	0.00E+00	0.00E+00	0.00E+00	1.88E-06
Ni-65	5.28E-07	6.86E-08	3.13E-08	0.00E+00	0.00E+00	0.00E+00	1.74E-06
Cu-64	0.00E+00	8.33E-08	3.91E-08	0.00E+00	2.10E-07	0.00E+00	7.10E-06
Zn-65	4.84E-06	1.54E-05	6.96E-06	0.00E+00	1.03E-05	0.00E+00	9.70E-06
Zn-69	1.03E-08	1.97E-08	1.37E-09	0.00E+00	1.28E-08	0.00E+00	2.96E-09
Zn-69m	1.70E-07	4.08E-07	3.73E-08	0.00E+00	2.47E-07	0.00E+00	2.49E-05
Br-82	0.00E+00	0.00E+00	2.26E-06	0.00E+00	0.00E+00	0.00E+00	2.59E-06
Br-83	0.00E+00	0.00E+00	4.02E-08	0.00E+00	0.00E+00	0.00E+00	5.79E-08
Br-84	0.00E+00	0.00E+00	5.21E-08	0.00E+00	0.00E+00	0.00E+00	4.09E-13
Br-85	0.00E+00	0.00E+00	2.14E-09	0.00E+00	0.00E+00	0.00E+00	0.00E+00
Rb-86	0.00E+00	2.11E-05	9.83E-06	0.00E+00	0.00E+00	0.00E+00	4.16E-06
Rb-88	0.00E+00	6.05E-08	3.21E-08	0.00E+00	0.00E+00	0.00E+00	8.36E-19
Rb-89	0.00E+00	4.01E-08	2.82E-08	0.00E+00	0.00E+00	0.00E+00	2.33E-21
Sr-89	3.08E-04	0.00E+00	8.84E-06	0.00E+00	0.00E+00	0.00E+00	4.94E-05
Sr-90	7.58E-03	0.00E+00	1.86E-03	0.00E+00	0.00E+00	0.00E+00	2.19E-04
Sr-91	5.67E-06	0.00E+00	2.29E-07	0.00E+00	0.00E+00	0.00E+00	2.70E-05
Sr-92	2.15E-06	0.00E+00	9.30E-08	0.00E+00	0.00E+00	0.00E+00	4.26E-05
Y-90	9.62E-09	0.00E+00	2.58E-10	0.00E+00	0.00E+00	0.00E+00	1.02E-04
Y-91m	9.09E-11	0.00E+00	3.52E-12	0.00E+00	0.00E+00	0.00E+00	2.67E-10
Y-91	1.41E-07	0.00E+00	3.77E-09	0.00E+00	0.00E+00	0.00E+00	7.76E-05
Y-92	8.45E-10	0.00E+00	2.47E-11	0.00E+00	0.00E+00	0.00E+00	1.48E-05
Y-93	2.68E-09	0.00E+00	7.40E-11	0.00E+00	0.00E+00	0.00E+00	8.50E-05
Zr-95	3.04E-08	9.75E-09	6.60E-09	0.00E+00	1.53E-08	0.00E+00	3.09E-05
Zr-97	1.68E-09	3.39E-10	1.55E-10	0.00E+00	5.12E-10	0.00E+00	1.05E-04
Nb-95	6.22E-09	3.46E-09	1.86E-09	0.00E+00	3.42E-09	0.00E+00	2.10E-05
Nb-97	5.22E-11	1.32E-11	4.82E-12	0.00E+00	1.54E-11	0.00E+00	4.87E-08
Mo-99	0.00E+00	4.31E-06	8.20E-07	0.00E+00	9.76E-06	0.00E+00	9.99E-06
Tc-99m	2.47E-10	6.98E-10	8.89E-09	0.00E+00	1.06E-08	3.42E-10	4.13E-07
Tc-101	2.54E-10	3.66E-10	3.59E-09	0.00E+00	6.59E-09	1.87E-10	1.10E-21
Ru-103	1.85E-07	0.00E+00	7.97E-08	0.00E+00	7.06E-07	0.00E+00	2.16E-05
Ru-105	1.54E-08	0.00E+00	6.08E-09	0.00E+00	1.99E-07	0.00E+00	9.42E-06
Ru-106	2.75E-06	0.00E+00	3.48E-07	0.00E+00	5.31E-06	0.00E+00	1.78E-04

OFFSITE DOSE CALCULATION/ENVIRONMENTAL MONITORING METHODOLOGIES

Table 6.4 (2 of 8)
 INGESTION DOSE FACTORS
 (mrem/pCi ingested)

	ADULT						
	bone	liver	t body	thyroid	kidney	lung	gi-lli
Ag-110m	1.60E-07	1.48E-07	8.79E-08	0.00E+00	2.91E-07	0.00E+00	6.04E-05
Sb-124	2.80E-06	5.29E-08	1.11E-06	6.79E-09	0.00E-00	2.18E-06	7.95E-05
Sb-125	1.79E-06	2.00E-08	4.26E-07	1.82E-09	0.00E-00	1.38E-06	1.97E-05
Sn-125	8.33E-06	1.68E-07	3.78E-07	1.39E-07	0.00E-00	0.00E-00	1.04E-04 R25
Te-125m	2.68E-06	9.71E-07	3.59E-07	8.06E-07	1.09E-05	0.00E+00	1.07E-05
Te-127m	6.77E-06	2.42E-06	8.25E-07	1.73E-06	2.75E-05	0.00E+00	2.27E-05
Te-127	1.10E-07	3.95E-08	2.38E-08	8.15E-08	4.48E-07	0.00E+00	8.68E-06
Te-129m	1.15E-05	4.29E-06	1.82E-06	3.95E-06	4.80E-05	0.00E+00	5.79E-05
Te-129	3.14E-08	1.18E-08	7.65E-09	2.41E-08	1.32E-07	0.00E+00	2.37E-08
Te-131m	1.73E-06	8.46E-07	7.05E-07	1.34E-06	8.57E-06	0.00E+00	8.40E-05
Te-131	1.97E-08	8.23E-09	6.22E-09	1.62E-08	8.63E-08	0.00E+00	2.79E-09
Te-132	2.52E-06	1.63E-06	1.53E-06	1.80E-06	1.57E-05	0.00E+00	7.71E-05
I-130	7.56E-07	2.23E-06	8.80E-07	1.89E-04	3.48E-06	0.00E+00	1.92E-06
I-131	4.16E-06	5.95E-06	3.41E-06	1.95E-03	1.02E-05	0.00E+00	1.57E-06
I-132	2.03E-07	5.43E-07	1.90E-07	1.90E-05	8.65E-07	0.00E+00	1.02E-07
I-133	1.42E-06	2.47E-06	7.53E-07	3.63E-04	4.31E-06	0.00E+00	2.22E-06
I-134	1.06E-07	2.88E-07	1.03E-07	4.99E-06	4.58E-07	0.00E+00	2.51E-10
I-135	4.43E-07	1.16E-06	4.28E-07	7.65E-05	1.86E-06	0.00E+00	1.31E-06
Cs-134	6.22E-05	1.48E-04	1.21E-04	0.00E+00	4.79E-05	1.59E-05	2.59E-06
Cs-136	6.51E-06	2.57E-05	1.85E-05	0.00E+00	1.43E-05	1.96E-06	2.92E-06
Cs-137	7.97E-05	1.09E-04	7.14E-05	0.00E+00	3.70E-05	1.23E-05	2.11E-06
Cs-138	5.52E-08	1.09E-07	5.40E-08	0.00E+00	8.01E-08	7.91E-09	4.65E-13
Ba-139	9.70E-08	6.91E-11	2.84E-09	0.00E+00	6.46E-11	3.92E-11	1.72E-07
Ba-140	2.03E-05	2.55E-08	1.33E-06	0.00E+00	8.67E-09	1.46E-08	4.18E-05
Ba-141	4.71E-08	3.56E-11	1.59E-09	0.00E+00	3.31E-11	2.02E-11	2.22E-17
Ba-142	2.13E-08	2.19E-11	1.34E-09	0.00E+00	1.85E-11	1.24E-11	3.00E-26
La-140	2.50E-09	1.26E-09	3.33E-10	0.00E+00	0.00E+00	0.00E+00	9.25E-05
La-142	1.28E-10	5.82E-11	1.45E-11	0.00E+00	0.00E+00	0.00E+00	4.25E-07
Ce-141	9.36E-09	6.33E-09	7.18E-10	0.00E+00	2.94E-09	0.00E+00	2.42E-05
Ce-143	1.65E-09	1.22E-06	1.35E-10	0.00E+00	5.37E-10	0.00E+00	4.56E-05
Ce-144	4.88E-07	2.04E-07	2.62E-08	0.00E+00	1.21E-07	0.00E+00	1.65E-04
Pr-143	9.20E-09	3.69E-09	4.56E-10	0.00E+00	2.13E-09	0.00E+00	4.03E-05
Pr-144	3.01E-11	1.25E-11	1.53E-12	0.00E+00	7.05E-12	0.00E+00	4.33E-18
Nd-147	6.29E-09	7.27E-09	4.35E-10	0.00E+00	4.25E-09	0.00E+00	3.49E-05
W-187	1.03E-07	8.61E-08	3.01E-08	0.00E+00	0.00E+00	0.00E+00	2.82E-05
Np-239	1.19E-09	1.17E-10	6.45E-11	0.00E+00	3.65E-10	0.00E+00	2.40E-05

NOTE: The tritium dose factor for bone is assumed to be equal to the total body dose factor.

REFERENCES:

Regulatory Guide 1.109, Table E-11.
 Dose Factors for Co-57, Zn-69m, Br-82, Nb-97, Sn-125, Sb-124 and Sb-125 are R25 from NUREG-0172 Age Specific Radiation Dose Commitment Factors for a One Year Chronic Intake, November, 1977, Table 4.

OFFSITE DOSE CALCULATION/ENVIRONMENTAL MONITORING METHODOLOGIES

Table 6.4 (3 of 8)
 INGESTION DOSE FACTORS
 (mrem/pCi ingested)

	TEEN						
	bone	liver	t body	thyroid	kidney	lung	gi-lli
H-3	1.06E-07	1.06E-07	1.06E-07	1.06E-07	1.06E-07	1.06E-07	1.06E-07
C-14	4.06E-06	8.12E-07	8.12E-07	8.12E-07	8.12E-07	8.12E-07	8.12E-07
Na-24	2.30E-06	2.30E-06	2.30E-06	2.30E-06	2.30E-06	2.30E-06	2.30E-06
P-32	2.76E-04	1.71E-05	1.07E-05	0.00E+00	0.00E+00	0.00E+00	2.32E-05
Cr-51	0.00E+00	0.00E+00	3.60E-09	2.00E-09	7.89E-10	5.14E-09	6.05E-07
Mn-54	0.00E+00	5.90E-06	1.17E-06	0.00E+00	1.76E-06	0.00E+00	1.21E-05
Mn-56	0.00E+00	1.58E-07	2.81E-08	0.00E+00	2.00E-07	0.00E+00	1.04E-05
Fe-55	3.78E-06	2.68E-06	6.25E-07	0.00E+00	0.00E+00	1.70E-06	1.16E-06
Fe-59	5.87E-06	1.37E-05	5.29E-06	0.00E+00	0.00E+00	4.32E-06	3.24E-05
Co-57	0.00E+00	2.38E-07	3.99E-07	0.00E+00	0.00E+00	0.00E+00	4.44E-06
Co-58	0.00E+00	9.72E-07	2.24E-06	0.00E+00	0.00E+00	0.00E+00	1.34E-05
Co-60	0.00E+00	2.81E-06	6.33E-06	0.00E+00	0.00E+00	0.00E+00	3.66E-05
Ni-63	1.77E-04	1.25E-05	6.00E-06	0.00E+00	0.00E+00	0.00E+00	1.99E-06
Ni-65	7.49E-07	9.57E-08	4.36E-08	0.00E+00	0.00E+00	0.00E+00	5.19E-06
Cu-64	0.00E+00	1.15E-07	5.41E-08	0.00E+00	2.91E-07	0.00E+00	8.92E-06
Zn-65	5.76E-06	2.00E-05	9.33E-06	0.00E+00	1.28E-05	0.00E+00	8.47E-06
Zn-69	1.47E-08	2.80E-08	1.96E-09	0.00E+00	1.83E-08	0.00E+00	5.16E-08
Zn-69m	2.40E-07	5.66E-07	5.19E-08	0.00E+00	3.44E-07	0.00E+00	3.11E-05
Br-82	0.00E+00	0.00E+00	3.04E-06	0.00E+00	0.00E+00	0.00E+00	0.00E+00
Br-83	0.00E+00	0.00E+00	5.74E-08	0.00E+00	0.00E+00	0.00E+00	0.00E+00
Br-84	0.00E+00	0.00E+00	7.22E-08	0.00E+00	0.00E+00	0.00E+00	0.00E+00
Br-85	0.00E+00	0.00E+00	3.05E-09	0.00E+00	0.00E+00	0.00E+00	0.00E+00
Rb-86	0.00E+00	2.98E-05	1.40E-05	0.00E+00	0.00E+00	0.00E+00	4.41E-06
Rb-88	0.00E+00	8.52E-08	4.54E-08	0.00E+00	0.00E+00	0.00E+00	7.30E-15
Rb-89	0.00E+00	5.50E-08	3.89E-08	0.00E+00	0.00E+00	0.00E+00	8.43E-17
Sr-89	4.40E-04	0.00E+00	1.26E-05	0.00E+00	0.00E+00	0.00E+00	5.24E-05
Sr-90	8.30E-03	0.00E+00	2.05E-03	0.00E+00	0.00E+00	0.00E+00	2.33E-04
Sr-91	8.07E-06	0.00E+00	3.21E-07	0.00E+00	0.00E+00	0.00E+00	3.66E-05
Sr-92	3.05E-06	0.00E+00	1.30E-07	0.00E+00	0.00E+00	0.00E+00	7.77E-05
Y-90	1.37E-08	0.00E+00	3.69E-10	0.00E+00	0.00E+00	0.00E+00	1.13E-04
Y-91m	1.29E-10	0.00E+00	4.93E-12	0.00E+00	0.00E+00	0.00E+00	6.09E-09
Y-91	2.01E-07	0.00E+00	5.39E-09	0.00E+00	0.00E+00	0.00E+00	8.24E-05
Y-92	1.21E-09	0.00E+00	3.50E-11	0.00E+00	0.00E+00	0.00E+00	3.32E-05
Y-93	3.83E-09	0.00E+00	1.05E-10	0.00E+00	0.00E+00	0.00E+00	1.17E-04
Zr-95	4.12E-08	1.30E-08	8.94E-09	0.00E+00	1.91E-08	0.00E+00	3.00E-05
Zr-97	2.37E-09	4.69E-10	2.16E-10	0.00E+00	7.11E-10	0.00E+00	1.27E-04
Nb-95	8.22E-09	4.56E-09	2.51E-09	0.00E+00	4.42E-09	0.00E+00	1.95E-05
Nb-97	7.37E-11	1.83E-11	6.68E-12	0.00E+00	2.14E-11	0.00E+00	4.37E-07
Mo-99	0.00E+00	6.03E-06	1.15E-06	0.00E+00	1.38E-05	0.00E+00	1.08E-05
Tc-99m	3.32E-10	9.26E-10	1.20E-08	0.00E+00	1.38E-08	5.14E-10	6.08E-07
Tc-101	3.60E-10	5.12E-10	5.03E-09	0.00E+00	9.26E-09	3.12E-10	8.75E-17
Ru-103	2.55E-07	0.00E+00	1.09E-07	0.00E+00	8.99E-07	0.00E+00	2.13E-05
Ru-105	2.18E-08	0.00E+00	8.46E-09	0.00E+00	2.75E-07	0.00E+00	1.76E-05
Ru-106	3.92E-06	0.00E+00	4.94E-07	0.00E+00	7.56E-06	0.00E+00	1.88E-04

OFFSITE DOSE CALCULATION/ENVIRONMENTAL MONITORING METHODOLOGIES

Table 6.4 (4 of 8)
 INGESTION DOSE FACTORS
 (mrem/pCi ingested)

	TEEN						
	bone	liver	t body	thyroid	kidney	lung	gi-lli
Ag-110m	2.05E-07	1.94E-07	1.18E-07	0.00E+00	3.70E-07	0.00E+00	5.45E-05
Sb-124	3.87E-06	7.13E-08	1.51E-06	8.78E-09	0.00E-00	3.38E-06	7.80E-05
Sb-125	2.48E-06	2.71E-08	5.80E-07	2.37E-09	0.00E+00	2.18E-06	1.93E-05
Sn-125	1.19E-05	2.37E-07	5.37E-07	1.86E-07	0.00E+00	0.00E+00	1.12E-04 R25
Te-125m	3.83E-06	1.38E-06	5.12E-07	1.07E-06	0.00E+00	0.00E+00	1.13E-05
Te-127m	9.67E-06	3.43E-06	1.15E-06	2.30E-06	3.92E-05	0.00E+00	2.41E-05
Te-127	1.58E-07	5.60E-08	3.40E-08	1.09E-07	6.40E-07	0.00E+00	1.22E-05
Te-129m	1.63E-05	6.05E-06	2.58E-06	5.26E-06	6.82E-05	0.00E+00	6.12E-05
Te-129	4.48E-08	1.67E-08	1.09E-08	3.20E-08	1.88E-07	0.00E+00	2.45E-07
Te-131m	2.44E-06	1.17E-06	9.76E-07	1.76E-06	1.22E-05	0.00E+00	9.39E-05
Te-131	2.79E-08	1.15E-08	8.72E-09	2.15E-08	1.22E-07	0.00E+00	2.29E-09
Te-132	3.49E-06	2.21E-06	2.08E-06	2.33E-06	2.12E-05	0.00E+00	7.00E-05
I-130	1.03E-06	2.98E-06	1.19E-06	2.43E-04	4.59E-06	0.00E+00	2.29E-06
I-131	5.85E-06	8.19E-06	4.40E-06	2.39E-03	1.41E-05	0.00E+00	1.62E-06
I-132	2.79E-07	7.30E-07	2.62E-07	2.46E-05	1.15E-06	0.00E+00	3.18E-07
I-133	2.01E-06	3.41E-06	1.04E-06	4.76E-04	5.98E-06	0.00E+00	2.58E-06
I-134	1.46E-07	3.87E-07	1.39E-07	6.45E-06	6.10E-07	0.00E+00	5.10E-09
I-135	6.10E-07	1.57E-06	5.82E-07	1.01E-04	2.48E-06	0.00E+00	1.74E-06
Cs-134	8.37E-05	1.97E-04	9.14E-05	0.00E+00	6.26E-05	2.39E-05	2.45E-06
Cs-136	8.59E-06	3.38E-05	2.27E-05	0.00E+00	1.84E-05	2.90E-06	2.72E-06
Cs-137	1.12E-04	1.49E-04	5.19E-05	0.00E+00	5.07E-05	1.97E-05	2.12E-06
Cs-138	7.76E-08	1.49E-07	7.45E-08	0.00E+00	1.10E-07	1.28E-08	6.76E-11
Ba-139	1.39E-07	9.78E-11	4.05E-09	0.00E+00	9.22E-11	6.74E-11	1.24E-06
Ba-140	2.84E-05	3.48E-08	1.83E-06	0.00E+00	1.18E-08	2.34E-08	4.38E-05
Ba-141	6.71E-08	5.01E-11	2.24E-09	0.00E+00	4.65E-11	3.43E-11	1.43E-13
Ba-142	2.99E-08	2.99E-11	1.84E-09	0.00E+00	2.53E-11	1.99E-11	9.18E-20
La-140	3.48E-09	1.71E-09	4.55E-10	0.00E+00	0.00E+00	0.00E+00	9.82E-05
La-142	1.79E-10	7.95E-11	1.98E-11	0.00E+00	0.00E+00	0.00E+00	2.42E-06
Ce-141	1.33E-08	8.88E-09	1.02E-09	0.00E+00	4.18E-09	0.00E+00	2.54E-05
Ce-143	2.35E-09	1.71E-06	1.91E-10	0.00E+00	7.67E-10	0.00E+00	5.14E-05
Ce-144	6.96E-07	2.88E-07	3.74E-08	0.00E+00	1.72E-07	0.00E+00	1.75E-04
Pr-143	1.31E-08	5.23E-09	6.52E-10	0.00E+00	3.04E-09	0.00E+00	4.31E-05
Pr-144	4.30E-11	1.76E-11	2.18E-12	0.00E+00	1.01E-11	0.00E+00	4.74E-14
Nd-147	9.38E-09	1.02E-08	6.11E-10	0.00E+00	5.99E-09	0.00E+00	3.68E-05
W-187	1.46E-07	1.19E-07	4.17E-08	0.00E+00	0.00E+00	0.00E+00	3.22E-05
Np-239	1.76E-09	1.66E-10	9.22E-11	0.00E+00	5.21E-10	0.00E+00	2.67E-05

NOTE: The tritium dose factor for bone is assumed to be equal to the total body dose factor.

REFERENCES:

Regulatory Guide 1.109, Table E-12.

Dose Factors for Co-57, Zn-69m, Br-82, Nb-97, Sn-125, Sb-124 and Sb-125 are R25 from NUREG-0172 Age Specific Radiation Dose Commitment Factors for a One Year Chronic Intake, November, 1977, Table 4.

OFFSITE DOSE CALCULATION/ENVIRONMENTAL MONITORING METHODOLOGIES

Table 6.4 (5 of 8)
 INGESTION DOSE FACTORS
 (mrem/pCi ingested)

	CHILD						
	bone	liver	t body	thyroid	kidney	lung	gi-lli
H-3	2.03E-07	2.03E-07	2.03E-07	2.03E-07	2.03E-07	2.03E-07	2.03E-07
C-14	1.21E-05	2.42E-06	2.42E-06	2.42E-06	2.42E-06	2.42E-06	2.42E-06
Na-24	5.80E-06	5.80E-06	5.80E-06	5.80E-06	5.80E-06	5.80E-06	5.80E-06
P-32	8.25E-04	3.86E-05	3.18E-05	0.00E+00	0.00E+00	0.00E+00	2.28E-05
Cr-51	0.00E+00	0.00E+00	8.90E-09	4.94E-09	1.35E-09	9.02E-09	4.72E-07
Mn-54	0.00E+00	1.07E-05	2.85E-06	0.00E+00	3.00E-06	0.00E+00	8.98E-06
Mn-56	0.00E+00	3.34E-07	7.54E-08	0.00E+00	4.04E-07	0.00E+00	4.84E-05
Fe-55	1.15E-05	6.10E-06	1.89E-06	0.00E+00	0.00E+00	3.45E-06	1.13E-06
Fe-59	1.65E-05	2.67E-05	1.33E-05	0.00E+00	0.00E+00	7.74E-06	2.78E-05
Co-57	0.00E+00	4.93E-07	9.98E-07	0.00E+00	0.00E+00	0.00E+00	4.04E-06
Co-58	0.00E+00	1.80E-06	5.51E-06	0.00E+00	0.00E+00	0.00E+00	1.05E-05
Co-60	0.00E+00	5.29E-06	1.56E-05	0.00E+00	0.00E+00	0.00E+00	2.93E-05
Ni-63	5.38E-04	2.88E-05	1.83E-05	0.00E+00	0.00E+00	0.00E+00	1.94E-06
Ni-65	2.22E-06	2.09E-07	1.22E-07	0.00E+00	0.00E+00	0.00E+00	2.56E-05
Cu-64	0.00E+00	2.45E-07	1.48E-07	0.00E+00	5.92E-07	0.00E+00	1.15E-05
Zn-65	1.37E-05	3.65E-05	2.27E-05	0.00E+00	2.30E-05	0.00E+00	6.41E-06
Zn-69	4.38E-08	6.33E-08	5.85E-09	0.00E+00	3.84E-08	0.00E+00	3.99E-06
Zn-69m	7.10E-07	1.21E-06	1.43E-07	0.00E+00	7.03E-07	0.00E+00	3.94E-05
Br-82	0.00E+00	0.00E+00	7.55E-06	0.00E+00	0.00E+00	0.00E+00	0.00E+00
Br-83	0.00E+00	0.00E+00	1.71E-07	0.00E+00	0.00E+00	0.00E+00	0.00E+00
Br-84	0.00E+00	0.00E+00	1.98E-07	0.00E+00	0.00E+00	0.00E+00	0.00E+00
Br-85	0.00E+00	0.00E+00	9.12E-09	0.00E+00	0.00E+00	0.00E+00	0.00E+00
Rb-86	0.00E+00	6.70E-05	4.12E-05	0.00E+00	0.00E+00	0.00E+00	4.31E-06
Rb-88	0.00E+00	1.90E-07	1.32E-07	0.00E+00	0.00E+00	0.00E+00	9.32E-09
Rb-89	0.00E+00	1.17E-07	1.04E-07	0.00E+00	0.00E+00	0.00E+00	1.02E-09
Sr-89	1.32E-03	0.00E+00	3.77E-05	0.00E+00	0.00E+00	0.00E+00	5.11E-05
Sr-90	1.70E-02	0.00E+00	4.31E-03	0.00E+00	0.00E+00	0.00E+00	2.29E-04
Sr-91	2.40E-05	0.00E+00	9.06E-07	0.00E+00	0.00E+00	0.00E+00	5.30E-05
Sr-92	9.03E-06	0.00E+00	3.62E-07	0.00E+00	0.00E+00	0.00E+00	1.71E-04
Y-90	4.11E-08	0.00E+00	1.10E-09	0.00E+00	0.00E+00	0.00E+00	1.17E-04
Y-91m	3.82E-10	0.00E+00	1.39E-11	0.00E+00	0.00E+00	0.00E+00	7.48E-07
Y-91	6.02E-07	0.00E+00	1.61E-08	0.00E+00	0.00E+00	0.00E+00	8.02E-05
Y-92	3.60E-09	0.00E+00	1.03E-10	0.00E+00	0.00E+00	0.00E+00	1.04E-04
Y-93	1.14E-08	0.00E+00	3.13E-10	0.00E+00	0.00E+00	0.00E+00	1.70E-04
Zr-95	1.16E-07	2.55E-08	2.27E-08	0.00E+00	3.65E-08	0.00E+00	2.66E-05
Zr-97	6.99E-09	1.01E-09	5.96E-10	0.00E+00	1.45E-09	0.00E+00	1.53E-04
Nb-95	2.25E-08	8.76E-09	6.26E-09	0.00E+00	8.23E-09	0.00E+00	1.62E-05
Nb-97	2.17E-10	3.92E-11	1.83E-11	0.00E+00	4.35E-11	0.00E+00	1.21E-05
Mo-99	0.00E+00	1.33E-05	3.29E-06	0.00E+00	2.84E-05	0.00E+00	1.10E-05
Tc-99m	9.23E-10	1.81E-09	3.00E-08	0.00E+00	2.63E-08	9.19E-10	1.03E-06
Tc-101	1.07E-09	1.12E-09	1.42E-08	0.00E+00	1.91E-08	5.92E-10	3.56E-09
Ru-103	7.31E-07	0.00E+00	2.81E-07	0.00E+00	1.84E-06	0.00E+00	1.89E-05
Ru-105	6.45E-08	0.00E+00	2.34E-08	0.00E+00	5.67E-07	0.00E+00	4.21E-05
Ru-106	1.17E-05	0.00E+00	1.46E-06	0.00E+00	1.58E-05	0.00E+00	1.82E-04

OFFSITE DOSE CALCULATION/ENVIRONMENTAL MONITORING METHODOLOGIES

Table 6.4 (6 of 8)
 INGESTION DOSE FACTORS
 (mrem/pCi ingested)

	CHILD						
	bone	liver	t body	thyroid	kidney	lung	gi-lli
Ag-110m	5.39E-07	3.64E-07	2.91E-07	0.00E+00	6.78E-07	0.00E+00	4.33E-05
Sb-124	1.11E-05	1.44E-07	3.89E-06	2.45E-08	0.00E+00	6.16E-06	6.94E-05
Sb-125	7.16E-06	5.52E-08	1.50E-06	6.63E-09	0.00E+00	3.99E-06	1.71E-05
Sn-125	3.55E-05	5.35E-07	1.59E-06	5.55E-07	0.00E+00	0.00E+00	1.10E-05 R25
Te-125m	1.14E-05	3.09E-06	1.52E-06	3.20E-06	0.00E+00	0.00E+00	1.10E-05
Te-127m	2.89E-05	7.78E-06	3.43E-06	6.91E-06	8.24E-05	0.00E+00	2.34E-05
Te-127	4.71E-07	1.27E-07	1.01E-07	3.26E-07	1.34E-06	0.00E+00	1.84E-05
Te-129m	4.87E-05	1.36E-05	7.56E-06	1.57E-05	1.43E-04	0.00E+00	5.94E-05
Te-129	1.34E-07	3.74E-08	3.18E-08	9.56E-08	3.92E-07	0.00E+00	8.34E-06
Te-131m	7.20E-06	2.49E-06	2.65E-06	5.12E-06	2.41E-05	0.00E+00	1.01E-04
Te-131	8.30E-08	2.53E-08	2.47E-08	6.35E-08	2.51E-07	0.00E+00	4.36E-07
Te-132	1.01E-05	4.47E-06	5.40E-06	6.51E-06	4.15E-05	0.00E+00	4.50E-05
I-130	2.92E-06	5.90E-06	3.04E-06	6.50E-04	8.82E-06	0.00E+00	2.76E-06
I-131	1.72E-05	1.73E-05	9.83E-06	5.72E-03	2.84E-05	0.00E+00	1.54E-06
I-132	8.00E-07	1.47E-06	6.76E-07	6.82E-05	2.25E-06	0.00E+00	1.73E-06
I-133	5.92E-06	7.32E-06	2.77E-06	1.36E-03	1.22E-05	0.00E+00	2.95E-06
I-134	4.19E-07	7.78E-07	3.58E-07	1.79E-05	1.19E-06	0.00E+00	5.16E-07
I-135	1.75E-06	3.15E-06	1.49E-06	2.79E-04	4.83E-06	0.00E+00	2.40E-06
Cs-134	2.34E-04	3.84E-04	8.10E-05	0.00E+00	1.19E-04	4.27E-05	2.07E-06
Cs-136	2.35E-05	6.46E-05	4.18E-05	0.00E+00	3.44E-05	5.13E-06	2.27E-06
Cs-137	3.27E-04	3.13E-04	4.62E-05	0.00E+00	1.02E-04	3.67E-05	1.96E-06
Cs-138	2.28E-07	3.17E-07	2.01E-07	0.00E+00	2.23E-07	2.40E-08	1.46E-07
Ba-139	4.14E-07	2.21E-10	1.20E-08	0.00E+00	1.93E-10	1.30E-10	2.39E-05
Ba-140	8.31E-05	7.28E-08	4.85E-06	0.00E+00	2.37E-08	4.34E-08	4.21E-05
Ba-141	2.00E-07	1.12E-10	6.51E-09	0.00E+00	9.69E-11	6.58E-10	1.14E-07
Ba-142	8.74E-08	6.29E-11	4.88E-09	0.00E+00	5.09E-11	3.70E-11	1.14E-09
La-140	1.01E-08	3.53E-09	1.19E-09	0.00E+00	0.00E+00	0.00E+00	9.84E-05
La-142	5.24E-10	1.67E-10	5.23E-11	0.00E+00	0.00E+00	0.00E+00	3.31E-05
Ce-141	3.97E-08	1.98E-08	2.94E-09	0.00E+00	8.68E-09	0.00E+00	2.47E-05
Ce-143	6.99E-09	3.79E-06	5.49E-10	0.00E+00	1.59E-09	0.00E+00	5.55E-05
Ce-144	2.08E-06	6.52E-07	1.11E-07	0.00E+00	3.61E-07	0.00E+00	1.70E-04
Pr-143	3.93E-08	1.18E-08	1.95E-09	0.00E+00	6.39E-09	0.00E+00	4.24E-05
Pr-144	1.29E-10	3.99E-11	6.49E-12	0.00E+00	2.11E-11	0.00E+00	8.59E-08
Nd-147	2.79E-08	2.26E-08	1.75E-09	0.00E+00	1.24E-08	0.00E+00	3.58E-05
W-187	4.29E-07	2.54E-07	1.14E-07	0.00E+00	0.00E+00	0.00E+00	3.57E-05
Np-239	5.25E-09	3.77E-10	2.65E-10	0.00E+00	1.09E-09	0.00E+00	2.79E-05

NOTE: The tritium dose factor for bone is assumed to be equal to the total body dose factor.

REFERENCES:

Regulatory Guide 1.109, Table E-13.

Dose Factors for Co-57, Zn-69m, Br-82, Nb-97, Sn-125, Sb-124 and Sb-125 are R25 from NUREG-0172 Age Specific Radiation Dose Commitment Factors for a One Year Chronic Intake, November, 1977, Table 4.

OFFSITE DOSE CALCULATION/ENVIRONMENTAL MONITORING METHODOLOGIES

Table 6.4 (7 of 8)
 INGESTION DOSE FACTORS
 (mrem/pCi ingested)

	INFANT						
	bone	liver	t body	thyroid	kidney	lung	gi-lli
H-3	3.08E-07	3.08E-07	3.08E-07	3.08E-07	3.08E-07	3.08E-07	3.08E-07
C-14	2.37E-05	5.06E-06	5.06E-06	5.06E-06	5.06E-06	5.06E-06	5.06E-06
Na-24	1.01E-05	1.01E-05	1.01E-05	1.01E-05	1.01E-05	1.01E-05	1.01E-05
P-32	1.70E-03	1.00E-04	6.59E-05	0.00E+00	0.00E+00	0.00E+00	2.30E-05
Cr-51	0.00E+00	0.00E+00	1.41E-08	9.20E-09	2.01E-09	1.79E-08	4.11E-07
Mn-54	0.00E+00	1.99E-05	4.51E-06	0.00E+00	4.41E-06	0.00E+00	7.31E-06
Mn-56	0.00E+00	8.18E-07	1.41E-07	0.00E+00	7.03E-07	0.00E+00	7.43E-05
Fe-55	1.39E-05	8.98E-06	2.40E-06	0.00E+00	0.00E+00	4.39E-06	1.14E-06
Fe-59	3.08E-05	5.38E-05	2.12E-05	0.00E+00	0.00E+00	1.59E-05	2.57E-05
Co-57	0.00E+00	1.15E-06	1.87E-06	0.00E+00	0.00E+00	0.00E+00	3.92E-06
Co-58	0.00E+00	3.60E-06	8.98E-06	0.00E+00	0.00E+00	0.00E+00	8.97E-06
Co-60	0.00E+00	1.08E-05	2.55E-05	0.00E+00	0.00E+00	0.00E+00	2.57E-05
Ni-63	6.34E-04	3.92E-05	2.20E-05	0.00E+00	0.00E+00	0.00E+00	1.95E-06
Ni-65	4.70E-06	5.32E-07	2.42E-07	0.00E+00	0.00E+00	0.00E+00	4.05E-05
Cu-64	0.00E+00	6.09E-07	2.82E-07	0.00E+00	1.03E-06	0.00E+00	1.25E-05
Zn-65	1.84E-05	6.31E-05	2.91E-05	0.00E+00	3.06E-05	0.00E+00	5.33E-05
Zn-69	9.33E-08	1.68E-07	1.25E-08	0.00E+00	6.98E-08	0.00E+00	1.37E-05
Zn-69m	1.50E-06	3.06E-06	2.79E-07	0.00E+00	1.24E-06	0.00E+00	4.24E-05
Br-82	0.00E+00	0.00E+00	1.27E-05	0.00E+00	0.00E+00	0.00E+00	0.00E+00
Br-83	0.00E+00	0.00E+00	3.63E-07	0.00E+00	0.00E+00	0.00E+00	0.00E+00
Br-84	0.00E+00	0.00E+00	3.82E-07	0.00E+00	0.00E+00	0.00E+00	0.00E+00
Br-85	0.00E+00	0.00E+00	1.94E-08	0.00E+00	0.00E+00	0.00E+00	0.00E+00
Rb-86	0.00E+00	1.70E-04	8.40E-05	0.00E+00	0.00E+00	0.00E+00	4.35E-06
Rb-88	0.00E+00	4.98E-07	2.73E-07	0.00E+00	0.00E+00	0.00E+00	4.85E-07
Rb-89	0.00E+00	2.86E-07	1.97E-07	0.00E+00	0.00E+00	0.00E+00	9.74E-08
Sr-89	2.51E-03	0.00E+00	7.20E-05	0.00E+00	0.00E+00	0.00E+00	5.16E-05
Sr-90	1.85E-02	0.00E+00	4.71E-03	0.00E+00	0.00E+00	0.00E+00	2.31E-04
Sr-91	5.00E-05	0.00E+00	1.81E-06	0.00E+00	0.00E+00	0.00E+00	5.92E-05
Sr-92	1.92E-05	0.00E+00	7.13E-07	0.00E+00	0.00E+00	0.00E+00	2.07E-04
Y-90	8.69E-08	0.00E+00	2.33E-09	0.00E+00	0.00E+00	0.00E+00	1.20E-04
Y-91m	8.10E-10	0.00E+00	2.76E-11	0.00E+00	0.00E+00	0.00E+00	2.70E-06
Y-91	1.13E-06	0.00E+00	3.01E-08	0.00E+00	0.00E+00	0.00E+00	8.10E-05
Y-92	7.65E-09	0.00E+00	2.15E-10	0.00E+00	0.00E+00	0.00E+00	1.46E-04
Y-93	2.43E-08	0.00E+00	6.62E-10	0.00E+00	0.00E+00	0.00E+00	1.92E-04
Zr-95	2.06E-07	5.02E-08	3.56E-08	0.00E+00	5.41E-08	0.00E+00	2.50E-05
Zr-97	1.48E-08	2.54E-09	1.16E-09	0.00E+00	2.56E-09	0.00E+00	1.62E-04
Nb-95	4.20E-08	1.73E-08	1.00E-08	0.00E+00	1.24E-08	0.00E+00	1.46E-05
Nb-97	4.59E-10	9.79E-11	3.53E-11	0.00E+00	7.65E-11	0.00E+00	3.09E-05
Mo-99	0.00E+00	3.40E-05	6.63E-06	0.00E+00	5.08E-05	0.00E+00	1.12E-05
Tc-99m	1.92E-09	3.96E-09	5.10E-08	0.00E+00	4.26E-08	2.07E-09	1.15E-06
Tc-101	2.27E-09	2.86E-09	2.83E-08	0.00E+00	3.40E-08	1.56E-09	4.86E-07
Ru-103	1.48E-06	0.00E+00	4.95E-07	0.00E+00	3.08E-06	0.00E+00	1.80E-05
Ru-105	1.36E-07	0.00E+00	4.58E-08	0.00E+00	1.00E-06	0.00E+00	5.41E-05
Ru-106	2.41E-05	0.00E+00	3.01E-06	0.00E+00	2.85E-05	0.00E+00	1.83E-04

OFFSITE DOSE CALCULATION/ENVIRONMENTAL MONITORING METHODOLOGIES

Table 6.4 (8 of 8)
 INGESTION DOSE FACTORS
 (mrem/pCi ingested)

	INFANT						
	bone	liver	t body	thyroid	kidney	lung	gi-lli
Ag-110m	9.96E-07	7.27E-07	4.81E-07	0.00E+00	1.04E-06	0.00E+00	3.77E-05
Sb-124	2.14E-05	3.15E-07	6.63E-06	5.68E-08	0.00E+00	1.34E-05	6.00E-05
Sb-125	1.23E-05	1.19E-07	2.53E-06	1.54E-08	0.00E+00	7.72E-06	1.64E-05
Sn-125	7.41E-05	1.38E-06	3.29E-06	1.36E-06	0.00E+00	0.00E+00	1.11E-04 R25
Te-125m	2.33E-05	7.79E-06	3.15E-06	7.84E-06	0.00E+00	0.00E+00	1.11E-05
Te-127m	5.85E-05	1.94E-05	7.08E-06	1.69E-05	1.44E-04	0.00E+00	2.36E-05
Te-127	1.00E-06	3.35E-07	2.15E-07	8.14E-07	2.44E-06	0.00E+00	2.10E-05
Te-129m	1.00E-04	3.43E-05	1.54E-05	3.84E-05	2.50E-04	0.00E+00	5.97E-05
Te-129	2.84E-07	9.79E-08	6.63E-08	2.38E-07	7.07E-07	0.00E+00	2.27E-05
Te-131m	1.52E-05	6.12E-06	5.05E-06	1.24E-05	4.21E-05	0.00E+00	1.03E-04
Te-131	1.76E-07	6.50E-08	4.94E-08	1.57E-07	4.50E-07	0.00E+00	7.11E-06
Te-132	2.08E-05	1.03E-05	9.61E-06	1.52E-05	6.44E-05	0.00E+00	3.81E-05
I-130	6.00E-06	1.32E-05	5.30E-06	1.48E-03	1.45E-05	0.00E+00	2.83E-06
I-131	3.59E-05	4.23E-05	1.86E-05	1.39E-02	4.94E-05	0.00E+00	1.51E-06
I-132	1.66E-06	3.37E-06	1.20E-06	1.58E-04	3.76E-06	0.00E+00	2.73E-06
I-133	1.25E-05	1.82E-05	5.33E-06	3.31E-03	2.14E-05	0.00E+00	3.08E-06
I-134	8.69E-07	1.78E-06	6.33E-07	4.15E-05	1.99E-06	0.00E+00	1.84E-06
I-135	3.64E-06	7.24E-06	2.64E-06	6.49E-04	8.07E-06	0.00E+00	2.62E-06
Cs-134	3.77E-04	7.03E-04	7.10E-05	0.00E+00	1.81E-04	7.42E-05	1.91E-06
Cs-136	4.59E-05	1.35E-04	5.04E-05	0.00E+00	5.38E-05	1.10E-05	2.05E-06
Cs-137	5.22E-04	6.11E-04	4.33E-05	0.00E+00	1.64E-04	6.64E-05	1.91E-06
Cs-138	4.81E-07	7.82E-07	3.79E-07	0.00E+00	3.90E-07	6.09E-08	1.25E-06
Ba-139	8.81E-07	5.84E-10	2.55E-08	0.00E+00	3.51E-10	3.54E-10	5.58E-05
Ba-140	1.71E-04	1.71E-07	8.81E-06	0.00E+00	4.06E-08	1.05E-07	4.20E-05
Ba-141	4.25E-07	2.91E-10	1.34E-08	0.00E+00	1.75E-10	1.77E-10	5.19E-06
Ba-142	1.84E-07	1.53E-10	9.06E-09	0.00E+00	8.81E-11	9.26E-11	7.59E-07
La-140	2.11E-08	8.32E-09	2.14E-09	0.00E+00	0.00E+00	0.00E+00	9.77E-05
La-142	1.10E-09	4.04E-10	9.67E-11	0.00E+00	0.00E+00	0.00E+00	6.86E-05
Ce-141	7.87E-08	4.80E-08	5.65E-09	0.00E+00	1.48E-08	0.00E+00	2.48E-05
Ce-143	1.48E-08	9.82E-06	1.12E-09	0.00E+00	2.86E-09	0.00E+00	5.73E-05
Ce-144	2.98E-06	1.22E-06	1.67E-07	0.00E+00	4.93E-07	0.00E+00	1.71E-04
Pr-143	8.13E-08	3.04E-08	4.03E-09	0.00E+00	1.13E-08	0.00E+00	4.29E-05
Pr-144	2.74E-10	1.06E-10	1.38E-11	0.00E+00	3.84E-11	0.00E+00	4.93E-06
Nd-147	5.53E-08	5.68E-08	3.48E-09	0.00E+00	2.19E-08	0.00E+00	3.60E-05
W-187	9.03E-07	6.28E-07	2.17E-07	0.00E+00	0.00E+00	0.00E+00	3.69E-05
Np-239	1.11E-08	9.93E-10	5.61E-10	0.00E+00	1.98E-09	0.00E+00	2.87E-05

NOTE: The tritium dose factor for bone is assumed to be equal to the total body dose factor.

REFERENCES:

Regulatory Guide 1.109, Table E-14.

Dose Factors for Co-57, Zn-69m, Br-82, Nb-97, Sn-125, Sb-124 and Sb-125 are R25 from NUREG-0172 Age Specific Radiation Dose Commitment Factors for a One Year Chronic Intake, November, 1977, Table 4.

OFFSITE DOSE CALCULATION/ENVIRONMENTAL MONITORING METHODOLOGIES

Table 6.5
 BIOACCUMULATION FACTORS FOR FRESHWATER FISH
 ($\mu\text{Ci}/\text{kg}$ per $\mu\text{Ci}/\text{ml}$)

H-3	9.0E-01	Y-91m	2.5E+01	I-134	4.0E+01	
C-14	4.6E+03	Y-91	2.5E+01	I-135	4.0E+01	
Na-24	1.0E+02	Y-92	2.5E+01	Cs-134	1.9E+03	
P-32	1.0E+05	Y-93	2.5E+01	Cs-136	1.9E+03	
Cr-51	2.0E+02	Zr-95	3.3E+00	Cs-137	1.9E+03	
Mn-54	4.0E+02	Zr-97	3.3E+00	Cs-138	1.9E+03	
Mn-56	4.0E+02	Nb-95	3.0E+04	Ba-139	4.0E+00	
Fe-55	1.0E+02	Nb-97	3.0E+04	Ba-140	4.0E+00	
Fe-59	1.0E+02	Mo-99	1.0E+01	Ba-141	4.0E+00	
Co-57	5.0E+01	Tc-99m	1.5E+01	Ba-142	4.0E+00	
Co-58	5.0E+01	Tc-101	1.5E+01	La-140	2.5E+01	
Co-60	5.0E+01	Ru-103	1.0E+01	La-144	2.5E+01	
Ni-63	1.0E+02	Ru-105	1.0E+01	Ce-141	1.0E+00	
Ni-65	1.0E+02	Ru-106	1.0E+01	Ce-143	1.0E+00	
Cu-64	5.0E+01	Ag-110m	2.31E+00	Ce-144	1.0E+00	R25
Zn-65	2.0E+03	Sb-124	1.0E+00	Pr-144	2.5E+01	
Zn-69	2.0E+03	Sb-125	1.0E+00	Pr-143	2.5E+01	
Zn-69m	2.0E+03	Sn-125	3.0E+03	Nd-147	2.5E+01	R25
Br-82	4.2E+02	Te-125m	4.0E+02	W-187	1.2E+03	
Br-83	4.2E+02	Te-127m	4.0E+02	Np-239	1.0E+01	
Br-84	4.2E+02	Te-127	4.0E+02			
Br-85	4.2E+02	Te-129m	4.0E+02			
Rb-86	2.0E+03	Te-129	4.0E+02			
Rb-88	2.0E+03	Te-131m	4.0E+02			
Rb-89	2.0E+03	Te-131	4.0E+02			
Sr-89	5.6E+01	Te-132	4.0E+02			
Sr-90	5.6E+01	I-130	4.0E+01			
Sr-91	5.6E+01	I-131	4.0E+01			
Sr-92	5.6E+01	I-132	4.0E+01			
Y-90	2.5E+01	I-133	4.0E+01			

REFERENCES:

Bioaccumulation factors for Ag-110m, Sb-124, Sb-125 and Sn-125 are from R25
 ORNL-4992, "A Methodology for Calculating Radiation Doses from
 Radioactivity Released to the Environment," March 1976, Table 4.12A.

Bioaccumulation factors for Iodine, Cesium, and Strontium nuclides are from
 NUREG/CR-1004, Table 3.2.4.

All other nuclides' bioaccumulation factors are from Regulatory Guide 1.109,
 Table A-1.

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Table 6.6 (1 of 2)
 EXTERNAL DOSE FACTORS FOR STANDING ON CONTAMINATED GROUND
 (mrem/h per pCi/m²)

<u>Nuclide</u>	<u>Total Body</u>	<u>Skin</u>
H-3	0.0	0.0
C-14	0.0	0.0
Na-24	2.50E-08	2.90E-08
P-32	0.0	0.0
Cr-51	2.20E-10	2.60E-10
Mn-54	5.80E-09	6.80E-09
Mn-56	1.10E-08	1.30E-08
Fe-55	0.0	0.0
Fe-59	8.00E-09	9.40E-09
Co-57	1.77E-09	2.21E-09
Co-58	7.00E-09	8.20E-09
Co-60	1.70E-08	2.00E-08
Ni-63	0.0	0.0
Ni-65	3.70E-09	4.30E-09
Cu-64	1.50E-09	1.70E-09
Zn-65	4.00E-09	4.60E-09
Zn-69	0.0	0.0
Zn-69m	5.50E-09	6.59E-09
Br-82	3.18E-08	3.90E-08
Br-83	6.40E-11	9.30E-11
Br-84	1.20E-08	1.40E-08
Br-85	0.0	0.0
Rb-86	6.30E-10	7.20E-10
Rb-88	3.50E-09	4.00E-09
Rb-89	1.50E-08	1.80E-08
Sr-89	5.60E-13	6.50E-13
Sr-91	7.10E-09	8.30E-09
Sr-92	9.00E-09	1.00E-08
Y-90	2.70E-12	2.60E-12
Y-91m	3.80E-09	4.40E-09
Y-91	2.40E-11	2.70E-11
Y-92	1.60E-09	1.90E-09
Y-93	5.70E-10	7.80E-10
Zr-95	5.00E-09	5.80E-09
Zr-97	5.50E-09	6.40E-09
Nb-95	5.10E-09	6.00E-09
Nb-97	8.11E-09	1.00E-08
Mo-99	1.90E-09	2.20E-09
Tc-99m	9.60E-10	1.10E-09
Tc-101	2.70E-09	3.00E-09
Ru-103	3.60E-09	4.20E-09
Ru-105	4.50E-09	5.10E-09
Ru-106	1.50E-09	1.80E-09

OFFSITE DOSE CALCULATION/ENVIRONMENTAL MONITORING METHODOLOGIES

Table 6.6 (2 of 2)
 EXTERNAL DOSE FACTORS FOR STANDING ON CONTAMINATED GROUND
 (mrem/h per pCi/m²)

<u>Nuclide</u>	<u>Total Body</u>	<u>Skin</u>
Ag-110m	1.80E-08	2.10E-08
Sb-124	2.17E-08	2.57E-08
Sb-125	5.48E-09	6.80E-09
Sn-125	3.58E-09	4.51E-09
Te-125m	3.50E-11	4.80E-11
Te-127m	1.10E-12	1.30E-12
Te-127	1.00E-11	1.10E-11
Te-129m	7.70E-10	9.00E-10
Te-129	7.10E-10	8.40E-10
Te-131m	8.40E-09	9.90E-09
Te-131	2.20E-09	2.60E-06
Te-132	1.70E-09	2.00E-09
I-130	1.40E-08	1.70E-08
I-131	2.80E-09	3.40E-09
I-132	1.70E-08	2.00E-08
I-133	3.70E-09	4.50E-09
I-134	1.60E-08	1.90E-08
I-135	1.20E-08	1.40E-08
Cs-134	1.20E-08	1.40E-08
Cs-136	1.50E-08	1.70E-08
Cs-137	4.20E-09	4.90E-09
Cs-138	2.10E-08	2.40E-08
Ba-139	2.40E-09	2.70E-09
Ba-140	2.10E-09	2.40E-09
Ba-141	4.30E-09	4.90E-09
Ba-142	7.90E-09	9.00E-09
La-140	1.50E-08	1.70E-08
La-142	1.50E-08	1.80E-08
Ce-141	5.50E-10	6.20E-10
Ce-143	2.20E-09	2.50E-09
Ce-144	3.20E-10	3.70E-10
Pr-143	0.0	0.0
Pr-144	2.00E-10	2.30E-10
Nd-147	1.00E-09	1.20E-09
W-187	3.10E-09	3.60E-09
Np-239	9.50E-10	1.10E-09

R25

REFERENCES:

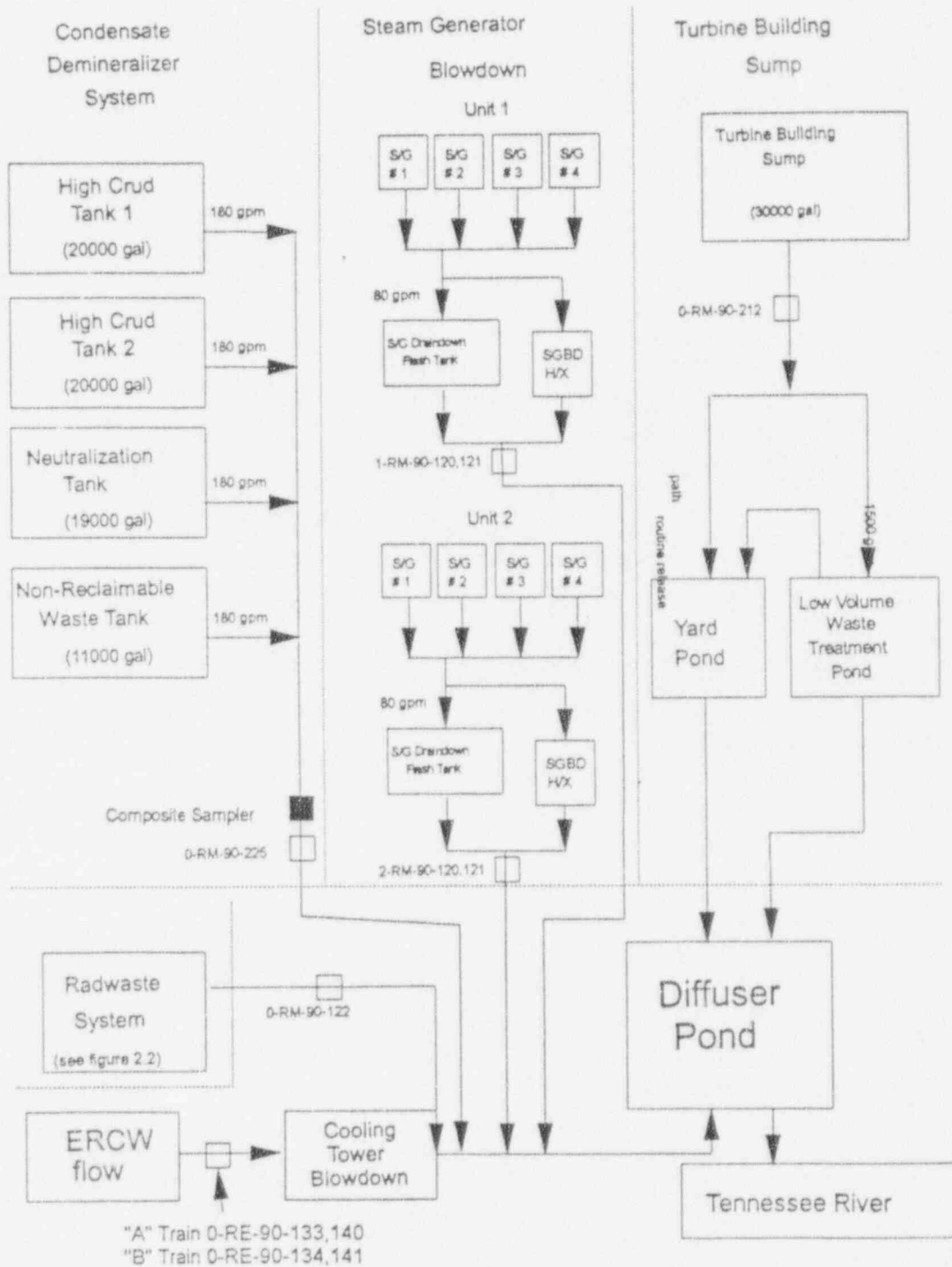
Regulatory Guide 1.109, Table E-6.

Dose Factors for Co-57, Zn-69m, Br-82, Nb-97, Sn-125, Sb-124 and Sb-125 are from Dose-Rate Conversion Factors for External Exposure to Photon and Electron Radiation from Radionuclides Occurring in Routine Releases from Nuclear Fuel Cycle Facilities, D. C. Kocher, Health Physics Volume 38, April 1980.

R25

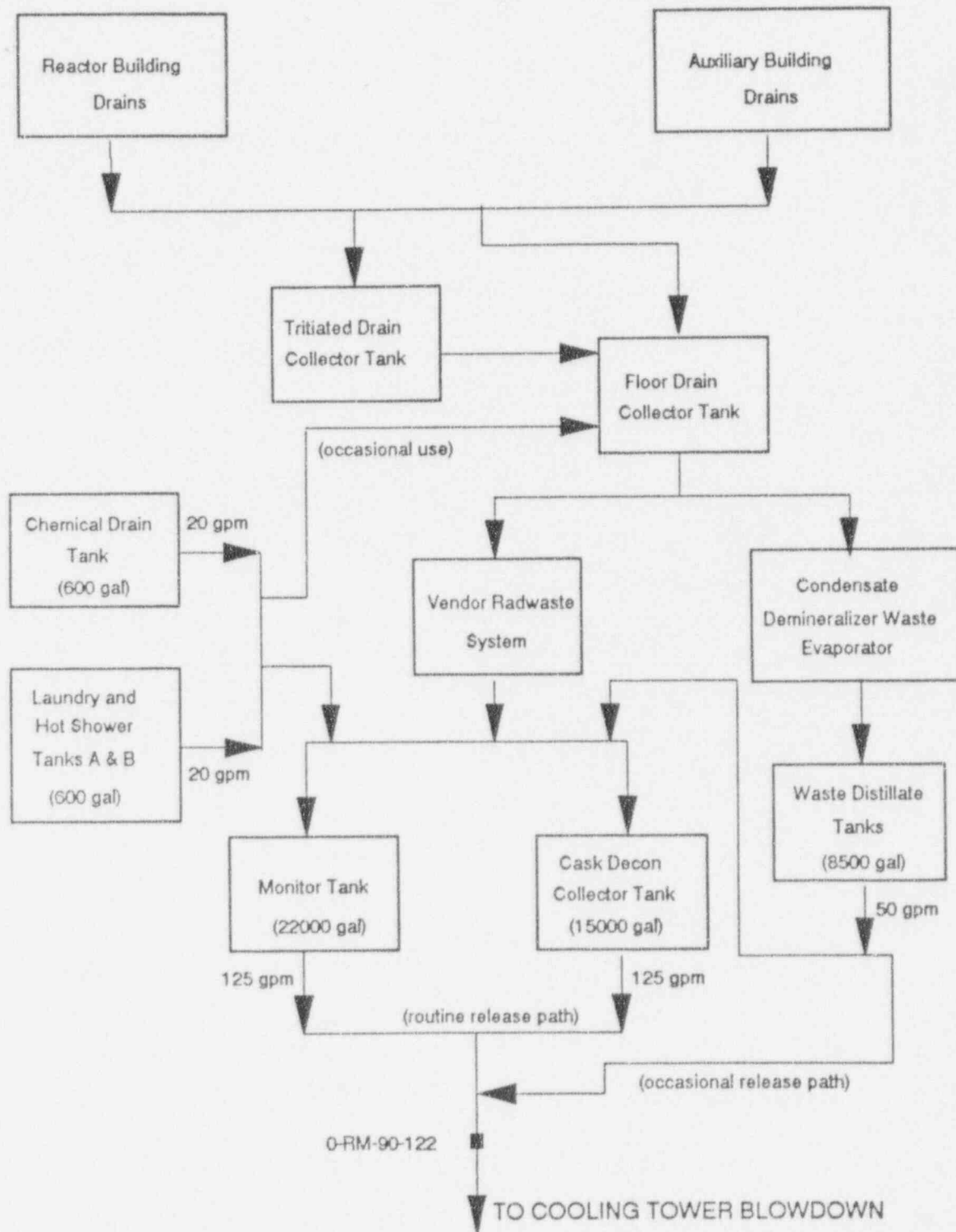
OFFSITE DOSE CALCULATION/ENVIRONMENTAL MONITORING METHODOLOGIES

Figure 6.1
 LIQUID EFFLUENT RELEASE POINTS



OFFSITE DOSE CALCULATION/ENVIRONMENTAL MONITORING METHODOLOGIES

Figure 6.2
 LIQUID RADWASTE SYSTEM



OFFSITE DOSE CALCULATION/ENVIRONMENTAL MONITORING METHODOLOGIES

SECTION 7.0

GASEOUS EFFLUENTS

OFFSITE DOSE CALCULATION/ENVIRONMENTAL MONITORING METHODOLOGIES

RELEASE POINTS DESCRIPTION

There are six exhausts at Sequoyah Nuclear Plant that are monitored for airborne effluents. These are: a Condenser Vacuum Exhaust for each unit, a Service Building Exhaust, an Auxiliary Building Exhaust and a Shield Building Exhaust for each unit. Figure 7.1 provides an outline of the airborne effluent release and discharge points with associated radiation monitor identifications.

Condenser Vacuum Exhaust

The Condenser Vacuum Exhausts (CVEs) are located in the turbine building. They exhaust at a maximum design flow rate of 45 cubic feet per minute. They are monitored by radiation monitors 1,2-RM-90-99,119. R27

Service Building Vent

Various low-level sources exhaust to the Service Building Vent. This exhausts at a total design flow of approximately 14,950 cfm. The portion this total flow originating from the Titration Room, the Waste Baler Room, and the Chemistry Lab is monitored by radiation monitor 0-RM-90-132. R25
R27
R27
R27

Auxiliary Building Exhaust (see Figure 7.2 for detail)

The annulus vacuum priming system exhausts through the containment vent to the Auxiliary Building. The Auxiliary Building exhaust mixes with the General Exhaust System and they cumulatively exhaust at a maximum design flow of 228,000 cfm. The exhaust is monitored by radiation monitor 0-RM-90-101.

Shield Building Vent (see Figure 7.2 for detail)

The Auxiliary Building Gas Treatment System (ABGTS) draws from the Auxiliary Building and exhausts to the waste gas header. There are nine Waste Gas Decay Tanks (WGDTs) that also empty into this header. Either ABGTS or the Emergency Gas Treatment System (EGTS) is run to release a WGDT. Each WGDT has a design capacity of 600 cubic feet and a design release rate of 22.5 cfm. Both the Containment Purge and the Incore Instrument Room Purge from each unit tie into the waste gas header. The Containment Purge exhausts at a maximum of 28,000 cfm and is monitored by radiation monitors 1,2-RM-90-130,131. If the Incore Instrument Room Purge is operating exclusively, it exhausts at 800 cfm. Under emergency conditions, and sometimes during normal operation, the EGTS is used to draw a vacuum in the annulus and exhaust to the Shield Building Vent. Auxiliary Building Isolation starts both the ABGTS and EGTS. The common header exhausts to the Shield Building Exhaust. There is one exhaust for each unit. This exhausts at a maximum design flow of 28,000 cfm and is monitored by radiation monitors 1,2-RE-90-400. R27
R27

Reformatting/Renumbering Changes Only R26
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OFFSITE DOSE CALCULATION/ENVIRONMENTAL MONITORING METHODOLOGIES

7.1 INSTRUMENT SETPOINTS

Airborne effluent monitor setpoints are determined to ensure that the dose rate at the SITE BOUNDARY does not exceed the dose rate limits given in ODCM Control 1.2.2.1 and to identify any unexpected releases.

7.1.1 Release Point Monitor Setpoints (1,2-RM-90-130,131 0-RM-90-118) R27

7.1.1.1 Containment Purge Effluent Monitors (1,2-RM-90-130,131) R27

These monitors are set at a cpm value equal to a percentage of the Technical Specification Limit of 8.5×10^{-3} $\mu\text{Ci/cc}$ of Xe-133 (Technical Specification 3.3.2.1, Table 3.3-4).

7.1.1.2 Waste Gas Decay Tank Effluent Monitor (0-RM-90-118)

For each release from a waste gas decay tank, two setpoints are calculated for the monitor: one based on the expected response of the monitor to the radioactivity in the effluent stream; and a calculated maximum setpoint which corresponds to the most restrictive dose rate limit given in ODCM Control 1.2.2.1. The expected monitor response is calculated as described below in Equation 7.1. The maximum calculated setpoint is calculated as described below in Equation 7.2. A comparison is made between these two calculated setpoints to determine which is used. The actual monitor setpoint for the release is set equal to X times the expected monitor response, or to the maximum calculated setpoint, whichever is less. X is an administrative factor designed to account for expected variations in monitor response (it will be defined in approved plant instructions). The X times expected response setpoint allows for the identification of any release of radioactivity above the expected amount. The maximum calculated setpoint ensures that the release will be stopped if it exceeds the 10 CFR 20 dose rate limits after dilution. R27

Expected Monitor Response

$$R = B + \sum_i \text{eff}_i C_i \quad (7.1)$$

where

- B = monitor background, cpm.
eff_i = efficiency factor for the monitor for nuclide i, cpm per $\mu\text{Ci/cc}$.
C_i = measured concentration of nuclide i, $\mu\text{Ci/cc}$.

OFFSITE DOSE CALCULATION/ENVIRONMENTAL MONITORING METHODOLOGIES

Calculated Maximum Setpoint

The calculated maximum setpoint, S_{max} in cpm, corresponding to the dose rate limit is determined using the following equation:

$$S_{max} = (A SF (\frac{DR_{lim}}{DR} (R - B))) + B \quad (7.2)$$

where

- A = dose rate allocation factor for the release point, dimensionless. The dose rate allocation factors for release points are defined in approved plant procedures.
- SF = safety factor for the monitor, dimensionless. Safety factors for each monitor are defined in approved plant procedures.
- DR_{lim} = the dose rate limit, mrem/year.
= 500 mrem/year to the total body for noble gases,
= 3000 mrem/year to the skin for noble gases, and
= 1500 mrem/year to the maximum organ for iodines and particulates.
- DR = the calculated dose rate for the release, mrem/year.
= DR_{TB} for total body (as described in Section 7.2.3.1),
= DR_S for skin (as described in Section 7.2.3.2), and
= DR_{org} for maximum organ (as described in Section 7.2.4).
- R = expected monitor response (as calculated in Equation 7.1), cpm. R27
- B = the monitor background, cpm. R27

7.1.2 Discharge Point Monitor Setpoints (1,2-RE-90-400, 0-RM-90-101, 0-RM-90-132, 1,2-RM-90-99,119) R27
R27

A normal default setpoint is determined for each discharge point monitor as described in Section 7.1.3. These setpoints on the discharge monitors will routinely be set equal to the default setpoints. When release permits are generated, the expected response and maximum calculated setpoints are calculated for the appropriate discharge monitor as described in Section 7.1.1.2. A comparison is made between the three setpoints as described below to choose the appropriate setpoint for the monitor during the release (after the release, the monitor should be returned to the default setpoint). For almost all releases, the setpoint for the discharge monitor will be the default setpoint. R27

OFFSITE DOSE CALCULATION/ENVIRONMENTAL MONITORING METHODOLOGIES

IF X^1 times the expected monitor response is less than the calculated maximum setpoint:

- a. IF X^1 times the expected monitor response is less than the normal default setpoint, AND the default setpoint is less than the maximum calculated setpoint, THEN the setpoint shall be set equal to the normal default setpoint,
- b. OTHERWISE the setpoint shall be set equal to X^1 times the expected monitor response.

¹ X is an administrative factor designed to account for expected variations in monitor response. It will be defined in approved plant instructions.

NOTE: For the shield building exhaust monitors (1,2-RE-90-400), the above calculations and comparisons are performed in cpm, then the final resulting setpoint is converted to units of $\mu\text{Ci}/\text{sec}$. R27

7.1.3 Discharge Point Effluent Monitor Default Setpoints

7.1.3.1 Shield Building Vents (1,2-RE-90-400), Auxiliary Building Vent (0-RM-90-101), and Service Building Vent (0-RM-90-132) R27

These discharge point effluent monitors are set to ensure compliance with ODCM Control 1.2.2.1. The default setpoints are defined as the maximum calculated setpoint described by Equation 7.2, calculated for Xe-133. The default setpoints for the shield building monitors are calculated in units of $\mu\text{Ci}/\text{sec}$. R27

7.1.3.2 Condenser Vacuum Exhaust Vent (1,2-RM-90-99,119) R27

This discharge point effluent monitor is set to ensure compliance with ODCM Control 1.2.2.1 and to identify the presence of primary to secondary leakage of radioactivity. The default setpoint is determined by calculating the maximum calculated setpoint described by Equation 7.2 for Xe-133, and then taking a percentage of this value as the setpoint. Once a primary to secondary leak is identified, the setpoint on this monitor may be adjusted upward to enable it to be used to identify any further increases in the leak rate. R27

OFFSITE DOSE CALCULATION/ENVIRONMENTAL MONITORING METHODOLOGIES

7.2 GASEOUS EFFLUENTS - DOSE RATES

7.2.1 REPORTING LIMITS

10 CFR 50.73 requires that any airborne radioactivity release that exceeds 2 times the applicable concentrations of the limits specified in Appendix B, Table II of 10 CFR 20 in UNRESTRICTED AREAS when averaged over a period of one hour be reported to the NRC within 30 days. For the purposes of meeting this requirement, it is assumed that the dose rate limits stated in ODCM Control 1.2.2.1 are the result of offsite concentrations equal to those listed in Appendix B, Table II of 10 CFR 20.

7.2.2 RELEASE SAMPLING

Prior to each release (excluding an Incore Instrument Room Purge), a grab sample is taken and analyzed to determine the concentration, $\mu\text{Ci/cc}$, of each noble gas nuclide. On at least a weekly basis, filters are analyzed to determine the amount of iodines and particulates released. Composite samples are maintained (as required by Table 2.2-2) to determine the concentration of certain nuclides (Sr-89, Sr-90, and alpha emitters).

For those nuclides whose activities are determined from composite samples the concentrations for the previous composite period will be assumed as the concentration for the next period to perform the calculations in Sections 7.3, 7.4, and 7.5. The actual measured concentrations will be used for the dose calculations described in Section 7.6.

7.2.3 NOBLE GAS DOSE RATES

Dose rates are calculated for total body and skin due to submersion within a cloud of noble gases using a semi-infinite cloud model.

7.2.3.1 Total Body Dose Rate

The dose rate to the total body, DR_{TB} in mrem/year, is calculated using the following equation:

$$\text{DR}_{\text{TB}} = (\bar{X}/Q) F \sum_i C_i \text{DF}_{\text{Bi}} \quad (7.3)$$

where

\bar{X}/Q = relative concentration, s/m^3 . Relative air concentrations are calculated for the land-site boundary in each of the sixteen sectors as described in Section 7.8.2 using the historical meteorological data for the period

OFFSITE DOSE CALCULATION/ENVIRONMENTAL MONITORING METHODOLOGIES

1972-1975 given in Table 7.2. For dose rate calculations, the highest value from the sixteen land-site boundary locations is used.

= 5.12E-06 s/m³ (from Table 7.1).

F = flowrate of effluent stream, cc/s.

C_i = concentration of noble gas nuclide i in effluent stream, μCi/cc.

DFB_i = total body dose factor due to gamma radiation for noble gas nuclide i, mrem/y per μCi/m³ (Table 7.3). R27

7.2.3.2 Skin Dose Rate

The dose rate to the skin, DR_S in mrem/year, is calculated using the following equation:

$$DR_S = (X/Q) F \sum_i C_i (DFS_i + 1.11 DF_{\gamma i}) \quad (7.4)$$

where

X/Q = relative concentration, s/m³. Relative air concentrations are calculated for the land-site boundary in each of the sixteen sectors as described in Section 7.8.2 using the historical meteorological data for the period 1972-1975 given in Table 7.2. For dose rate calculations, the highest value from the sixteen land-site boundary locations is used.

= 5.12E-06 s/m³ (from Table 7.1).

F = flowrate of effluent stream, cc/s.

C_i = concentration of noble gas nuclide i in effluent stream, μCi/cc.

DFS_i = skin dose factor due to beta radiation for noble gas nuclide i, mrem/y per μCi/m³ (Table 7.3). R27

1.11 = the average ratio of tissue to air energy absorption coefficients, mrem/mrad.

DF_{γi} = dose conversion factor for external gamma for noble gas nuclide i, mrad/year per μCi/m³ (Table 7.3). R27

OFFSITE DOSE CALCULATION/ENVIRONMENTAL MONITORING METHODOLOGIES

7.2.4 I-131, I-133, TRITIUM AND ALL RADIONUCLIDES IN PARTICULATE FORM WITH HALF-LIVES OF GREATER THAN 8 DAYS - ORGAN DOSE RATE

Organ dose rates due to I-131, I-133, Tritium and all radionuclides in particulate form with half-lives of greater than 8 days, DR_{org} in mrem/year, are calculated for all age groups (adult, teen, child, and infant) and all organs (bone, liver, total body, thyroid, kidney, lung, and GI Tract) using the following equation:

$$DR_{org} = F [C_T (X/Q) [R_{IT} + R_{CTP}] + \sum_i C_i [(X/Q) R_{Ii} + (D/Q) [R_{Cpi} + R_{Gi}]]] \quad (7.5)$$

where:

- F = flowrate of effluent stream, cc/s.
- C_T = concentration of tritium in effluent stream, $\mu\text{Ci}/\text{cc}$.
- X/Q = relative concentration, s/m^3 . Relative air concentrations are calculated for the land-site boundary in each of the sixteen sectors as described in Section 7.8.2 using the historical meteorological data for the period 1972-1975 given in Table 7.2. For dose rate calculations, the highest value from the sixteen land-site boundary locations is used.
- R_{IT} = $5.12\text{E}-06 \text{ s}/\text{m}^3$ (from Table 7.1).
- R_{CTP} = inhalation dose factor for tritium, mrem/year per $\mu\text{Ci}/\text{m}^3$. Dose factor is calculated as described in Section 7.7.13.
- R_{CTP} = Grass-cow-milk dose factor for tritium, mrem/year per $\mu\text{Ci}/\text{m}^3$. Dose factor is calculated as described in Section 7.7.7.
- C_i = concentration of nuclide i in effluent stream, $\mu\text{Ci}/\text{cc}$.
- R_{Ii} = inhalation dose factor for each identified nuclide i, mrem/year per $\mu\text{Ci}/\text{m}^3$. Dose factors are calculated as described in Section 7.7.13.
- D/Q = relative deposition, $1/\text{m}^2$. Relative deposition is calculated for the land-site boundary in each of the sixteen sectors as described in Section 7.8.3 using the historical meteorological data for the period 1972-1975 given in Table 7.2. For dose rate calculations, the highest value from the sixteen land-site boundary locations is used.
- R_{Cpi} = $1.29\text{E}-08 \text{ 1}/\text{m}^2$ (from Table 7.1).
- R_{Cpi} = Grass-cow-milk dose factor for each identified nuclide i, m^2 -mrem/year per $\mu\text{Ci}/\text{s}$. Dose factors are calculated as described in Section 7.7.1.
- R_{Gi} = ground plane dose factor for each identified nuclide i, m^2 -mrem/year per $\mu\text{Ci}/\text{s}$. Dose factors are calculated as described in Section 7.7.14.

The maximum organ dose rate is selected from among the dose rates calculated for all the organs and all age groups.

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7.3 DOSE - NOBLE GASES

Doses to be calculated are gamma and beta air doses due to exposure to an infinite cloud of noble gases. These doses will be calculated at the land-site boundary location with the highest annual-average X/Q based on 1972-1975 meteorological data (Table 7.2). This location is chosen from the SITE BOUNDARY locations listed in Table 7.1. Dispersion factors are calculated using the methodology described in Section 7.8.2. R27

No credit is taken for radioactive decay.

7.3.1 Gamma dose to air

The gamma air dose, D_Y in mrad, is calculated for each release using the following equation:

$$D_Y = 1.9E-06 (X/Q) \sum_i Q_i DF_{Yi} T \quad (7.6)$$

where:

- 1.9E-06 = conversion factor, years per minute.
- X/Q = highest land-site boundary annual-average relative concentration, 5.12×10^{-6} s/m³ (from Table 7.1).
- Q_i = release rate for nuclide i, μ Ci/s.
- DF_{Yi} = dose conversion factor for external gamma for nuclide i (Table 7.3), mrad/year per μ Ci/m³. R27
- T = duration of release, minutes.

The gamma-air dose calculated by this method will be used in the cumulative dose calculations discussed in Section 7.3.3.

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7.3.2 Beta dose to air

The beta air dose, D_{β} in mrad, is calculated for each release using the following equation:

$$D_{\beta} = 1.9E-06 (X/Q) \sum_i Q_i DF_{\beta i} T \quad (7.7)$$

where:

- 1.9E-06 = conversion factor, years per minute.
- X/Q = highest land-site boundary annual-average relative concentration, 5.12×10^{-6} s/m³ (from Table 7.1).
- Q_i = release rate for nuclide i , $\mu\text{Ci/s}$.
- $DF_{\beta i}$ = dose conversion factor for external beta for nuclide i , mrad/year per $\mu\text{Ci/m}^3$ (from Table 7.3). R27
- T = duration of release, minutes.

The beta-air dose calculated by this method will be used in the cumulative dose calculations discussed in Section 7.3.3.

7.3.3 Cumulative Dose - Noble Gas

Quarterly and annual sums of all doses are calculated for each release as described below to compare to the limits listed in ODCM Control 1.2.2.2.

For noble gases, cumulative doses are calculated for gamma and beta air doses. Doses due to each release are summed with the doses for all previous release in the quarter or year to obtain cumulative quarterly and annual doses.

7.3.4 Comparison to Limits

The cumulative calendar quarter and calendar year doses are compared to their respective limits once per 31 days to determine compliance.

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7.4 DOSE DUE TO I-131, I-133, TRITIUM AND ALL RADIONUCLIDES IN PARTICULATE FORM WITH HALF-LIVES OF GREATER THAN 8 DAYS

7.4.1 Organ dose Calculation

Organ doses due to I-131, I-133, tritium and all radionuclides in particulate form with half-lives of greater than 8 days are calculated for each release for the critical receptor. The critical receptor is defined as the land-site boundary in the sector with the highest annual average X/Q . The annual average X/Q and D/Q are calculated using the methodology in Sections 7.8.2 and 7.8.3 using the historical 1972-1975 meteorological data (Table 7.2). Pathways considered to exist at this location are inhalation, ground plane exposure, grass-cow-milk ingestion, grass-cow-beef ingestion and fresh leafy and stored vegetable ingestion. All age groups are considered (adult, teen, child and infant). Dose factors for these age groups and pathways are calculated as described in Section 7.7. For the ground exposure pathway, which has no age or organ specific dose factors, the total body dose will be added to the internal organ doses for all age groups. No credit is taken for radioactive decay.

The general equation for the calculation of organ dose is:

$$D_{org} = 3.17E-08 T \sum_i \sum_P R_{pi} [W_p Q_i] \quad (7.8)$$

where:

- 3.17E-08 = conversion factor, year/second
- T = duration of release, seconds.
- R_{pi} = dose factor for pathway P for each identified nuclide i, m^2 -mrem/year per $\mu Ci/s$ for ground plane, grass-cow-milk, grass-cow-meat, and vegetation pathways, and mrem/year per $\mu Ci/m^3$ for inhalation and tritium ingestion pathways. Equations for calculating these dose factors are given in Section 7.7.
- W_p = dispersion factor for the location and pathway,
 = X/Q for the inhalation and tritium ingestion pathways,
 = $5.12E-06 s/m^3$.
 = D/Q for the food and ground plane pathways,
 = $1.29E-08 m^{-2}$
- Q_i = release rate for radionuclide i, $\mu Ci/s$

From the four age groups considered, the maximum is determined by comparing all organ doses for all age groups. The age group with the highest single organ dose is selected as the critical age group. The organ doses for the critical age group will be used in the cumulative doses discussed in Section 7.4.2.

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7.4.2 Cumulative Doses

Quarterly and annual sums of all doses are calculated for each release as described below to compare to the limits listed in ODCM Control 1.2.2.3.

For maximum organ dose, cumulative quarterly and annual doses are maintained for each of the eight organs considered. The cumulative dose is obtained by summing the doses for each organ of the critical age group (as calculated in Section 7.4.1) as determined for each release with the organ doses for all previous releases in the quarter or year to obtain the cumulative quarterly and annual doses. Thus, the cumulative organ doses will be conservative values, consisting of doses belonging to various age groups depending on the mix of radionuclides. The highest of these cumulative organ doses is used for the comparison to the limits described in ODCM Control 1.2.2.3.

7.4.3 Comparison to Limits

The cumulative calendar quarter and calendar year doses are compared to their respective limits once per 31 days to determine compliance.

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7.5 DOSE PROJECTIONS

In accordance with ODCM Surveillance Requirement 2.2.2.4, dose projections will be performed. This will be done by maintaining running 31-day totals for the gamma dose, the beta dose and the maximum organ dose. Once per 31 days, these 31-day running totals will be compared to the limits given in ODCM Control 1.2.2.4 to determine compliance.

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If the projected doses exceed any of these limits, the GASEOUS RADWASTE TREATMENT SYSTEM and the VENTILATION EXHAUST TREATMENT SYSTEM shall be used to reduce radioactive materials in gaseous effluents to areas at or beyond the SITE BOUNDARY.

7.5.1 GASEOUS RADWASTE TREATMENT SYSTEM DESCRIPTION

The GASEOUS RADWASTE TREATMENT SYSTEM (GRTS) described below shall be maintained and operated to keep releases ALARA.

A flow diagram for the GRTS is given in Figure 7.3. The system consists of two waste-gas compressor packages, nine gas decay tanks, and the associated piping, valves, and instrumentation. Gaseous wastes are received from the following: degassing of the reactor coolant and purging of the volume control tank prior to a cold shutdown, displacing of cover gases caused by liquid accumulation in the tanks connected to the vent header, and boron recycle process operation.

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7.6 QUARTERLY DOSE CALCULATIONS

A complete dose analysis utilizing the total estimated gaseous releases for each calendar quarter will be performed and reported as required in ODCM Administrative Control 5.2. Methodology for this analysis is that which is described in this section using the quarterly release values reported by the plant personnel. All real pathways and receptor locations identified by the most recent land use survey are considered. In addition, actual meteorological data representative of a ground level release for each corresponding calendar quarter will be used. For iodine releases, it is assumed that half the iodine released is in organic form. Organic iodine causes a dose only by inhalation. For cow-milk and beef ingestion doses, the fraction of the time the animals are on stored feed (identified in the survey) is used in the calculation.

The highest organ dose for a real receptor is determined by summing the dose contribution from all identified pathways for each receptor including ground contamination, inhalation, vegetable ingestion (for identified garden locations), cow and/or goat milk ingestion (if a cow or goat is identified for the location), beef ingestion (the beef ingestion dose for the location of highest beef dose for all receptors will be considered the beef dose for all receptors).

7.6.1 NOBLE GAS - GAMMA AIR DOSE

Gamma air doses due to exposure to noble gases, D_{γ} in mrem, are calculated using the following equation:

$$D_{\gamma} = X_{im} DF_{\gamma i} \quad (7.9)$$

where:

X_{im} = concentration of nuclide i at location m , $\mu\text{Ci}/\text{m}^3$. Air concentrations are calculated as described by Equation 7.14.
 $DF_{\gamma i}$ = dose conversion factor for external gamma for nuclide i , mrad/year per $\mu\text{Ci}/\text{m}^3$ (Table 7.3).

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7.6.2 NOBLE GAS - BETA AIR DOSE

Beta air doses due to exposure to noble gases, D_{β} in mrem, are calculated using the following equation:

$$D_{\beta} = X_{im} DF_{\beta i} \quad (7.10)$$

where:

X_{im} = concentration of nuclide i at location m , $\mu\text{Ci}/\text{m}^3$. Air concentrations are calculated as described by Equation 7.14.
 $DF_{\beta i}$ = dose conversion factor for external beta for nuclide i , mrad/year per $\mu\text{Ci}/\text{m}^3$ (Table 7.3).

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7.6.3 RADIOIODINE, PARTICULATE AND TRITIUM - MAXIMUM ORGAN DOSE

Organ doses due to radioiodine, particulate and tritium releases, D_{org} in mrem, are calculated using the following equation:

$$D_{org} = 3.17E-08 \left(\frac{X}{Q} \right) \sum_P R_{PT} Q_T \sum_i \left[\left(\frac{D}{Q} \right) \sum_P R_{Pi} + \left(\frac{D}{Q} \right) R_{Gi} + \left(\frac{X}{Q} \right) R_{Ii} \right] Q_i \quad (7.11)$$

where:

- 3.17E-08 = conversion factor, year/second.
- X/Q = Relative concentration for location under consideration, s/m^3 . Relative concentrations are calculated as described by Equation 7.15.
- R_{PT} = ingestion dose factor for pathway P for tritium, m^2 -mrem/year per $\mu Ci/s$. Ingestion pathways available for consideration are the same as those listed above for R_{Pi} . Equations for calculating ingestion dose factors for tritium are given in Sections 7.7.7 through 7.7.12.
- Q_T = adjusted release rate for tritium for location under consideration, $\mu Ci/s$. Calculated in the same manner as Q_i above.
- R_{Pi} = ingestion dose factor for pathway P for each identified nuclide i (except tritium), m^2 -mrem/year per $\mu Ci/s$. Ingestion pathways available for consideration include:
 pasture grass-cow-milk ingestion
 stored feed-cow-milk ingestion
 pasture grass-goat-milk ingestion
 stored feed-goat-milk ingestion
 pasture grass-beef ingestion
 stored feed-beef ingestion
 fresh leafy vegetable ingestion
 stored vegetable ingestion
 Equations for calculating these ingestion dose factors are given in Sections 7.7.1 through 7.7.6.
- D/Q = Relative deposition for location under consideration, m^{-2} . Relative deposition is calculated as described in Equation 7.16.
- R_{Gi} = Dose factor for standing on contaminated ground, m^2 -mrem/year per $\mu Ci/s$. The equation for calculating the ground plane dose factor is given in Section 7.7.14.
- R_{Ii} = Inhalation dose factor, mrem/year per $\mu Ci/m^3$. The equation for calculating the inhalation dose factor is given in Section 7.7.13.

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Q_i = adjusted release rate for nuclide i for location under consideration, $\mu\text{Ci/s}$. The initial release rate is adjusted to account for decay between the release point and the location, depending on the frequency of wind speeds applicable to that sector. Hence, the adjusted release rate is equal to the actual release rate decayed for an average travel time during the period.

$$Q_{i0} = \sum_{j=1}^g f_j \exp(-\lambda_i x/u_j) \quad (7.12)$$

where

- Q_{i0} = initial average release rate for nuclide i over the period, $\mu\text{Ci/s}$.
- f_j = joint relative frequency of occurrence of winds in windspeed class j blowing toward this exposure point, expressed as a fraction.
- λ_i = radiological decay constant for nuclide i , s^{-1} .
- x = downwind distance, meters.
- u_j = midpoint value of wind speed class interval j , m/s .

7.6.4 POPULATION DOSES

For determining population doses to the 50-mile population around the plant, each compass sector is broken down into elements. These elements are defined in Table 7.4. For each of these sector elements, an average dose is calculated, and then multiplied by the population in that sector element. Dispersion factors are calculated for the midpoint of each sector element (see Table 7.4). R27

For population doses resulting from ingestion, it is conservatively assumed that all food eaten by the average individual is grown locally. R27

The general equation used for calculating the population dose in a given sector element is:

$$\text{Dose}_{\text{pop}} = \sum_P \text{RATIO}_P * \text{POP}_N * \text{AGE} * 0.001 * \text{DOSE}_P \quad (7.13)$$

where

- RATIO_P = ratio of average to maximum dose for pathway P . (Average ingestion rates are obtained from Regulatory Guide 1.109, Table E-4.)
- = 0.5 for submersion and ground exposure pathways, a shielding/occupancy factor.

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- = 1.0 for the inhalation pathway.
- = 0.515, 0.515, 0.5, and 0.355 for milk, for infant, child, teen and adult, respectively. (It is assumed that the ratio of average to maximum infant milk ingestion rates is the same as that for child.)
- = 1.0, 0.90, 0.91, 0.86 for beef ingestion, for infant, child, teen and adult, respectively.
- = 1.0, 0.38, 0.38, 0.37 for vegetable ingestion, for infant, child, teen and adult, respectively. (It is assumed that the average individual eats no fresh leafy vegetables, only stored vegetables.)
- POP_N = the population of the sector element, persons (Table 7.5). R27
- AGE = fraction of the population belonging to each age group.
- = 0.015, 0.168, 0.153, 0.665 for infant, child, teen and adult, respectively (fractions taken from NUREG/CR-1004, Table 3.39).
- 0.001 = conversion from mrem to rem.
- DOSE_P = the dose for pathway P to the maximum individual at the location under consideration, mrem. For ingestion pathways, this dose is multiplied by an average decay correction to account for decay as the food is moved through the food distribution cycle. This average decay correction, ADC, is defined as follows:

For milk and vegetables, $ADC = \exp(-\lambda_i t)$

where

- λ_i = decay constant for nuclide i, seconds.
- t = distribution time for food product under consideration (values from Regulatory Guide 1.109, Table D-1).
 = 1.21E+06 seconds (14 days) for vegetables.
 = 3.46E+05 seconds (4 days) for milk.

For meat, $ADC = \frac{\exp(-\lambda_i t) \lambda_i t_{cb}}{1 - \exp(-\lambda_i t_{cb})}$

where

- λ_i = decay constant for nuclide i, seconds.
- t = additional distribution time for meat, over and above the time for slaughter to consumption described in Section 7.7.3, 7 days.
- t_{cb} = time to consume a whole beef, as described in Section 7.7.3.

For beef ingestion, the additional factors in the calculation of ADC negate the integration of the dose term over the period during which a whole beef is consumed, for the calculation of population dose. In other words, this assumes that the maximum individual freezes and eats a whole beef, while the average individual buys smaller portions at a time.

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Population doses are summed over all sector elements to obtain a total population dose for the 50-mile population.

7.6.5 REPORTING OF DOSES

The calculated quarterly doses and calculated population doses described in this section are reported in the Annual Effluent Release Report as required by ODCM Administrative Control 5.2.

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7.7 GASEOUS RELEASES - Dose Factors

7.7.1 PASTURE GRASS-COW/GOAT-MILK INGESTION DOSE FACTORS - RC_{Pi}
 (m²-mrem/year per microcuries/second)

$$RC_{Pi} = 10^6 DFL_{iao} U_{ap} F_{mi} Q_f \exp(-\lambda_i t_{fm}) f_p \left\{ \frac{r(1-\exp(-\lambda_E t_{ep}))}{Y_p \lambda_E} + \frac{B_{iv}(1-\exp(-\lambda_i t_b))}{P \lambda_i} \right\}$$

where:

- 10⁶ = conversion factor, picocurie/microcurie.
- DFL_{iao} = ingestion dose conversion factor for nuclide i, age group a, organ o, mrem/picocurie (Table 6.4).
- U_{ap} = milk ingestion rate for age group a, liters/year.
- F_{mi} = transfer factor for nuclide i from animal's feed to milk, days/liter (Table 6.2).
- Q_f = animal's consumption rate, kg/day.
- λ_i = decay constant for nuclide i, seconds⁻¹ (Table 6.2).
- t_{fm} = transport time from milking to receptor, seconds.
- f_p = fraction of time animal spends on pasture, dimensionless.
- r = fraction of activity retained on pasture grass, dimensionless.
- λ_E = the effective decay constant, due to radioactive decay and weathering, seconds⁻¹, equal to λ_i + λ_w.
- λ_w = weathering decay constant for leaf and plant surfaces, seconds⁻¹.
- t_{ep} = time pasture is exposed to deposition, seconds.
- Y_p = agricultural productivity by unit area of pasture grass, kg/m².
- B_{iv} = transfer factor for nuclide i from soil to vegetation, picocuries/kg (wet weight of vegetation) per picocuries/kg (dry soil).
- t_b = time period over which accumulation on the ground is evaluated, seconds.
- P = effective surface density of soil, kg/m².

NOTE: Factors defined above which do not reference a table for their numerical values are given in Table 6.3.

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7.7.2 STORED FEED-COW/GOAT-MILK INGESTION DOSE FACTORS - R_{CSi}
 (m^2 -mrem/year per microcuries/second)

$$R_{CSi} = 10^6 \text{ DFL}_{iao} U_{ap} F_{mi} Q_f f_s \exp(-\lambda_i t_{fm}) \frac{(1 - \exp(-\lambda_i t_{csf}))}{t_{csf} \lambda_i} \left\{ \frac{r(1 - \exp(-\lambda_E t_{esf}))}{Y_{sf} \lambda_E} + \frac{B_{iv}(1 - \exp(-\lambda_i t_b))}{P \lambda_i} \right\}$$

where:

- 10^6 = conversion factor, picocurie/microcurie.
- DFL_{iao} = ingestion dose conversion factor for nuclide i , age group a , organ o , mrem/picocurie (Table 6.4).
- U_{ap} = milk ingestion rate for age group a , liters/year.
- F_{mi} = transfer factor for nuclide i from animal's feed to milk, days/liter (Table 6.2).
- Q_f = animal's consumption rate, kg/day.
- f_s = fraction of time animal spends on stored feed, dimensionless.
- λ_i = decay constant for nuclide i , seconds $^{-1}$ (Table 6.2).
- t_{fm} = transport time from milking to receptor, seconds.
- t_{csf} = time between harvest of stored feed and consumption by animal, seconds.
- r = fraction of activity retained on pasture grass, dimensionless.
- λ_E = the effective decay constant, due to radioactive decay and weathering, seconds $^{-1}$, equal to $\lambda_i + \lambda_w$.
- λ_w = weathering decay constant for leaf and plant surfaces, seconds $^{-1}$.
- t_{esf} = time stored feed is exposed to deposition, seconds.
- Y_{sf} = agricultural productivity by unit area of stored feed, kg/m 2 .
- B_{iv} = transfer factor for nuclide i from soil to vegetation, picocuries/kg (wet weight of vegetation) per picocuries/kg (dry soil).
- t_b = time period over which accumulation on the ground is evaluated, seconds.
- P = effective surface density of soil, kg/m 2 .

NOTE: Factors defined above which do not reference a table for their numerical values are given in Table 6.3.

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7.7.3 PASTURE GRASS-BEEF INGESTION DOSE FACTORS - R_{MPi}
 (m^2 -mrem/year per microcuries/second)

$$R_{MPi} = 10^6 \text{ DFL}_{iao} U_{am} F_{fi} Q_f \frac{(1 - \exp(-\lambda_i t_{cb})) \exp(-\lambda_i t_s)}{\lambda_i t_{cb}} \left\{ \frac{r(1 - \exp(-\lambda_E t_{ep}))}{Y_P \lambda_E} + \frac{B_{iv}(1 - \exp(-\lambda_i t_b))}{P \lambda_i} \right\}$$

where:

- 10^6 = conversion factor, picocurie/microcurie.
- DFL_{iao} = ingestion dose conversion factor for nuclide i, age group a, organ o, mrem/picocurie (Table 6.4).
- U_{am} = meat ingestion rate for age group a, kg/year.
- F_{fi} = transfer factor for nuclide i from cow's feed to meat, days/kg (Table 6.2).
- Q_f = cow's consumption rate, kg/day.
- λ_i = decay constant for nuclide i, seconds⁻¹ (Table 6.2).
- t_{cb} = time for receptor to consume a whole beef, seconds.
- t_s = transport time from slaughter to consumer, seconds.
- f_p = fraction of time cow spends on pasture, dimensionless.
- r = fraction of activity retained on pasture grass, dimensionless.
- λ_E = the effective decay constant, due to radioactive decay and weathering, seconds⁻¹, equal to $\lambda_i + \lambda_w$.
- λ_w = weathering decay constant for leaf and plant surfaces, seconds⁻¹.
- t_{ep} = time pasture is exposed to deposition, seconds.
- Y_P = agricultural productivity by unit area of pasture grass, kg/m².
- B_{iv} = transfer factor for nuclide i from soil to vegetation, picocuries/kg (wet weight of vegetation) per picocuries/kg (dry soil).
- t_b = time over which accumulation on the ground is evaluated, seconds.
- P = effective surface density of soil, kg/m².

NOTE: Factors defined above which do not reference a table for their numerical values are given in Table 6.3.

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7.7.4 STORED FEED-BEEF INGESTION DOSE FACTORS - R_{MSi}
 (m^2 -mrem/year per microcuries/second)

$$R_{MSi} = 10^6 \text{ DFL}_{iao} U_{am} F_{fi} Q_f \frac{(1 - \exp(-\lambda_i t_{cb})) \exp(-\lambda_i t_s)}{\lambda_i t_{cb}} f_s \frac{(1 - \exp(-\lambda_i t_{csf}))}{\lambda_i t_{csf}} \left\{ \frac{r(1 - \exp(-\lambda_E t_{esf}))}{Y_{sf} \lambda_E} + \frac{B_{iv}(1 - \exp(-\lambda_i t_b))}{P \lambda_i} \right\}$$

where:

- 10^6 = conversion factor, picocurie/microcurie.
- DFL_{iao} = ingestion dose conversion factor for nuclide i, age group a, organ o, mrem/picocurie (Table 6.4).
- U_{am} = meat ingestion rate for age group a, kg/year.
- F_{fi} = transfer factor for nuclide i from cow's feed to meat, days/kg (Table 6.2).
- Q_f = cow's consumption rate, kg/day.
- λ_i = decay constant for nuclide i, seconds⁻¹ (Table 6.2).
- t_{cb} = time for receptor to consume a whole beef, seconds.
- t_s = transport time from slaughter to consumer, seconds.
- f_s = fraction of time cow spends on stored feed, dimensionless.
- t_{csf} = time between harvest of stored feed and consumption by cow, seconds.
- r = fraction of activity retained on pasture grass, dimensionless.
- t_{esf} = time stored feed is exposed to deposition, seconds.
- Y_{sf} = agricultural productivity by unit area of stored feed, kg/m².
- λ_E = the effective decay constant, due to radioactive decay and weathering, seconds⁻¹, equal to $\lambda_i + \lambda_w$.
- λ_w = weathering decay constant for leaf and plant surfaces, seconds⁻¹.
- B_{iv} = transfer factor for nuclide i from soil to vegetation, picocuries/kg (wet weight of vegetation) per picocuries/kg (dry soil).
- t_b = time over which accumulation on the ground is evaluated, seconds.
- P = effective surface density of soil, kg/m².

NOTE: Factors defined above which do not reference a table for their numerical values are given in Table 6.3.

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7.7.5 FRESH LEAFY VEGETABLE INGESTION DOSE FACTORS - R_{VFi}
 (m^2 -mrem/year per microcuries/second)

$$R_{VFi} = 10^6 \text{ DFL}_{iao} e(-\lambda_i t_{hc}) U_{FLa} f_L \left\{ \frac{r(1-e(-\lambda_E t_e))}{Y_f \lambda_E} + \frac{B_{iv}(1-e(-\lambda_i t_b))}{P \lambda_i} \right\}$$

where:

- 10^6 = conversion factor, picocurie/microcurie.
- DFL_{iao} = ingestion dose conversion factor for nuclide i, age group a, organ o, mrem/picocurie (Table 6.4).
- λ_i = decay constant for nuclide i, seconds⁻¹ (Table 6.2).
- t_{hc} = average time between harvest of vegetables and their consumption and/or storage, seconds.
- U_{FLa} = consumption rate of fresh leafy vegetables by the receptor in age group a, kg/year.
- f_L = fraction of fresh leafy vegetables grown locally, dimensionless.
- r = fraction of deposited activity retained on vegetables, dimensionless.
- λ_E = the effective decay constant, due to radioactive decay and weathering, seconds⁻¹.
 $= \lambda_i + \lambda_w$
- λ_w = decay constant for removal of activity on leaf and plant surfaces by weathering, seconds⁻¹.
- t_e = exposure time in garden for fresh leafy and/or stored vegetables, seconds.
- Y_f = agricultural yield for fresh leafy vegetables, kg/m².
- B_{iv} = transfer factor for nuclide i from soil to vegetables, picocuries/kg (wet weight of vegetation) per picocuries/kg (dry soil).
- t_b = time period over which accumulation on the ground is evaluated, seconds.
- P = effective surface density of soil, kg/m².

NOTE: Factors defined above which do not reference a table for their numerical values are given in Table 6.3.

OFFSITE DOSE CALCULATION/ENVIRONMENTAL MONITORING METHODOLOGIES

7.7.6 STORED VEGETABLE INGESTION DOSE FACTORS - R_{VSi}
 m²-mrem/year per microcuries/second)

$$R_{VSi} = 10^6 \text{ DFL}_{iao} \exp(-\lambda_i t_{hc}) U_{Sa} f_g \frac{(1-e(-\lambda_i t_{sv}))}{\lambda_i t_{sv}} \left\{ \frac{r(1-e(-\lambda_E t_e))}{Y_{sv} \lambda_E} + \frac{B_{iv}(1-e(-\lambda_i t_b))}{P \lambda_i} \right\}$$

where:

- 10⁶ = conversion factor, picocurie/microcurie.
- DFL_{iao} = ingestion dose conversion factor for nuclide i, age group a, organ o, mrem/picocurie (Table 6.4).
- λ_i = decay constant for nuclide i, seconds⁻¹ (Table 6.2).
- t_{hc} = average time between harvest of vegetables and their consumption and/or storage, seconds.
- U_{Sa} = consumption rate of stored vegetables by the receptor in age group a, kg/year.
- f_g = fraction of stored vegetables grown locally, dimensionless.
- t_{sv} = time between storage of vegetables and their consumption, seconds.
- r = fraction of deposited activity retained on vegetables, dimensionless.
- λ_E = the effective decay constant, due to radioactive decay and weathering, seconds⁻¹.
 = $\lambda_i + \lambda_w$
- λ_w = decay constant for removal of activity on leaf and plant surfaces by weathering, seconds⁻¹.
- t_e = exposure time in garden for fresh leafy and/or stored vegetables, seconds.
- Y_{sv} = agricultural yield for stored vegetables, kg/m².
- B_{iv} = transfer factor for nuclide i from soil to vegetables, picocuries/kg (wet weight of vegetation) per picocuries/kg (dry soil).
- t_b = time period over which accumulation on the ground is evaluated, seconds.
- P = effective surface density of soil, kg/m².

NOTE: Factors defined above which do not reference a table for their numerical values are given in Table 6.3.

OFFSITE DOSE CALCULATION/ENVIRONMENTAL MONITORING METHODOLOGIES

7.7.7 TRITIUM-PASTURE GRASS-COW/GOAT-MILK DOSE FACTOR - R_{CTP}
(mrem/year per microcuries/ m^3)

$$R_{CTP} = 10^3 10^6 DFL_{Tao} F_{mT} Q_f U_{ap} [0.75(0.5/H)] f_p \exp(-\lambda_T t_{fm})$$

where:

10^3	= conversion factor, grams/kg.
10^6	= conversion factor, picocuries/microcuries.
DFL_{Tao}	= ingestion dose conversion factor for tritium for age group a, organ o, mrem/picocurie (Table 6.4).
F_{mT}	= transfer factor for tritium from animal's feed to milk, days/liter (Table 6.2).
Q_f	= animal's consumption rate, kg/day.
U_{ap}	= milk ingestion rate for age group a, liters/year.
0.75	= the fraction of total feed that is water.
0.5	= the ratio of the specific activity of the feed grass water to the atmospheric water.
H	= absolute humidity of the atmosphere, g/m^3 .
f_p	= fraction of time animal spends on pasture, dimensionless.
λ_T	= decay constant for tritium, $seconds^{-1}$ (Table 6.2).
t_{fm}	= transport time from milking to receptor, seconds.

NOTE: Factors defined above which do not reference a table for their numerical values are given in Table 6.3.

OFFSITE DOSE CALCULATION/ENVIRONMENTAL MONITORING METHODOLOGIES

7.7.8 TRITIUM-STORED FEED-COW/GOAT-MILK DOSE FACTOR - R_{CTS}
 (mrem/year per microcuries/m³)

$$R_{MTS} = 10^3 \cdot 10^6 \cdot DFL_{Tao} \cdot F_{ft} \cdot Q_f \cdot U_{am} \cdot [0.75(0.5/H)] \cdot f_s \cdot \frac{\exp(-\lambda_T t_s)}{(1-\exp(-\lambda_T t_{ep}))} \cdot \frac{\exp(-\lambda_T t_{cb})}{(1-\exp(-\lambda_T t_{cb}))}$$

where:

R25

- 10³ = conversion factor, grams/kg.
- 10⁶ = conversion factor, picocuries/microcuries.
- DFL_{Tao} = ingestion dose conversion factor for H-3 for age group a, organ o, mrem/picocurie (Table 6.4).
- F_{ft} = transfer factor for H-3 from cow's feed to meat, days/kg (Table 6.2).
- Q_f = cow's consumption rate, kg/day.
- U_{am} = meat ingestion rate for age group a, kg/year.
- 0.75 = the fraction of total feed that is water.
- 0.5 = the ratio of the specific activity of the feed grass water to the atmospheric water.
- H = absolute humidity of the atmosphere, g/m³.
- f_s = fraction of time cow spends on stored feed, dimensionless.
- λ_T = decay constant for tritium, seconds⁻¹ (Table 6.2).
- t_s = transport time from slaughter to consumer, seconds.
- t_{csf} = time to consume stored feed, seconds.
- t_{cb} = time for receptor to consume a whole beef, seconds.

R25

NOTE: Factors defined above which do not reference a table for their numerical values are given in Table 6.3.

OFFSITE DOSE CALCULATION/ENVIRONMENTAL MONITORING METHODOLOGIES

7.7.9 TRITIUM-PASTURE GRASS-BEEF DOSE FACTOR - R_{MT}
 (mrem/year per microcuries/m³)

$$R_{MTP} = 10^3 10^6 DFL_{Tao} F_{fT} Q_f U_{am} [0.75(0.5/H)] f_p \exp(-\lambda_T t_s) \frac{(1-\exp(-\lambda_T t_{ep}))}{\lambda_T t_{ep}} \frac{(1-\exp(-\lambda_T t_{cb}))}{\lambda_T t_{cb}}$$

where:

- 10³ = conversion factor, grams/kg.
- 10⁶ = conversion factor, picocuries/microcuries.
- DFL_{Tao} = ingestion dose conversion factor for H-3 for age group a, organ o, mrem/picocurie (Table 6.4).
- F_{fT} = transfer factor for H-3 from cow's feed to meat, days/kg (Table 6.2).
- Q_f = cow's consumption rate, kg/day.
- U_{am} = meat ingestion rate for age group a, kg/year.
- 0.75 = the fraction of total feed that is water.
- 0.5 = the ratio of the specific activity of the feed grass water to the atmospheric water.
- H = absolute humidity of the atmosphere, g/m³.
- f_p = fraction of time cow spends on pasture, dimensionless.
- λ_T = decay constant for tritium, seconds⁻¹ (Table 6.2).
- t_s = transport time from slaughter to consumer, seconds.
- t_{ep} = time pasture is exposed to deposition, seconds.
- t_{cb} = time for receptor to consume a whole beef, seconds.

NOTE: Factors defined above which do not reference a table for their numerical values are given in Table 6.3.

OFFSITE DOSE CALCULATION/ENVIRONMENTAL MONITORING METHODOLOGIES

7.7.10 TRITIUM-STORED FEED-BEEF DOSE FACTOR - R_{MTS}
 (mrem/year per microcuries/m³)

$$R_{MTS} = 10^3 \cdot 10^6 \cdot DFL_{Tao} \cdot F_{fT} \cdot Q_f \cdot U_{am} \cdot [0.75(0.5/H)] \cdot f_s \cdot \exp(-\lambda_T t_s) \cdot \frac{(1 - \exp(-\lambda_T t_{ep}))}{\lambda_T t_{csf}} \cdot \frac{(1 - \exp(-\lambda_T t_{cb}))}{\lambda_T t_{cb}}$$

where:

- 10^3 = conversion factor, grams/kg.
- 10^6 = conversion factor, picocuries/microcuries.
- DFL_{Tao} = ingestion dose conversion factor for H-3 for age group a, organ o, mrem/picocurie (Table 6.4).
- F_{fT} = transfer factor for H-3 from cow's feed to meat, days/kg (Table 6.2).
- Q_f = cow's consumption rate, kg/day.
- U_{am} = meat ingestion rate for age group a, kg/year.
- 0.75 = the fraction of total feed that is water.
- 0.5 = the ratio of the specific activity of the feed grass water to the atmospheric water.
- H = absolute humidity of the atmosphere, g/m³.
- f_s = fraction of time cow spends on stored feed, dimensionless.
- λ_T = decay constant for tritium, seconds⁻¹ (Table 6.2).
- t_s = transport time from slaughter to consumer, seconds.
- t_{csf} = time to consume stored feed, seconds.
- t_{cb} = time for receptor to consume a whole beef, seconds.

R25

R25

NOTE: Factors defined above which do not reference a table for their numerical values are given in Table 6.3.

OFFSITE DOSE CALCULATION/ENVIRONMENTAL MONITORING METHODOLOGIES

7.7.11 TRITIUM-FRESH LEAFY VEGETABLES DOSE FACTOR - R_{VTF}
(mrem/year per microcuries/ m^3)

$$R_{VTF} = 10^3 10^6 DFL_{Tao} [0.75(0.5/H)] U_{FLa} f_L \exp(-\lambda_T t_{hc})$$

where:

- 10^3 = conversion factor, grams/kg.
- 10^6 = conversion factor, picocuries/microcuries.
- DFL_{Tao} = ingestion dose conversion factor for tritium for age group a, organ o, mrem/picocurie (Table 6.4).
- 0.75 = the fraction of total vegetation that is water.
- 0.5 = the ratio of the specific activity of the vegetables water to the atmospheric water.
- H = absolute humidity of the atmosphere, g/m^3 .
- U_{FLa} = consumption rate of fresh leafy vegetables by the receptor in age group a, kg/year.
- f_L = fraction of fresh leafy vegetables grown locally, dimensionless.
- λ_T = decay constant for tritium, $seconds^{-1}$ (Table 6.2).
- t_{hc} = time between harvest of vegetables and their consumption and/or storage, seconds.

NOTE: Factors defined above which do not reference a table for their numerical values are given in Table 6.3.

OFFSITE DOSE CALCULATION/ENVIRONMENTAL MONITORING METHODOLOGIES

7.7.12 TRITIUM-STORED VEGETABLES DOSE FACTOR - R_{VTS}
(mrem/year per microcuries/m³)

$$R_{VTS} = 10^3 10^6 DFL_{Tao} [0.75(0.5/H)] U_{Sa} f_g \frac{(1 - \exp(-\lambda_T t_{sv}))}{\lambda_T t_{sv}} \exp(-\lambda_T t_{hc})$$

where:

- 10³ = conversion factor, grams/kg.
- 10⁶ = conversion factor, picocuries/microcuries.
- DFL_{Tao} = ingestion dose conversion factor for tritium for age group a, organ o, mrem/picocurie (Table 6.4).
- 0.75 = the fraction of total vegetation that is water.
- 0.5 = the ratio of the specific activity of the vegetation water to the atmospheric water.
- H = absolute humidity of the atmosphere, g/m³.
- U_{Sa} = consumption rate of stored vegetables by the receptor in age group a, kg/year.
- f_g = fraction of stored vegetables grown locally, dimensionless.
- λ_T = decay constant for tritium, seconds⁻¹ (Table 6.2).
- t_{sv} = time between harvest of stored vegetables and their consumption and/or storage, seconds.
- t_{hc} = time between harvest of vegetables and their storage, seconds.

NOTE: Factors defined above which do not reference a table for their numerical values are given in Table 6.3.

OFFSITE DOSE CALCULATION/ENVIRONMENTAL MONITORING METHODOLOGIES

7.7.13 INHALATION DOSE FACTORS- R_{Ii}
(mrem/year per microcuries/ m^3)

$$R_{Ii} = DFA_{iao} BR_a 10^6$$

where:

DFA_{iao} = inhalation dose conversion factor for nuclide i,
age group a and organ o, mrem/picocurie (Table 7.7).
 BR_a = breathing rate for age group a, m^3 /year (Table 6.3).
 10^6 = conversion factor, picocurie/microcurie.

7.7.14 GROUND PLANE DOSE FACTORS - R_{Gi}
(m^2 -mrem/year per microcuries/second)

$$R_{Gi} = DFG_{io} 1/\lambda_i 10^6 8760 [1 - \exp(-\lambda_i t_b)]$$

where:

DFG_{io} = dose conversion factor for standing on contaminated ground
for nuclide i and organ o (total body and skin), mrem/hr per
picocurie/ m^2 (Table 6.6).
 λ_i = decay constant of nuclide i, seconds⁻¹ (Table 6.2).
 10^6 = conversion factor, picocurie/microcurie.
8760 = conversion factor, hours/year.
 t_b = time period over which the ground accumulation is evaluated,
seconds (Table 6.3).

OFFSITE DOSE CALCULATION/ENVIRONMENTAL MONITORING METHODOLOGIES

7.8 DISPERSION METHODOLOGY

Dispersion factors are calculated for radioactive effluent releases using hourly average meteorological data consisting of wind speed and direction measurements at 10m and temperature measurements at 9m and 46m.

A sector-average dispersion equation consistent with Regulatory Guide 1.111 is used. The dispersion model considers plume depletion (using information from Figure 7.4), and building wake effects. Terrain effects on dispersion are not considered.

Hourly average meteorological data are expressed as a joint-frequency distribution of wind speed, wind direction, and atmospheric stability. The joint-frequency distribution which represents the historical meteorological data for the period January 1972 to December 1975 is given in Table 7.2.

The wind speed classes that are used are as follows:

<u>Number</u>	<u>Range (m/s)</u>	<u>Midpoint (m/s)</u>
1	<0.3	0.13
2	0.3-0.6	0.45
3	0.7-1.5	1.10
4	1.6-2.4	1.99
5	2.5-3.3	2.88
6	3.4-5.5	4.45
7	5.6-8.2	6.91
8	8.3-10.9	9.59
9	>10.9	10.95

The stability classes that will be used are the standard A through G classifications. The stability classes 1-7 will correspond to A=1, B=2, ..., G=7.

7.8.1 AIR CONCENTRATION - X ($\mu\text{Ci}/\text{m}^3$)

Air concentrations of nuclides at downwind locations are calculated using the following equation:

$$X_i = \sum_{j=1}^9 \sum_{k=1}^7 (2/\pi)^{1/2} \frac{f_{jk} Q_i P}{\sum_{zk} u_j (2\pi x/n)} \exp(-\lambda_i x/u_j) \quad (7.14)$$

where

f_{jk} = joint relative frequency of occurrence of winds in windspeed class j, stability class k, blowing toward this exposure point, expressed as a fraction.

OFFSITE DOSE CALCULATION/ENVIRONMENTAL MONITORING METHODOLOGIES

- Q_i = average annual release rate of radionuclide i , $\mu\text{Ci/s}$.
 p = fraction of radionuclide remaining in plume (Figure 7.4).
 Σ_{zk} = vertical dispersion coefficient for stability class k which includes a building wake adjustment,
 = $(\sigma_{zk}^2 + cA/\pi)^{1/2}$,
 or = $\sqrt{3} \sigma_{zk}$, whichever is smaller.
 where
 σ_{zk} is the vertical dispersion coefficient for stability class k (m) (Figure 7.5),
 c is a building shape factor ($c=0.5$),
 A is the minimum building cross-sectional area (1800 m^2).
 u_j = midpoint value of wind speed class interval j , m/s .
 x = downwind distance, m .
 n = number of sectors, 16.
 λ_i = radioactive decay coefficient of radionuclide i , s^{-1}
 $2\pi x/n$ = sector width at point of interest, m .

7.8.2 RELATIVE CONCENTRATION - X/Q (sec/m^3)

Relative concentrations of nuclides at downwind locations are calculated using the following equation:

$$X/Q = \sum_{j=1}^9 \sum_{k=1}^7 (2/\pi)^{1/2} \frac{f_{jk}}{\Sigma_{zk} u_j (2\pi x/n)} \quad (7.15)$$

where

- f_{jk} = joint relative frequency of occurrence of winds in windspeed class j , stability class k , blowing toward this exposure point, expressed as a fraction.
 Σ_{zk} = vertical dispersion coefficient for stability class k which includes a building wake adjustment,
 = $(\sigma_{zk}^2 + cA/\pi)^{1/2}$,
 or = $\sqrt{3} \sigma_{zk}$, whichever is smaller.
 where
 σ_{zk} is the vertical dispersion coefficient for stability class k (m) (Figure 7.5),
 c is a building shape factor ($c=0.5$),
 A is the minimum building cross-sectional area (1800 m^2).
 u_j = midpoint value of wind speed class interval j , m/s .
 x = downwind distance, m .
 n = number of sectors, 16.
 $2\pi x/n$ = sector width at point of interest, m .

OFFSITE DOSE CALCULATION/ENVIRONMENTAL MONITORING METHODOLOGIES

7.8.3 RELATIVE DEPOSITION- D/Q (m⁻²)

Relative deposition of nuclides at downwind locations is calculated using the following equation:

$$D/Q = \sum_{j=1}^9 \sum_{k=1}^7 \frac{f_{jk} DR}{(2\pi x/n)} \quad (7.16)$$

where

- f_k = joint relative frequency of occurrence of winds in windspeed class j and stability class k , blowing toward this exposure point, expressed as a fraction.
- DR = relative deposition rate, m⁻¹ (from Figure 7.6).
- x = downwind distance, m.
- n = number of sectors, 16.
- $2\pi x/n$ = sector width at point of interest, m.

OFFSITE DOSE CALCULATION/ENVIRONMENTAL MONITORING METHODOLOGIES

Table 7.1
 SQN - OFFSITE RECEPTOR LOCATION DATA

POINT		DISTANCE from plant (m)	X/Q (s/m ²)	D/Q (1/m ²)
Site Boundary	N	950	5.12E-06	1.29E-08
Site Boundary	NNE	2260	1.93E-06	5.28E-09
Site Boundary	NE	1910	2.32E-06	6.33E-09
Site Boundary	ENE	1680	1.12E-06	2.64E-09
Site Boundary	E	1570	7.10E-07	1.46E-09
Site Boundary	ESE	1460	7.91E-07	1.58E-09
Site Boundary	SE	1460	9.14E-07	2.41E-09
Site Boundary	SSE	1550	1.34E-06	3.23E-09
Site Boundary	S	1570	2.37E-06	4.18E-09
Site Boundary	SSW	1840	4.51E-06	9.26E-09
Site Boundary	SW	2470	1.38E-06	2.63E-09
Site Boundary	WSW	910	2.93E-06	3.86E-09
Site Boundary	W	670	3.63E-06	3.74E-09
Site Boundary	WNW	660	2.49E-06	2.44E-09
Site Boundary	NW	660	2.85E-06	3.67E-09
Site Boundary	NNW	730	3.95E-06	6.59E-09
Liquid Discharge	S	870	N/A	N/A

NOTE: For quarterly airborne dose calculations, doses will also be calculated for all locations identified in the most recent land use census, and for any additional points deemed necessary.

OFFSITE DOSE CALCULATION/ENVIRONMENTAL MONITORING METHODOLOGIES

Table 7.2 (1 of 7)
 JOINT PERCENTAGE FREQUENCIES OF WIND DIRECTION AND WIND SPEED
 FOR DIFFERENT STABILITY CLASSES*

Sequoyah Nuclear Plant Meteorological Facility*
 Jan. 1, 72 - Dec. 31, 75

Stability Class A
 Delta T_L-1.9 deg. C/100m

	Calm	Wind Speed (mph)								Total
		0.6- 1.4	1.5- 3.4	3.5- 5.4	5.5- 7.4	7.5- 12.4	12.5- 18.4	18.5- 24.4	>24.5	
N	0.01	0.01	0.01	0.03	0.04	0.04	0.0	0.0	0.0	0.13
NNE	0.0	0.0	0.04	0.19	0.20	0.16	0.01	0.0	0.0	0.60
NE	0.0	0.0	0.08	0.20	0.15	0.13	0.0	0.0	0.0	0.56
ENE	0.0	0.0	0.03	0.03	0.01	0.0	0.0	0.0	0.0	0.07
E	0.0	0.0	0.01	0.0	0.0	0.0	0.0	0.0	0.0	0.01
ESE	0.0	0.0	0.01	0.01	0.0	0.0	0.01	0.0	0.0	0.03
SE	0.0	0.0	0.01	0.02	0.0	0.0	0.0	0.0	0.0	0.03
SSE	0.0	0.0	0.01	0.03	0.02	0.02	0.01	0.0	0.0	0.09
S	0.0	0.0	0.01	0.04	0.06	0.05	0.01	0.0	0.0	0.17
SSW	0.0	0.0	0.01	0.09	0.18	0.16	0.01	0.0	0.0	0.45
SW	0.0	0.0	0.04	0.12	0.10	0.09	0.02	0.0	0.0	0.37
WSW	0.0	0.0	0.02	0.03	0.03	0.02	0.02	0.0	0.0	0.12
W	0.0	0.0	0.01	0.0	0.01	0.02	0.0	0.0	0.0	0.04
WNW	0.0	0.0	0.0	0.0	0.0	0.01	0.01	0.0	0.0	0.02
NW	0.0	0.0	0.01	0.01	0.01	0.05	0.01	0.0	0.0	0.09
NNW	0.0	0.0	0.01	0.0	0.02	0.08	0.01	0.0	0.0	0.12
Sub- total	0.01	0.01	0.31	0.80	0.83	0.83	0.12	0.0	0.0	2.90

958 stability class A occurrences out of total 32723 valid temperature difference readings.

934 valid wind direction/wind speed readings out of total 958 stability class A occurrences.

All columns and calm total 100 percent of net valid readings

* Meteorological Facility located 0.74 miles SW of Sequoyah Nuclear Plant.
 Temperature Instruments 33 and 150 feet above ground.
 Wind instruments 33 feet above ground.

OFFSITE DOSE CALCULATION/ENVIRONMENTAL MONITORING METHODOLOGIES

Table 7.2 (2 of 7)
 JOINT PERCENTAGE FREQUENCIES OF WIND DIRECTION AND WIND SPEED
 FOR DIFFERENT STABILITY CLASSES*

Sequoyah Nuclear Plant Meteorological Facility*
 Jan. 1, 72 - Dec. 31, 75

Stability Class B
 $-1.9 < \Delta T_1 - 1.7 \text{ deg. C/100m}$

	Calm	Wind Speed (mph)								Total
		0.6- 1.4	1.5- 3.4	3.5- 5.4	5.5- 7.4	7.5- 12.4	12.5- 18.4	18.5- 24.4	>24.5	
N	0.0	0.0	0.01	0.01	0.02	0.03	0.0	0.0	0.0	0.07
NNE	0.0	0.0	0.05	0.23	0.20	0.18	0.01	0.0	0.0	0.67
NE	0.01	0.0	0.08	0.29	0.09	0.06	0.0	0.0	0.0	0.52
ENE	0.0	0.0	0.03	0.03	0.01	0.0	0.0	0.0	0.0	0.07
E	0.0	0.0	0.02	0.01	0.0	0.0	0.0	0.0	0.0	0.03
ESE	0.0	0.0	0.0	0.01	0.0	0.0	0.0	0.0	0.0	0.01
SE	0.0	0.0	0.01	0.02	0.0	0.01	0.0	0.0	0.0	0.04
SSE	0.0	0.0	0.01	0.03	0.0	0.02	0.0	0.0	0.0	0.06
S	0.0	0.0	0.03	0.03	0.07	0.04	0.01	0.0	0.0	0.18
SSW	0.0	0.0	0.04	0.09	0.20	0.20	0.03	0.0	0.0	0.56
SW	0.0	0.0	0.03	0.11	0.14	0.10	0.02	0.0	0.0	0.40
WSW	0.0	0.0	0.01	0.01	0.03	0.02	0.01	0.01	0.0	0.09
W	0.0	0.0	0.0	0.0	0.01	0.01	0.0	0.0	0.0	0.02
WNW	0.0	0.0	0.0	0.01	0.01	0.03	0.0	0.0	0.0	0.05
NW	0.0	0.0	0.0	0.0	0.01	0.05	0.0	0.0	0.0	0.06
NNW	0.0	0.0	0.01	0.02	0.02	0.06	0.01	0.0	0.0	0.12
SUB-										
TOTAL	0.01	0.0	0.33	0.90	0.81	0.81	0.09	0.01	0.0	2.95

969 stability class B occurrences out of total 32723 valid temperature difference readings.

953 valid wind direction/wind speed readings out of total 969 stability class B occurrences.

All columns and calm total 100 percent of net valid readings.

*Meteorological facility located 0.74 miles SW of Sequoyah Nuclear Plant.
 Temperature instruments 33 and 150 feet above ground.
 Wind instruments 33 feet above ground.

OFFSITE DOSE CALCULATION/ENVIRONMENTAL MONITORING METHODOLOGIES

Table 7.2 (3 of 7)
 JOINT PERCENTAGE FREQUENCIES OF WIND DIRECTION AND WIND SPEED
 FOR DIFFERENT STABILITY CLASSES*

Sequoyah Nuclear Plant Meteorological Facility*
 Jan. 1, 72 - Dec. 31, 75

Stability Class C
 $-1.7 < \Delta T_{\Delta} < -1.5$ deg. C/100m

	Calm	Wind Speed (mph)								Total
		0.6- 1.4	1.5- 3.4	3.5- 5.4	5.5- 7.4	7.5- 12.4	12.5- 18.4	18.5- 24.4	>24.5	
N	0.0	0.0	0.01	0.02	0.02	0.02	0.0	0.0	0.0	0.07
NNE	0.0	0.0	0.05	0.12	0.11	0.11	0.0	0.0	0.0	0.39
NE	0.0	0.0	0.05	0.14	0.05	0.03	0.0	0.0	0.0	0.27
ENE	0.0	0.0	0.03	0.02	0.0	0.0	0.0	0.0	0.0	0.05
E	0.0	0.0	0.01	0.01	0.0	0.0	0.0	0.0	0.0	0.02
ESE	0.0	0.0	0.01	0.01	0.0	0.0	0.0	0.0	0.0	0.02
SE	0.0	0.0	0.01	0.01	0.0	0.0	0.0	0.0	0.0	0.02
SSE	0.0	0.0	0.01	0.02	0.0	0.02	0.0	0.0	0.0	0.05
S	0.0	0.0	0.03	0.04	0.06	0.05	0.0	0.0	0.0	0.18
SSW	0.0	0.0	0.01	0.11	0.14	0.13	0.02	0.0	0.0	0.41
SW	0.0	0.0	0.03	0.08	0.12	0.07	0.01	0.0	0.0	0.31
WSW	0.0	0.0	0.01	0.02	0.03	0.02	0.0	0.01	0.0	0.08
W	0.0	0.0	0.0	0.01	0.0	0.01	0.01	0.0	0.0	0.03
WNW	0.0	0.0	0.0	0.01	0.01	0.01	0.0	0.0	0.0	0.03
NW	0.0	0.0	0.0	0.0	0.02	0.03	0.01	0.0	0.0	0.06
NNW	0.0	0.0	0.0	0.02	0.02	0.05	0.0	0.0	0.0	0.09
SUB-										
TOTAL	0.0	0.0	0.26	0.64	0.58	0.55	0.05	0.0	0.0	2.08

684 stability class C occurrences out of total 32723 valid temperature difference readings.

672 valid wind direction/wind speed readings out of total 684 stability class C occurrences.

All columns and calm total 100 percent of net valid readings.

*Meteorological facility located 0.74 miles SW of Sequoyah Nuclear Plant.
 Temperature instruments 33 and 150 feet above ground.
 Wind instruments 33 feet above ground.

OFFSITE DOSE CALCULATION/ENVIRONMENTAL MONITORING METHODOLOGIES

Table 7.2 (4 of 7)
 JOINT PERCENTAGE FREQUENCIES OF WIND DIRECTION AND WIND SPEED
 FOR DIFFERENT STABILITY CLASSES*

Sequoyah Nuclear Plant Meteorological Facility*
 Jan. 1, 72 - Dec. 31, 75

Stability Class D
 $-1.5 < \Delta T_{\Delta} - 0.5 \text{ deg. C/100m}$

	Calm	Wind Speed (mph)								Total
		0.6- 1.4	1.5- 3.4	3.5- 5.4	5.5- 7.4	7.5- 12.4	12.5- 18.4	18.5- 24.4	>24.5	
N	0.003	0.01	0.24	0.22	0.16	0.17	0.0	0.0	0.0	0.80
NNE	0.017	0.06	0.73	1.03	0.84	0.78	0.07	0.0	0.0	3.51
NE	0.006	0.02	0.76	0.88	0.42	0.42	0.05	0.0	0.0	2.55
ENE	0.003	0.01	0.21	0.11	0.03	0.0	0.0	0.0	0.0	0.36
E	0.003	0.01	0.12	0.03	0.02	0.01	0.0	0.0	0.0	0.19
ESE	0.003	0.01	0.06	0.02	0.0	0.0	0.0	0.0	0.0	0.09
SE	0.0	0.0	0.12	0.08	0.0	0.0	0.0	0.0	0.0	0.20
SSE	0.0	0.0	0.15	0.15	0.05	0.06	0.01	0.01	0.0	0.43
S	0.003	0.01	0.31	0.53	0.38	0.25	0.02	0.0	0.0	1.50
SSW	0.003	0.01	0.44	1.25	0.95	0.70	0.07	0.0	0.0	3.42
SW	0.003	0.01	0.47	1.17	1.03	0.52	0.03	0.01	0.0	3.24
WSW	0.0	0.0	0.22	0.34	0.18	0.21	0.07	0.01	0.0	1.03
W	0.003	0.01	0.06	0.08	0.10	0.19	0.02	0.01	0.0	0.47
WNW	0.003	0.01	0.06	0.05	0.11	0.18	0.01	0.0	0.0	0.42
NW	0.0	0.0	0.08	0.08	0.22	0.31	0.03	0.0	0.0	0.72
NNW	0.003	0.01	0.15	0.14	0.25	0.36	0.02	0.0	0.0	0.93
SUB-										
TOTAL	0.05	0.18	4.18	6.16	4.74	4.16	0.40	0.04	0.0	19.86

6567 stability class D occurrences out of total 32723 valid temperature difference readings.

6345 valid wind direction/wind speed readings out of total 6567 stability class D occurrences.

All columns and calm total 100 percent of net valid readings.

*Meteorological facility located 0.74 miles SW of Sequoyah Nuclear Plant.
 Temperature instruments 33 and 150 feet above ground.
 Wind instruments 33 feet above ground.

OFFSITE DOSE CALCULATION/ENVIRONMENTAL MONITORING METHODOLOGIES

Table 7.2 (5 of 7)
 JOINT PERCENTAGE FREQUENCIES OF WIND DIRECTION AND WIND SPEED
 FOR DIFFERENT STABILITY CLASSES*

Sequoyah Nuclear Plant Meteorological Facility*
 Jan. 1, 72 - Dec. 31, 75

Stability Class E
 $-0.5 < \Delta T_1 < 1.5$ deg. C/100m

	Calm	Wind Speed (mph)								Total
		0.6- 1.4	1.5- 3.4	3.5- 5.4	5.5- 7.4	7.5- 12.4	12.5- 18.4	18.5- 24.4	>24.5	
N	0.017	0.23	1.26	0.83	0.39	0.27	0.0	0.0	0.0	2.98
NNE	0.023	0.31	2.83	2.46	1.07	0.92	0.03	0.0	0.0	7.62
NE	0.011	0.15	1.03	0.71	0.31	0.18	0.01	0.0	0.0	2.39
ENE	0.009	0.12	0.48	0.16	0.04	0.0	0.0	0.0	0.0	0.80
E	0.010	0.14	0.24	0.05	0.01	0.01	0.0	0.0	0.0	0.45
ESE	0.007	0.09	0.11	0.01	0.01	0.01	0.01	0.0	0.0	0.24
SE	0.007	0.10	0.37	0.06	0.01	0.01	0.0	0.0	0.0	0.55
SSE	0.008	0.11	0.58	0.24	0.13	0.23	0.04	0.02	0.0	1.35
S	0.013	0.17	1.33	1.49	0.91	1.05	0.08	0.0	0.0	5.03
SSW	0.007	0.10	1.67	2.32	1.67	1.45	0.11	0.0	0.0	7.32
SW	0.013	0.17	1.59	2.07	1.30	0.99	0.10	0.0	0.0	6.22
WSW	0.010	0.13	0.87	0.55	0.35	0.40	0.06	0.0	0.0	2.36
W	0.007	0.10	0.42	0.28	0.21	0.22	0.03	0.0	0.0	1.26
WNW	0.010	0.14	0.37	0.22	0.19	0.27	0.02	0.0	0.0	1.21
NW	0.007	0.10	0.50	0.37	0.43	0.38	0.02	0.0	0.0	1.80
NNW	0.011	0.15	0.80	0.68	0.57	0.40	0.01	0.0	0.0	2.61
Sub- total	0.17	2.31	14.45	12.50	7.60	6.79	0.52	0.02	0.0	44.19

14624 stability class E occurrences out of total 32723 valid temperature difference readings.

14146 valid wind direction/wind speed readings out of total 14624 stability class E occurrences.

All columns and calm total 100 percent of net valid readings.

*Meteorological facility located 0.74 miles SW of Sequoyah Nuclear Plant.
 Temperature instruments 33 and 150 feet above ground.
 Wind instruments 33 feet above ground.

OFFSITE DOSE CALCULATION/ENVIRONMENTAL MONITORING METHODOLOGIES

Table 7.2 (6 of 7)
 JOINT PERCENTAGE FREQUENCIES OF WIND DIRECTION AND WIND SPEED
 FOR DIFFERENT STABILITY CLASSES*

R27

Sequoyah Nuclear Plant Meteorological Facility*
 Jan. 1, 72 - Dec. 31, 75

Stability Class F
 $1.5 < \Delta T \leq 4.0$ deg. C/100m

	Calm	Wind Speed (mph)								Total
		0.6- 1.4	1.5- 3.4	3.5- 5.4	5.5- 7.4	7.5- 12.4	12.5- 18.4	18.5- 24.4	>24.5	
N	0.011	0.21	1.37	0.44	0.04	0.0	0.0	0.0	0.0	2.06
NNE	0.018	0.35	3.61	0.84	0.05	0.0	0.0	0.0	0.0	4.85
NE	0.011	0.21	1.15	0.28	0.01	0.0	0.0	0.0	0.0	1.65
ENE	0.008	0.16	0.39	0.03	0.0	0.0	0.0	0.0	0.0	0.58
E	0.010	0.20	0.22	0.0	0.0	0.0	0.0	0.0	0.0	0.42
ESE	0.007	0.13	0.18	0.02	0.0	0.0	0.0	0.0	0.0	0.33
SE	0.007	0.14	0.23	0.02	0.0	0.0	0.0	0.0	0.0	0.39
SSE	0.008	0.15	0.37	0.07	0.03	0.01	0.0	0.0	0.0	0.63
S	0.009	0.17	0.77	0.30	0.10	0.06	0.0	0.0	0.0	1.40
SSW	0.006	0.12	1.13	0.71	0.26	0.11	0.0	0.0	0.0	2.33
SW	0.005	0.10	0.99	0.86	0.27	0.13	0.0	0.0	0.0	2.35
WSW	0.005	0.09	0.46	0.19	0.04	0.01	0.0	0.0	0.0	0.79
W	0.004	0.07	0.20	0.07	0.01	0.0	0.0	0.0	0.0	0.35
WNW	0.005	0.10	0.24	0.07	0.01	0.0	0.0	0.0	0.0	0.42
NW	0.003	0.05	0.29	0.15	0.05	0.01	0.0	0.0	0.0	0.55
NNW	0.005	0.09	0.52	0.34	0.05	0.01	0.0	0.0	0.0	1.01
SUB-										
TOTAL	0.12	2.34	12.12	4.39	0.92	0.34	0.0	0.0	0.0	20.11

6542 stability class F occurrences out of total 32723 valid temperature difference readings.

6461 valid wind direction/wind speed readings out of total 6542 stability class F occurrences.

All columns and calm total 100 percent of net valid readings.

*Meteorological facility located 0.74 miles SW of Sequoyah Nuclear Plant.
 Temperature instruments 33 and 150 feet above ground.
 Wind instruments 33 feet above ground.

OFFSITE DOSE CALCULATION/ENVIRONMENTAL MONITORING METHODOLOGIES

Table 7.2 (7 of 7)
 JOINT PERCENTAGE FREQUENCIES OF WIND DIRECTION AND WIND SPEED
 FOR DIFFERENT STABILITY CLASSES*

Sequoyah Nuclear Plant Meteorological Facility*
 Jan. 1, 72 - Dec. 31, 75

Stability Class G
 Delta T > 4.0 deg. C/100m

	Calm	Wind Speed (mph)								Total
		0.6- 1.4	1.5- 3.4	3.5- 5.4	5.5- 7.4	7.5- 12.4	12.5- 18.4	18.5- 24.4	>24.5	
N	0.003	0.06	0.33	0.09	0.0	0.0	0.0	0.0	0.0	0.48
NNE	0.005	0.10	1.03	0.20	0.0	0.0	0.0	0.0	0.0	1.33
NE	0.005	0.09	0.74	0.12	0.0	0.0	0.0	0.0	0.0	0.95
ENE	0.007	0.13	0.42	0.02	0.0	0.0	0.0	0.0	0.0	0.57
E	0.007	0.14	0.18	0.01	0.0	0.0	0.0	0.0	0.0	0.33
ESE	0.006	0.11	0.08	0.01	0.0	0.0	0.0	0.0	0.0	0.20
SE	0.005	0.09	0.08	0.0	0.0	0.0	0.0	0.0	0.0	0.17
SSE	0.008	0.16	0.21	0.0	0.01	0.0	0.0	0.0	0.0	0.37
S	0.006	0.11	0.39	0.04	0.02	0.0	0.0	0.0	0.0	0.55
SSW	0.003	0.06	0.48	0.32	0.06	0.01	0.0	0.0	0.0	0.89
SW	0.002	0.03	0.44	0.42	0.0	0.0	0.0	0.0	0.0	0.95
WSW	0.001	0.01	0.11	0.07	0.0	0.0	0.0	0.0	0.0	0.19
W	0.002	0.03	0.08	0.02	0.0	0.0	0.0	0.0	0.0	0.13
WNW	0.001	0.01	0.03	0.01	0.0	0.01	0.0	0.0	0.0	0.06
NW	0.001	0.02	0.06	0.03	0.0	0.0	0.0	0.0	0.0	0.11
NNW	0.001	0.02	0.08	0.03	0.0	0.0	0.0	0.0	0.0	0.13
SUB-										
TOTAL	0.06	1.17	4.74	1.39	0.09	0.2	0.0	0.0	0.0	7.41

2379 stability class G occurrences out of total 32723 valid temperature difference readings.

2378 valid wind direction/wind speed readings out of total 2379 stability class G occurrences.

All columns and calm total 100 percent of net valid readings.

*Meteorological facility located 0.74 Miles SW of Sequoyah Nuclear Plant.
 Temperature instruments 33 and 150 feet above ground.
 Wind instruments 33 feet above ground.

OFFSITE DOSE CALCULATION/ENVIRONMENTAL MONITORING METHODOLOGIES

Table 7.3
 DOSE FACTORS FOR SUBMERSION IN NOBLE GASES

	Submersion dose mrem/yr per $\mu\text{Ci}/\text{m}^3$		Air dose mrad/yr per $\mu\text{Ci}/\text{m}^3$	
	DF _{B_i}	DF _{S_i}	DF _{Y_i}	DF _{B_i}
Kr-83m	7.56E-02	---	1.93E+01	2.88E+02
Kr-85m	1.17E+03	1.46E+03	1.23E+03	1.97E+03
Kr-85	1.61E+01	1.34E+03	1.72E+01	1.95E+03
Kr-87	5.92E+03	9.73E+03	6.17E+03	1.03E+04
Kr-88	1.47E+04	2.37E+03	1.52E+04	2.93E+03
Kr-89	1.66E+04	1.01E+04	1.73E+04	1.06E+04
Kr-90	1.56E+04	7.29E+03	1.63E+04	7.83E+03
Xe-131m	9.15E+01	4.76E+02	1.56E+02	1.11E+03
Xe-133m	2.51E+02	9.94E+02	3.27E+02	1.48E+03
Xe-133	2.94E+02	3.06E+02	3.53E+02	1.05E+03
Xe-135m	3.12E+03	7.11E+02	3.36E+03	7.39E+02
Xe-135	1.81E+03	1.86E+03	1.92E+03	2.46E+03
Xe-137	1.42E+03	1.22E+04	1.51E+03	1.27E+04
Xe-138	8.83E+03	4.13E+03	9.21E+03	4.75E+03
Ar-41	8.84E+03	2.69E+03	9.30E+03	3.28E+03

Reference:
 Regulatory Guide 1.109, Table B-1.

OFFSITE DOSE CALCULATION/ENVIRONMENTAL MONITORING METHODOLOGIES

Table 7.4
SECTOR ELEMENTS CONSIDERED FOR POPULATION DOSES

Range of Sector Element	Midpoint of Sector Element
Site boundary - 1 mile	0.8 mile
1 - 2 miles	1.5 miles
2 - 3 miles	2.5 miles
3 - 4 miles	3.5 miles
4 - 5 miles	4.5 miles
5 - 10 miles	7.5 miles
10 - 20 miles	15 miles
20 - 30 miles	25 miles
30 - 40 miles	35 miles
40 - 50 miles	45 miles

OFFSITE DOSE CALCULATION/ENVIRONMENTAL MONITORING METHODOLOGIES

Table 7.5
 POPULATION WITHIN EACH SECTOR ELEMENT

	Sector Midpoint (miles)									
	0.8	1.5	2.5	3.5	4.5	7.5	15	25	35	45
N	20	41	213	129	66	1784	5453	3470	2610	11145
NNE	0	30	123	182	62	600	10628	4910	8250	10625
NE	0	0	67	67	94	581	2884	6998	7047	18080
ENE	0	11	24	222	300	773	4707	5747	29477	18679
E	0	70	11	191	137	918	17440	6808	5072	4129
ESE	0	118	113	194	137	1849	46521	5044	1896	13624
SE	0	179	322	168	205	1507	6005	5461	15641	3417
SSE	0	125	370	750	601	2347	13242	8596	34279	11648
S	0	67	143	229	811	3930	28008	26690	19642	11622
SSW	0	82	140	400	170	8927	96966	55597	21349	11978
SW	0	10	306	634	194	9787	94225	23455	11641	11109
WSW	20	190	642	1124	1669	19089	28405	4106	15081	9548
W	10	20	233	657	657	5225	1580	6350	5699	7707
WNW	10	30	365	598	598	2622	6540	4920	6699	2450
NW	50	80	292	569	336	2696	1410	1750	1217	15856
NNW	10	263	80	75	213	1610	471	3130	2835	5719

OFFSITE DOSE CALCULATION/ENVIRONMENTAL MONITORING METHODOLOGIES

Table 7.6 (1 of 8)
 INHALATION DOSE FACTORS
 (mrem/pCi inhaled)

	ADULT						
	bone	liver	t body	thyroid	kidney	lung	gi-lli
H-3	1.58E-07	1.58E-07	1.58E-07	1.58E-07	1.58E-07	1.58E-07	1.58E-07
C-14	2.27E-06	4.25E-07	4.26E-07	4.26E-07	4.26E-07	4.26E-07	4.26E-07
Na-24	1.28E-06	1.28E-06	1.28E-06	1.28E-06	1.28E-06	1.28E-06	1.28E-06
P-32	1.65E-04	9.64E-06	6.26E-06	0.00E+00	0.00E+00	0.00E+00	1.08E-05
Cr-51	0.00E+00	0.00E+00	1.25E-08	7.44E-09	2.85E-09	1.80E-06	4.15E-07
Mn-54	0.00E+00	4.95E-06	7.87E-07	0.00E+00	1.23E-06	1.75E-04	9.67E-06
Mn-56	0.00E+00	1.55E-10	2.29E-11	0.00E+00	1.63E-10	1.18E-06	2.53E-06
Fe-55	3.07E-06	2.12E-06	4.93E-07	0.00E+00	0.00E+00	9.01E-06	7.54E-07
Fe-59	1.47E-06	3.47E-06	1.32E-06	0.00E+00	0.00E+00	1.27E-04	2.35E-05
Co-57	0.00E+00	8.65E-08	8.39E-08	0.00E+00	0.00E+00	4.62E-05	3.93E-06
Co-58	0.00E+00	1.98E-07	2.59E-07	0.00E+00	0.00E+00	1.16E-04	1.33E-05
Co-60	0.00E+00	1.44E-06	1.85E-06	0.00E+00	0.00E+00	7.46E-04	3.56E-05
Ni-63	5.40E-05	3.93E-06	1.81E-06	0.00E+00	0.00E+00	2.23E-05	1.67E-06
Ni-65	1.92E-10	2.62E-11	1.14E-11	0.00E+00	0.00E+00	7.00E-07	1.54E-06
Cu-64	0.00E+00	1.83E-10	7.69E-11	0.00E+00	5.78E-10	8.48E-07	6.12E-06
Zn-65	4.05E-06	1.29E-05	5.82E-06	0.00E+00	8.62E-06	1.08E-04	6.68E-06
Zn-69	4.23E-12	8.14E-12	5.65E-13	0.00E+00	5.27E-12	1.15E-07	2.04E-09
Zn-69m	1.02E-09	2.45E-09	2.24E-10	0.00E+00	1.48E-09	2.38E-06	1.71E-05
Br-82	0.00E+00	0.00E+00	1.69E-06	0.00E+00	0.00E+00	0.00E+00	1.30E-06
Br-83	0.00E+00	0.00E+00	3.01E-08	0.00E+00	0.00E+00	0.00E+00	2.90E-08
Br-84	0.00E+00	0.00E+00	3.91E-08	0.00E+00	0.00E+00	0.00E+00	2.05E-13
Br-85	0.00E+00	0.00E+00	1.60E-09	0.00E+00	0.00E+00	0.00E+00	0.00E+00
Rb-86	0.00E+00	1.69E-05	7.37E-06	0.00E+00	0.00E+00	0.00E+00	2.08E-06
Rb-88	0.00E+00	4.84E-08	2.41E-08	0.00E+00	0.00E+00	0.00E+00	4.18E-19
Rb-89	0.00E+00	3.20E-08	2.12E-08	0.00E+00	0.00E+00	0.00E+00	1.16E-21
Sr-89	3.80E-05	0.00E+00	1.09E-06	0.00E+00	0.00E+00	1.75E-04	4.37E-05
Sr-90	1.24E-02	0.00E+00	7.62E-04	0.00E+00	0.00E+00	1.20E-03	9.02E-05
Sr-91	7.74E-09	0.00E+00	3.13E-10	0.00E+00	0.00E+00	4.56E-06	2.39E-05
Sr-92	8.43E-10	0.00E+00	3.64E-11	0.00E+00	0.00E+00	2.06E-06	5.38E-06
Y-90	2.61E-07	0.00E+00	7.01E-09	0.00E+00	0.00E+00	2.12E-05	6.32E-05
Y-91m	3.26E-11	0.00E+00	1.27E-12	0.00E+00	0.00E+00	2.40E-07	1.66E-10
Y-91	5.78E-05	0.00E+00	1.55E-06	0.00E+00	0.00E+00	2.13E-04	4.81E-05
Y-92	1.29E-09	0.00E+00	3.77E-11	0.00E+00	0.00E+00	1.96E-06	9.19E-06
Y-93	1.18E-08	0.00E+00	3.26E-10	0.00E+00	0.00E+00	6.06E-06	5.27E-05
Zr-95	1.34E-05	4.30E-06	2.91E-06	0.00E+00	6.77E-06	2.21E-04	1.88E-05
Zr-97	1.21E-08	2.45E-09	1.13E-09	0.00E+00	3.71E-09	9.84E-06	6.54E-05
Nb-95	1.76E-06	9.77E-07	5.26E-07	0.00E+00	9.67E-07	6.31E-05	1.30E-05
Nb-97	2.78E-11	7.03E-12	2.56E-12	0.00E+00	8.18E-12	3.00E-07	3.02E-08
Mo-99	0.00E+00	1.51E-08	2.87E-09	0.00E+00	3.64E-08	1.14E-05	3.10E-05
Tc-99m	1.29E-13	3.64E-13	4.63E-12	0.00E+00	5.52E-12	9.55E-08	5.20E-07
Tc-101	5.22E-15	7.52E-15	7.38E-14	0.00E+00	1.35E-13	4.99E-08	1.36E-21
Ru-103	1.91E-07	0.00E+00	8.23E-08	0.00E+00	7.29E-07	6.31E-05	1.38E-05
Ru-105	9.88E-11	0.00E+00	3.89E-11	0.00E+00	1.27E-10	1.37E-06	6.02E-06
Ru-106	8.64E-06	0.00E+00	1.09E-06	0.00E+00	1.67E-05	1.17E-03	1.14E-04
Ag-110m	1.35E-06	1.25E-06	7.43E-07	0.00E+00	2.46E-06	5.79E-04	3.78E-05

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OFFSITE DOSE CALCULATION/ENVIRONMENTAL MONITORING METHODOLOGIES

Table 7.6 (2 of 8)
 INHALATION DOSE FACTORS
 (mrem/pCi inhaled)

	ADULT							
	bone	liver	t body	thyroid	kidney	lung	gi-lli	
Sb-124	3.90E-06	7.36E-08	1.55E-06	9.44E-09	0.00E+00	3.10E-04	5.08E-05	
Sb-125	6.67E-06	7.44E-08	1.58E-06	6.75E-09	0.00E+00	2.18E-04	1.26E-05	
Sn-125	1.16E-06	3.12E-08	7.03E-08	2.59E-08	0.00E+00	7.37E-05	6.81E-05	R25
Te-125m	4.27E-07	1.98E-07	5.84E-08	1.31E-07	1.55E-06	3.92E-05	8.83E-06	
Te-127m	1.58E-06	7.21E-07	1.96E-07	4.11E-07	5.72E-06	1.20E-04	1.87E-05	
Te-127	1.75E-10	8.03E-11	3.87E-11	1.32E-10	6.37E-10	8.14E-07	7.17E-06	
Te-129m	1.22E-06	5.84E-07	1.98E-07	4.30E-07	4.57E-06	1.45E-04	4.79E-05	
Te-129	6.22E-12	2.99E-12	1.55E-12	4.87E-12	2.34E-11	2.42E-07	1.96E-08	
Te-131m	8.74E-09	5.45E-09	3.63E-09	6.88E-09	3.86E-08	1.82E-05	6.95E-05	
Te-131	1.39E-12	7.44E-13	4.49E-13	1.17E-12	5.46E-12	1.74E-07	2.30E-09	
Te-132	3.25E-08	2.69E-08	2.02E-08	2.37E-08	1.82E-07	3.60E-05	6.37E-05	
I-130	5.72E-07	1.68E-06	6.60E-07	1.42E-04	2.61E-06	0.00E+00	9.61E-07	
I-131	3.15E-06	4.47E-06	2.56E-06	1.49E-03	7.66E-06	0.00E+00	7.85E-07	
I-132	1.45E-07	4.07E-07	1.45E-07	1.43E-05	6.48E-07	0.00E+00	5.08E-08	
I-133	1.08E-06	1.85E-06	5.65E-07	2.69E-04	3.23E-06	0.00E+00	1.11E-06	
I-134	8.05E-08	2.16E-07	7.69E-08	3.73E-06	3.44E-07	0.00E+00	1.26E-10	
I-135	3.35E-07	8.73E-07	3.21E-07	5.60E-05	1.39E-06	0.00E+00	6.56E-07	
Cs-134	4.66E-05	1.06E-04	9.10E-05	0.00E+00	3.59E-05	1.22E-05	1.30E-06	
Cs-136	4.88E-06	1.83E-05	1.38E-05	0.00E+00	1.07E-05	1.50E-06	1.46E-06	
Cs-137	5.98E-05	7.76E-05	5.35E-05	0.00E+00	2.78E-05	9.40E-06	1.05E-06	
Cs-138	4.14E-08	7.76E-08	4.05E-08	0.00E+00	6.00E-08	6.07E-09	2.33E-13	
Ba-139	1.17E-10	8.32E-14	3.42E-12	0.00E+00	7.78E-14	4.70E-07	1.12E-07	
Ba-140	4.88E-06	6.13E-09	3.21E-07	0.00E+00	2.09E-09	1.59E-04	2.73E-05	
Ba-141	1.25E-11	9.41E-15	4.20E-13	0.00E+00	8.75E-15	2.42E-07	1.45E-17	
Ba-142	3.29E-12	3.38E-15	2.07E-13	0.00E+00	2.86E-15	1.49E-07	1.96E-26	
La-140	4.30E-08	2.17E-08	5.73E-09	0.00E+00	0.00E+00	1.70E-05	5.73E-05	
La-142	8.54E-11	3.88E-11	9.65E-12	0.00E+00	0.00E+00	7.91E-07	2.64E-07	
Ce-141	2.49E-06	1.69E-06	1.91E-07	0.00E+00	7.83E-07	4.52E-05	1.50E-05	
Ce-143	2.33E-08	1.72E-08	1.91E-09	0.00E+00	7.60E-09	9.97E-06	2.83E-05	
Ce-144	4.29E-04	1.79E-04	2.30E-05	0.00E+00	1.06E-04	9.72E-04	1.02E-04	
Pr-143	1.17E-06	4.69E-07	5.80E-08	0.00E+00	2.70E-07	3.51E-05	2.50E-05	
Pr-144	3.76E-12	1.56E-12	1.91E-13	0.00E+00	8.81E-13	1.27E-07	2.69E-18	
Nd-147	6.59E-07	7.62E-07	4.56E-08	0.00E+00	4.45E-07	2.76E-05	2.16E-05	
W-187	1.06E-09	8.85E-10	3.10E-10	0.00E+00	0.00E+00	3.63E-06	1.94E-05	
Np-239	2.87E-08	2.82E-09	1.55E-09	0.00E+00	8.75E-09	4.70E-06	1.49E-05	

NOTE: The tritium dose factor for bone is assumed to be equal to the total body dose factor.

Reference:

Regulatory Guide 1.109, Table E-7.

Dose Factors for Co-57, Zn-69m, Br-82, Nb-97, Sn-125, Sb-124 and Sb-125 are from NUREG-0172 Age Specific Radiation Dose Commitment Factors for a One Year Chronic Intake, November 1977, Table 8. R25

OFFSITE DOSE CALCULATION/ENVIRONMENTAL MONITORING METHODOLOGIES

Table 7.6 (3 of 8)
 INHALATION DOSE FACTORS
 (mrem/pCi inhaled)

	TEEN						
	bone	liver	t. body	thyroid	kidney	lung	gi-lli
H-3	1.59E-07	1.59E-07	1.59E-07	1.59E-07	1.59E-07	1.59E-07	1.59E-07
C-14	3.25E-06	6.09E-07	6.09E-07	6.09E-07	6.09E-07	6.09E-07	6.09E-07
Na-24	1.72E-06	1.72E-06	1.72E-06	1.72E-06	1.72E-06	1.72E-06	1.72E-06
P-32	2.36E-04	1.37E-05	8.95E-06	0.00E+00	0.00E+00	0.00E+00	1.16E-05
Cr-51	0.00E+00	0.00E+00	1.69E-08	9.37E-09	3.84E-09	2.62E-06	3.75E-07
Mn-54	0.00E+00	6.39E-06	1.05E-06	0.00E+00	1.59E-06	2.48E-04	8.35E-06
Mn-56	0.00E+00	2.12E-10	3.15E-11	0.00E+00	2.24E-10	1.90E-06	7.18E-06
Fe-55	4.18E-06	2.98E-06	6.93E-07	0.00E+00	0.00E+00	1.55E-05	7.99E-07
Fe-59	1.99E-06	4.62E-06	1.79E-06	0.00E+00	0.00E+00	1.91E-04	2.23E-05
Co-57	0.00E+00	1.18E-07	1.15E-07	0.00E+00	0.00E+00	7.33E-05	3.93E-06
Co-58	0.00E+00	2.59E-07	3.47E-07	0.00E+00	0.00E+00	1.68E-04	1.19E-05
Co-60	0.00E+00	1.89E-06	2.48E-06	0.00E+00	0.00E+00	1.09E-03	3.24E-05
Ni-63	7.25E-05	5.43E-06	2.47E-06	0.00E+00	0.00E+00	3.84E-05	1.77E-06
Ni-65	2.73E-10	3.66E-11	1.59E-11	0.00E+00	0.00E+00	1.17E-06	4.59E-06
Cu-64	0.00E+00	2.54E-10	1.06E-10	0.00E+00	8.01E-10	1.39E-06	7.68E-06
Zn-65	4.82E-06	1.67E-05	7.80E-06	0.00E+00	1.08E-05	1.55E-04	5.83E-06
Zn-69	6.04E-12	1.15E-11	8.07E-13	0.00E+00	7.53E-12	1.98E-07	3.56E-08
Zn-69m	1.44E-09	3.39E-09	3.11E-10	0.00E+00	2.06E-09	3.92E-06	2.14E-05
Br-82	0.00E+00	0.00E+00	2.28E-06	0.00E+00	0.00E+00	0.00E+00	0.00E+00
Br-83	0.00E+00	0.00E+00	4.30E-08	0.00E+00	0.00E+00	0.00E+00	0.00E+00
Br-84	0.00E+00	0.00E+00	5.41E-08	0.00E+00	0.00E+00	0.00E+00	0.00E+00
Br-85	0.00E+00	0.00E+00	2.29E-09	0.00E+00	0.00E+00	0.00E+00	0.00E+00
Rb-86	0.00E+00	2.38E-05	1.05E-05	0.00E+00	0.00E+00	0.00E+00	2.21E-06
Rb-88	0.00E+00	6.82E-08	3.40E-08	0.00E+00	0.00E+00	0.00E+00	3.65E-15
Rb-89	0.00E+00	4.40E-08	2.91E-08	0.00E+00	0.00E+00	0.00E+00	4.22E-17
Sr-89	5.43E-05	0.00E+00	1.56E-06	0.00E+00	0.00E+00	3.02E-04	4.64E-05
Sr-90	1.35E-02	0.00E+00	8.35E-04	0.00E+00	0.00E+00	2.06E-03	9.56E-05
Sr-91	1.10E-08	0.00E+00	4.39E-10	0.00E+00	0.00E+00	7.59E-06	3.24E-05
Sr-92	1.19E-09	0.00E+00	5.08E-11	0.00E+00	0.00E+00	3.43E-06	1.49E-05
Y-90	3.73E-07	0.00E+00	1.00E-08	0.00E+00	0.00E+00	3.66E-05	6.99E-05
Y-91m	4.63E-11	0.00E+00	1.77E-12	0.00E+00	0.00E+00	4.00E-07	3.77E-09
Y-91	8.26E-05	0.00E+00	2.21E-06	0.00E+00	0.00E+00	3.67E-04	5.11E-05
Y-92	1.84E-09	0.00E+00	5.36E-11	0.00E+00	0.00E+00	3.35E-06	2.06E-05
Y-93	1.69E-08	0.00E+00	4.65E-10	0.00E+00	0.00E+00	1.04E-05	7.24E-05
Zr-95	1.82E-05	5.73E-06	3.94E-06	0.00E+00	8.42E-06	3.36E-04	1.86E-05
Zr-97	1.72E-08	3.40E-09	1.57E-09	0.00E+00	5.15E-09	1.62E-05	7.88E-05
Nb-95	2.32E-06	1.29E-06	7.08E-07	0.00E+00	1.25E-06	9.39E-05	1.21E-05
Nb-97	3.92E-11	9.72E-12	3.55E-12	0.00E+00	1.14E-11	4.91E-07	2.71E-07
Mo-99	0.00E+00	2.11E-08	4.03E-09	0.00E+00	5.14E-08	1.92E-05	3.36E-05
Tc-99m	1.73E-13	4.83E-13	6.24E-12	0.00E+00	7.20E-12	1.44E-07	7.66E-07
Tc-101	7.40E-15	1.05E-14	1.03E-13	0.00E+00	1.90E-13	8.34E-08	1.09E-16
Ru-103	2.63E-07	0.00E+00	7.7E-07	0.00E+00	9.29E-07	9.79E-05	1.36E-05
Ru-105	1.40E-10	0.00E+00	5.42E-11	0.00E+00	1.76E-10	2.27E-06	1.13E-05
Ru-106	1.23E-05	0.00E+00	1.55E-06	0.00E+00	2.38E-05	2.01E-03	1.20E-04
Ag-110m	1.73E-06	1.64E-06	9.99E-07	0.00E+00	3.13E-06	8.44E-04	3.41E-05

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OFFSITE DOSE CALCULATION/ENVIRONMENTAL MONITORING METHODOLOGIES

Table 7.6 (4 of 8)
 INHALATION DOSE FACTORS
 (mrem/pCi inhaled)

	TEEN						
	bone	liver	t body	thyroid	kidney	lung	gi-lli
Sb-124	5.38E-06	9.92E-08	2.10E-06	1.22E-08	0.00E+00	4.81E-04	4.98E-05
Sb-125	9.23E-06	1.01E-07	2.15E-06	8.80E-09	0.00E+00	3.42E-04	1.24E-05
Sn-125	1.66E-06	4.42E-08	9.99E-08	3.45E-08	0.00E+00	1.26E-04	7.29E-05 R25
Te-125m	6.10E-07	2.80E-07	8.34E-08	1.75E-07	0.00E+00	6.70E-05	9.38E-06
Te-127m	2.25E-06	1.02E-06	2.73E-07	5.48E-07	8.17E-06	2.07E-04	1.99E-05
Te-127	2.51E-10	1.14E-10	5.52E-11	1.77E-10	9.10E-10	1.40E-06	1.01E-05
Te-129m	1.74E-06	8.23E-07	2.81E-07	5.72E-07	6.49E-06	2.47E-04	5.06E-05
Te-129	8.87E-12	4.22E-12	2.20E-12	6.48E-12	3.32E-11	4.12E-07	2.02E-07
Te-131m	1.23E-08	7.51E-09	5.03E-09	9.06E-09	5.49E-08	2.97E-05	7.76E-05
Te-131	1.97E-12	1.04E-12	6.30E-13	1.55E-12	7.72E-12	2.92E-07	1.89E-09
Te-132	4.50E-08	3.63E-08	2.74E-08	3.07E-08	2.44E-07	5.61E-05	5.79E-05
I-130	7.80E-07	2.24E-06	8.96E-07	1.86E-04	3.44E-06	0.00E+00	1.14E-06
I-131	4.43E-06	6.14E-06	3.30E-06	1.83E-03	1.05E-05	0.00E+00	8.11E-07
I-132	1.99E-07	5.47E-07	1.97E-07	1.89E-05	8.65E-07	0.00E+00	1.59E-07
I-133	1.52E-06	2.56E-06	7.78E-07	3.65E-04	4.49E-06	0.00E+00	1.29E-06
I-134	1.11E-07	2.90E-07	1.05E-07	4.94E-06	4.58E-07	0.00E+00	2.55E-09
I-135	4.62E-07	1.18E-06	4.36E-07	7.76E-05	1.86E-06	0.00E+00	8.69E-07
Cs-134	6.28E-05	1.41E-04	6.86E-05	0.00E+00	4.69E-05	1.83E-05	1.22E-06
Cs-136	6.44E-06	2.42E-05	1.71E-05	0.00E+00	1.38E-05	2.22E-06	1.36E-06
Cs-137	8.33E-05	1.06E-04	3.89E-05	0.00E+00	3.80E-05	1.51E-05	1.06E-06
Cs-138	5.82E-08	1.07E-07	5.58E-08	0.00E+00	8.28E-08	9.84E-09	3.38E-11
Ba-139	1.67E-10	1.18E-13	4.87E-12	0.00E+00	1.11E-13	8.08E-07	8.06E-07
Ba-140	6.84E-06	8.38E-09	4.40E-07	0.00E+00	2.85E-09	2.54E-04	2.86E-05
Ba-141	1.78E-11	1.32E-14	5.93E-13	0.00E+00	1.23E-14	4.11E-07	9.33E-14
Ba-142	4.62E-12	4.63E-15	2.84E-13	0.00E+00	3.92E-15	2.39E-07	5.99E-20
La-140	5.99E-08	2.95E-08	7.82E-09	0.00E+00	0.00E+00	2.68E-05	6.09E-05
La-142	1.20E-10	5.31E-11	1.32E-11	0.00E+00	0.00E+00	1.27E-06	1.50E-06
Ce-141	3.55E-06	2.37E-06	2.71E-07	0.00E+00	1.11E-06	7.67E-05	1.58E-05
Ce-143	3.32E-08	2.42E-08	2.70E-09	0.00E+00	1.08E-08	1.63E-05	3.19E-05
Ce-144	6.11E-04	2.53E-04	3.28E-05	0.00E+00	1.51E-04	1.67E-03	1.08E-04
Pr-143	1.67E-06	6.64E-07	8.28E-08	0.00E+00	3.86E-07	6.04E-05	2.67E-05
Pr-144	5.37E-12	2.20E-12	2.72E-13	0.00E+00	1.26E-12	2.19E-07	2.94E-14
Nd-147	9.83E-07	1.07E-06	6.41E-08	0.00E+00	6.28E-07	4.65E-05	2.28E-05
W-187	1.50E-09	1.22E-09	4.29E-10	0.00E+00	0.00E+00	5.92E-06	2.21E-05
Np-239	4.23E-08	3.99E-09	2.21E-09	0.00E+00	1.25E-08	8.11E-06	1.65E-05

NOTE: The tritium dose factor for bone is assumed to be equal to the total body dose factor.

Reference:

Regulatory Guide 1.109, Table E-8.

Dose Factors for Co-57, Zn-69m, Br-82, Nb-97, Sn-125, Sb-124 and Sb-125
 are from NUREG-0172 Age Specific Radiation Dose Commitment
Factors for a One Year Chronic Intake, November 1977, Table 8. R25

OFFSITE DOSE CALCULATION/ENVIRONMENTAL MONITORING METHODOLOGIES

Table 7.6 (5 of 8)
 INHALATION DOSE FACTORS
 (mrem/pCi inhaled)

	CHILD						
	bone	liver	t body	thyroid	kidney	lung	gi-lli
H-3	3.04E-07	3.04E-07	3.04E-07	3.04E-07	3.04E-07	3.04E-07	3.04E-07
C-14	9.70E-06	1.82E-06	1.82E-06	1.82E-06	1.82E-06	1.82E-06	1.82E-06
Na-24	4.35E-06	4.35E-06	4.35E-06	4.35E-06	4.35E-06	4.35E-06	4.35E-06
P-32	7.04E-04	3.09E-05	2.67E-05	0.00E+00	0.00E+00	0.00E+00	1.14E-05
Cr-51	0.00E+00	0.00E+00	4.17E-08	2.31E-08	6.57E-09	4.59E-06	2.93E-07
Mn-54	0.00E+00	1.16E-05	2.57E-06	0.00E+00	2.71E-06	4.26E-04	6.19E-06
Mn-56	0.00E+00	4.48E-10	8.43E-11	0.00E+00	4.52E-10	3.55E-06	3.33E-05
Fe-55	1.28E-05	6.80E-06	2.10E-06	0.00E+00	0.00E+00	3.00E-05	7.75E-07
Fe-59	5.59E-06	9.04E-06	4.51E-06	0.00E+00	0.00E+00	3.43E-04	1.91E-05
Co-57	0.00E+00	2.44E-07	2.88E-07	0.00E+00	0.00E+00	1.37E-04	3.58E-06
Co-58	0.00E+00	4.79E-07	8.55E-07	0.00E+00	0.00E+00	2.99E-04	9.29E-06
Co-60	0.00E+00	3.55E-06	6.12E-06	0.00E+00	0.00E+00	1.91E-03	2.60E-05
Ni-63	2.22E-04	1.25E-05	7.56E-06	0.00E+00	0.00E+00	7.43E-05	1.71E-06
Ni-65	8.08E-10	7.99E-11	4.44E-11	0.00E+00	0.00E+00	2.21E-06	2.27E-05
Cu-64	0.00E+00	5.39E-10	2.90E-10	0.00E+00	1.63E-09	2.59E-06	9.92E-06
Zn-65	1.15E-05	3.06E-05	1.90E-05	0.00E+00	1.93E-05	2.69E-04	4.41E-06
Zn-69	1.81E-11	2.61E-11	2.41E-12	0.00E+00	1.58E-11	3.84E-07	2.75E-06
Zn-69m	4.26E-09	7.28E-09	8.59E-10	0.00E+00	4.22E-09	7.36E-06	2.71E-05
Br-82	0.00E+00	0.00E+00	5.66E-06	0.00E+00	0.00E+00	0.00E+00	0.00E+00
Br-83	0.00E+00	0.00E+00	1.28E-07	0.00E+00	0.00E+00	0.00E+00	0.00E+00
Br-84	0.00E+00	0.00E+00	1.48E-07	0.00E+00	0.00E+00	0.00E+00	0.00E+00
Br-85	0.00E+00	0.00E+00	6.84E-09	0.00E+00	0.00E+00	0.00E+00	0.00E+00
Rb-86	0.00E+00	5.36E-05	3.09E-05	0.00E+00	0.00E+00	0.00E+00	2.16E-06
Rb-88	0.00E+00	1.52E-07	9.90E-08	0.00E+00	0.00E+00	0.00E+00	4.66E-09
Rb-89	0.00E+00	9.33E-08	7.83E-08	0.00E+00	0.00E+00	0.00E+00	5.11E-10
Sr-89	1.62E-04	0.00E+00	4.66E-06	0.00E+00	0.00E+00	5.83E-04	4.52E-05
Sr-90	2.73E-02	0.00E+00	1.74E-03	0.00E+00	0.00E+00	3.99E-03	9.28E-05
Sr-91	3.28E-08	0.00E+00	1.24E-09	0.00E+00	0.00E+00	1.44E-05	4.70E-05
Sr-92	3.54E-09	0.00E+00	1.42E-10	0.00E+00	0.00E+00	6.49E-06	6.55E-05
Y-90	1.11E-06	0.00E+00	2.99E-08	0.00E+00	0.00E+00	7.07E-05	7.24E-05
Y-91m	1.37E-10	0.00E+00	4.98E-12	0.00E+00	0.00E+00	7.60E-07	4.64E-07
Y-91	2.47E-04	0.00E+00	6.59E-06	0.00E+00	0.00E+00	7.10E-04	4.97E-05
Y-92	5.50E-09	0.00E+00	1.57E-10	0.00E+00	0.00E+00	6.46E-06	6.46E-05
Y-93	5.04E-08	0.00E+00	1.38E-09	0.00E+00	0.00E+00	2.01E-05	1.05E-04
Zr-95	5.13E-05	1.13E-05	1.00E-05	0.00E+00	1.61E-05	6.03E-04	1.65E-05
Zr-97	5.07E-08	7.34E-09	4.32E-09	0.00E+00	1.05E-08	3.06E-05	9.49E-05
Nb-95	6.35E-06	2.48E-06	1.77E-06	0.00E+00	2.33E-06	1.66E-04	1.00E-05
Nb-97	1.16E-10	2.08E-11	9.74E-12	0.00E+00	2.31E-11	9.23E-07	7.52E-06
Mo-99	0.00E+00	4.66E-08	1.15E-08	0.00E+00	1.06E-07	3.66E-05	3.42E-05
Tc-99m	4.81E-13	9.41E-13	1.56E-11	0.00E+00	1.37E-11	2.57E-07	1.30E-06
Tc-101	2.19E-14	2.30E-14	2.91E-13	0.00E+00	3.92E-13	1.58E-07	4.41E-09
Ru-103	7.55E-07	0.00E+00	2.90E-07	0.00E+00	1.90E-06	1.79E-04	1.21E-05
Ru-105	4.13E-10	0.00E+00	1.50E-10	0.00E+00	3.63E-10	4.30E-06	2.69E-05
Ru-106	3.68E-05	0.00E+00	4.57E-06	0.00E+00	4.97E-05	3.87E-03	1.16E-04
Ag-110m	4.56E-06	3.08E-06	2.47E-06	0.00E+00	5.74E-06	1.48E-03	2.71E-05

Reformatting/Renumbering Changes Only
 0131v

OFFSITE DOSE CALCULATION/ENVIRONMENTAL MONITORING METHODOLOGIES

Table 7.6 (6 of 8)
 INHALATION DOSE FACTORS
 (mrem/pCi inhaled)

	CHILD							
	bone	liver	t body	thyroid	kidney	lung	gi-lli	
Sb-124	1.55E-05	2.00E-07	5.41E-06	3.41E-08	0.00E+00	8.76E-04	4.43E-05	
Sb-125	2.66E-05	2.05E-07	5.59E-06	2.46E-08	0.00E+00	6.27E-04	1.09E-05	
Sn-125	4.95E-06	9.94E-08	2.95E-07	1.03E-07	0.00E+00	2.43E-04	7.17E-05	R25
Te-125m	1.82E-06	6.29E-07	2.47E-07	5.20E-07	0.00E+00	1.29E-04	9.13E-06	
Te-127m	6.72E-06	2.31E-06	8.16E-07	1.64E-06	1.72E-05	4.00E-04	1.93E-05	
Te-127	7.49E-10	2.57E-10	1.65E-10	5.30E-10	1.91E-09	2.71E-06	1.52E-05	
Te-129m	5.19E-06	1.85E-06	8.22E-07	1.71E-06	1.36E-05	4.76E-04	4.91E-05	
Te-129	2.64E-11	9.45E-12	6.44E-12	1.93E-11	6.94E-11	7.93E-07	6.89E-06	
Te-131m	3.63E-08	1.60E-08	1.37E-08	2.64E-08	1.08E-07	5.56E-05	8.32E-05	
Te-131	5.87E-12	2.28E-12	1.78E-12	4.59E-12	1.59E-11	5.55E-07	3.60E-07	
Te-132	1.30E-07	7.36E-08	7.12E-08	8.58E-08	4.79E-07	1.02E-04	3.72E-05	
I-130	2.21E-06	4.43E-06	2.28E-06	4.99E-04	6.61E-06	0.00E+00	1.38E-06	
I-131	1.30E-05	1.30E-05	7.37E-06	4.39E-03	2.13E-05	0.00E+00	7.68E-07	
I-132	5.72E-07	1.10E-06	5.07E-07	5.23E-05	1.69E-06	0.00E+00	8.65E-07	
I-133	4.48E-06	5.49E-06	2.08E-06	1.04E-03	9.13E-06	0.00E+00	1.48E-06	
I-134	3.17E-07	5.84E-07	2.69E-07	1.37E-05	8.92E-07	0.00E+00	2.58E-07	
I-135	1.33E-06	2.36E-06	1.12E-06	2.14E-04	3.62E-06	0.00E+00	1.20E-06	
Cs-134	1.76E-04	2.74E-04	6.07E-05	0.00E+00	8.93E-05	3.27E-05	1.04E-06	
Cs-136	1.76E-05	4.62E-05	3.14E-05	0.00E+00	2.58E-05	3.93E-06	1.13E-06	
Cs-137	2.45E-04	2.23E-04	3.47E-05	0.00E+00	7.63E-05	2.81E-05	9.78E-07	
Cs-138	1.71E-07	2.27E-07	1.50E-07	0.00E+00	1.68E-07	1.84E-08	7.29E-08	
Ba-139	4.98E-10	2.66E-13	1.45E-11	0.00E+00	2.33E-13	1.56E-06	1.56E-05	
Ba-140	2.00E-05	1.75E-08	1.17E-06	0.00E+00	5.71E-09	4.71E-04	2.75E-05	
Ba-141	5.29E-11	2.95E-14	1.72E-12	0.00E+00	2.56E-14	7.89E-07	7.44E-08	
Ba-142	1.35E-11	9.73E-15	7.54E-13	0.00E+00	7.87E-15	4.44E-07	7.41E-10	
La-140	1.74E-07	6.08E-08	2.04E-08	0.00E+00	0.00E+00	4.94E-05	6.10E-05	
La-142	3.50E-10	1.11E-10	3.49E-11	0.00E+00	0.00E+00	2.35E-06	2.05E-05	
Ce-141	1.06E-05	5.28E-06	7.83E-07	0.00E+00	2.31E-06	1.47E-04	1.53E-05	
Ce-143	9.89E-08	5.37E-08	7.77E-09	0.00E+00	2.26E-08	3.12E-05	3.44E-05	
Ce-144	1.83E-03	5.72E-04	9.77E-05	0.00E+00	3.17E-04	3.23E-03	1.05E-04	
Pr-143	4.99E-06	1.50E-06	2.47E-07	0.00E+00	8.11E-07	1.17E-04	2.63E-05	
Pr-144	1.61E-11	4.99E-12	8.10E-13	0.00E+00	2.64E-12	4.23E-07	5.32E-08	
Nd-147	2.92E-06	2.36E-06	1.84E-07	0.00E+00	1.30E-06	8.87E-05	2.22E-05	
W-187	4.41E-09	2.61E-09	1.17E-09	0.00E+00	0.00E+00	1.11E-05	2.46E-05	
Np-239	1.26E-07	9.04E-09	6.35E-09	0.00E+00	2.63E-08	1.57E-05	1.73E-05	

NOTE: The tritium dose factor for bone is assumed to be equal to the total body dose factor.

Reference:

Regulatory Guide 1.109, Table E-9.

Dose Factors for Co-57, Zn-69m, Br-82, Nb-97, Sn-125, Sb-124 and Sb-125 R25
 are from NUREG-0172 Age Specific Radiation Dose Commitment
Factors for a One Year Chronic Intake, November 1977, Table 8.

OFFSITE DOSE CALCULATION/ENVIRONMENTAL MONITORING METHODOLOGIES

Table 7.6 (7 of 8)
 INHALATION DOSE FACTORS
 (mrem/pCi inhaled)

	bone	liver	t body	INFANT			
				thyroid	kidney	lung	gi-lli
H-3	4.62E-07	4.62E-07	4.62E-07	4.62E-07	4.62E-07	4.62E-07	4.62E-07
C-14	1.89E-05	3.79E-06	3.79E-06	3.79E-06	3.79E-06	3.79E-06	3.79E-06
Na-24	7.54E-06	7.54E-06	7.54E-06	7.54E-06	7.54E-06	7.54E-06	7.54E-06
P-32	1.45E-03	8.03E+05	5.53E-05	0.00E+00	0.00E+00	0.00E+00	1.15E-05
Cr-51	0.00E+00	0.00E+00	6.39E-08	4.11E-08	9.45E-09	9.17E-06	2.55E-07
Mn-54	0.00E+00	1.81E-05	3.56E-06	0.00E+00	3.56E-06	7.14E-04	5.04E-06
Mn-56	0.00E+00	1.10E-09	1.58E-10	0.00E+00	7.86E-10	8.95E-06	5.12E-05
Fe-55	1.41E-05	8.39E-06	2.38E-06	0.00E+00	0.00E+00	6.21E-05	7.82E-07
Fe-59	9.69E-06	1.68E-05	6.77E-06	0.00E+00	0.00E+00	7.25E-04	1.77E-05
Co-57	0.00E+00	4.65E-07	4.58E-07	0.00E+00	0.00E+00	2.71E-04	3.47E-06
Co-58	0.00E+00	8.71E-07	1.30E-06	0.00E+00	0.00E+00	5.55E-04	7.95E-06
Co-60	0.00E+00	5.73E-06	8.41E-06	0.00E+00	0.00E+00	3.22E-03	2.28E-05
Ni-63	2.42E-04	1.46E-05	8.29E-06	0.00E+00	0.00E+00	1.49E-04	1.73E-06
Ni-65	1.71E-09	2.03E-10	8.79E-11	0.00E+00	0.00E+00	5.80E-06	3.58E-05
Cu-64	0.00E+00	1.34E-09	5.53E-10	0.00E+00	2.84E-09	6.64E-06	1.07E-05
Zn-65	1.38E-05	4.47E-05	2.22E-05	0.00E+00	2.32E-05	4.62E-04	3.67E-05
Zn-69	3.85E-11	6.91E-11	5.13E-12	0.00E+00	2.87E-11	1.05E-06	9.44E-06
Zn-69m	8.98E-09	1.84E-08	1.67E-09	0.00E+00	7.45E-09	1.91E-05	2.92E-05
Br-82	0.00E+00	0.00E+00	9.49E-06	0.00E+00	0.00E+00	0.00E+00	0.00E+00
Br-83	0.00E+00	0.00E+00	2.72E-07	0.00E+00	0.00E+00	0.00E+00	0.00E+00
Br-84	0.00E+00	0.00E+00	2.86E-07	0.00E+00	0.00E+00	0.00E+00	0.00E+00
Br-85	0.00E+00	0.00E+00	1.46E-08	0.00E+00	0.00E+00	0.00E+00	0.00E+00
Rb-86	0.00E+00	1.36E-04	6.30E-05	0.00E+00	0.00E+00	0.00E+00	2.17E-06
Rb-88	0.00E+00	3.98E-07	2.05E-07	0.00E+00	0.00E+00	0.00E+00	2.42E-07
Rb-89	0.00E+00	2.29E-07	1.47E-07	0.00E+00	0.00E+00	0.00E+00	4.87E-08
Sr-89	2.84E-04	0.00E+00	8.15E-06	0.00E+00	0.00E+00	1.45E-03	4.57E-05
Sr-90	2.92E-02	0.00E+00	1.85E-03	0.00E+00	0.00E+00	8.03E-03	9.36E-05
Sr-91	6.83E-08	0.00E+00	2.47E-09	0.00E+00	0.00E+00	3.76E-05	5.24E-05
Sr-92	7.50E-09	0.00E+00	2.79E-10	0.00E+00	0.00E+00	1.70E-05	1.00E-04
Y-90	2.35E-06	0.00E+00	6.30E-08	0.00E+00	0.00E+00	1.92E-04	7.43E-05
Y-91m	2.91E-10	0.00E+00	9.90E-12	0.00E+00	0.00E+00	1.99E-06	1.68E-06
Y-91	4.20E-04	0.00E+00	1.12E-05	0.00E+00	0.00E+00	1.75E-03	5.02E-05
Y-92	1.17E-08	0.00E+00	3.29E-10	0.00E+00	0.00E+00	1.75E-05	9.04E-05
Y-93	1.07E-07	0.00E+00	2.91E-09	0.00E+00	0.00E+00	5.46E-05	1.19E-04
Zr-95	8.24E-05	1.99E-05	1.45E-05	0.00E+00	2.22E-05	1.25E-03	1.55E-05
Zr-97	1.07E-07	1.83E-08	8.36E-09	0.00E+00	1.85E-08	7.88E-05	1.00E-04
Nb-95	1.12E-05	4.59E-06	2.70E-06	0.00E+00	3.37E-06	3.42E-04	9.05E-06
Nb-97	2.44E-10	5.21E-11	1.88E-11	0.00E+00	4.07E-11	2.37E-06	1.92E-05
Mo-99	0.00E+00	1.18E-07	2.31E-08	0.00E+00	1.89E-07	9.63E-05	3.48E-05
Tc-99m	9.98E-13	2.06E-12	2.66E-11	0.00E+00	2.22E-11	5.79E-07	1.45E-06
Tc-101	4.65E-14	5.88E-14	5.80E-13	0.00E+00	6.99E-13	4.17E-07	6.03E-07 R29
Ru-103	1.44E-06	0.00E+00	4.85E-07	0.00E+00	3.03E-06	3.94E-04	1.15E-05
Ru-105	8.74E-10	0.00E+00	2.93E-10	0.00E+00	6.42E-10	1.12E-05	3.46E-05
Ru-106	6.20E-05	0.00E+00	7.77E-06	0.00E+00	7.61E-05	8.26E-03	1.17E-04
Ag-110m	7.13E-06	5.16E-06	3.57E-06	0.00E+00	7.80E-06	2.62E-03	2.36E-05

OFFSITE DOSE CALCULATION/ENVIRONMENTAL MONITORING METHODOLOGIES

Table 7.6 (8 of 8)
 INHALATION DOSE FACTORS
 (mrem/pCi inhaled)

	INFANT							gi-lli
	bone	liver	t body	thyroid	kidney	lung		
Sb-124	2.71E-05	3.97E-07	8.56E-06	7.18E-08	0.00E+00	1.89E-03	4.22E-05	
Sb-125	3.69E-05	3.41E-07	7.78E-06	4.45E-08	0.00E+00	1.17E-03	1.05E-05	
Sn-125	1.01E-05	2.51E-07	6.00E-07	2.47E-07	0.00E+00	6.43E-04	7.26E-05	R25
Te-125m	3.40E-06	1.42E-06	4.70E-07	1.16E-06	0.00E+00	3.19E-04	9.22E-06	
Te-127m	1.19E-05	4.93E-06	1.48E-06	3.48E-06	2.68E-05	9.37E-04	1.95E-05	
Te-127	1.59E-09	6.81E-10	3.49E-10	1.32E-09	3.47E-09	7.39E-06	1.74E-05	
Te-129m	1.01E-05	4.35E-06	1.59E-06	3.91E-06	2.27E-05	1.20E-03	4.93E-05	
Te-129	5.63E-11	2.48E-11	1.34E-11	4.82E-11	1.25E-10	2.14E-06	1.88E-05	
Te-131m	7.62E-08	3.93E-08	2.59E-08	6.38E-08	1.89E-07	1.42E-04	8.51E-05	
Te-131	1.24E-11	5.87E-12	3.57E-12	1.13E-11	2.85E-11	1.47E-06	5.87E-06	
Te-132	2.66E-07	1.69E-07	1.26E-07	1.99E-07	7.39E-07	2.43E-04	3.15E-05	
I-130	4.54E-06	9.91E-06	3.98E-06	1.14E-03	1.09E-05	0.00E+00	1.42E-06	
I-131	2.71E-05	3.17E-05	1.40E-05	1.06E-02	3.70E-05	0.00E+00	7.56E-07	
I-132	1.21E-06	2.53E-06	8.99E-07	1.21E-04	2.82E-06	0.00E+00	1.36E-06	
I-133	9.46E-06	1.37E-05	4.00E-06	2.54E-03	1.60E-05	0.00E+00	1.54E-06	
I-134	6.58E-07	1.34E-06	4.75E-07	3.18E-05	1.49E-06	0.00E+00	9.21E-07	
I-135	2.76E-06	5.43E-06	1.98E-06	4.97E-04	6.05E-06	0.00E+00	1.31E-06	
Cs-134	2.83E-04	5.02E-04	5.32E-05	0.00E+00	1.36E-04	5.69E-05	9.53E-07	
Cs-136	3.45E-05	9.61E-05	3.78E-05	0.00E+00	4.03E-05	8.40E-06	1.02E-06	
Cs-137	3.92E-04	4.37E-04	3.25E-05	0.00E+00	1.23E-04	5.09E-05	9.53E-07	
Cs-138	3.61E-07	5.58E-07	2.84E-07	0.00E+00	2.93E-07	4.67E-08	6.26E-07	
Ba-139	1.06E-09	7.03E-13	3.07E-11	0.00E+00	4.23E-13	4.25E-06	3.64E-05	
Ba-140	4.00E-05	4.00E-08	2.07E-06	0.00E+00	9.59E-09	1.14E-03	2.74E-05	
Ba-141	1.12E-10	7.70E-14	3.55E-12	0.00E+00	4.64E-14	2.12E-06	3.39E-06	
Ba-142	2.84E-11	2.36E-14	1.40E-12	0.00E+00	1.36E-14	1.11E-06	4.95E-07	
La-140	3.61E-07	1.43E-07	3.68E-08	0.00E+00	0.00E+00	1.20E-04	6.06E-05	
La-142	7.36E-10	2.69E-10	6.46E-11	0.00E+00	0.00E+00	5.87E-06	4.25E-05	
Ce-141	1.98E-05	1.19E-05	1.42E-06	0.00E+00	3.75E-06	3.69E-04	1.54E-05	
Ce-143	2.09E-07	1.38E-07	1.58E-08	0.00E+00	4.03E-08	8.30E-05	3.55E-05	
Ce-144	2.28E-03	8.65E-04	1.26E-04	0.00E+00	3.84E-04	7.03E-03	1.06E-04	
Pr-143	1.00E-05	3.74E-06	4.99E-07	0.00E+00	1.41E-06	3.09E-04	2.66E-05	
Pr-144	3.42E-11	1.32E-11	1.72E-12	0.00E+00	4.80E-12	1.15E-06	3.06E-06	
Nd-147	5.67E-06	5.81E-06	3.57E-07	0.00E+00	2.25E-06	2.30E-04	2.23E-05	
W-187	9.26E-09	6.44E-09	2.23E-09	0.00E+00	0.00E+00	2.83E-05	2.54E-05	
Np-239	2.65E-07	2.37E-08	1.34E-08	0.00E+00	4.73E-08	4.25E-05	1.78E-05	

NOTE: The tritium dose factor for bone is assumed to be equal to the total body dose factor.

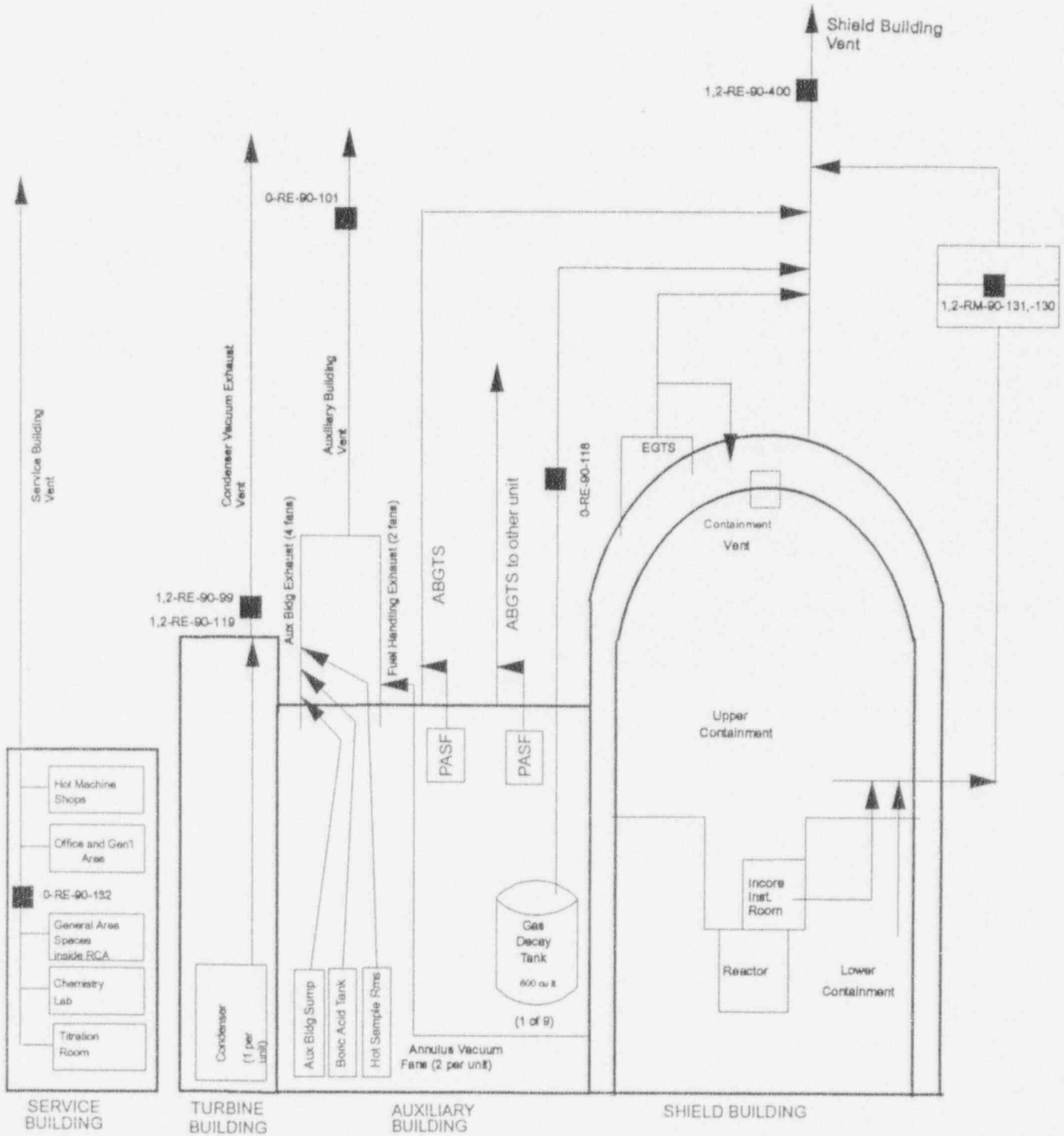
Reference:

Regulatory Guide 1.109, Table E-10.

Dose Factors for Co-57, Zn-69m, Br-82, Nb-97, Sn-125, Sb-124 and Sb-125 R25
 are from NUREG-0172 Age Specific Radiation Dose Commitment
Factors for a One Year Chronic Intake, November 1977, Table 8.

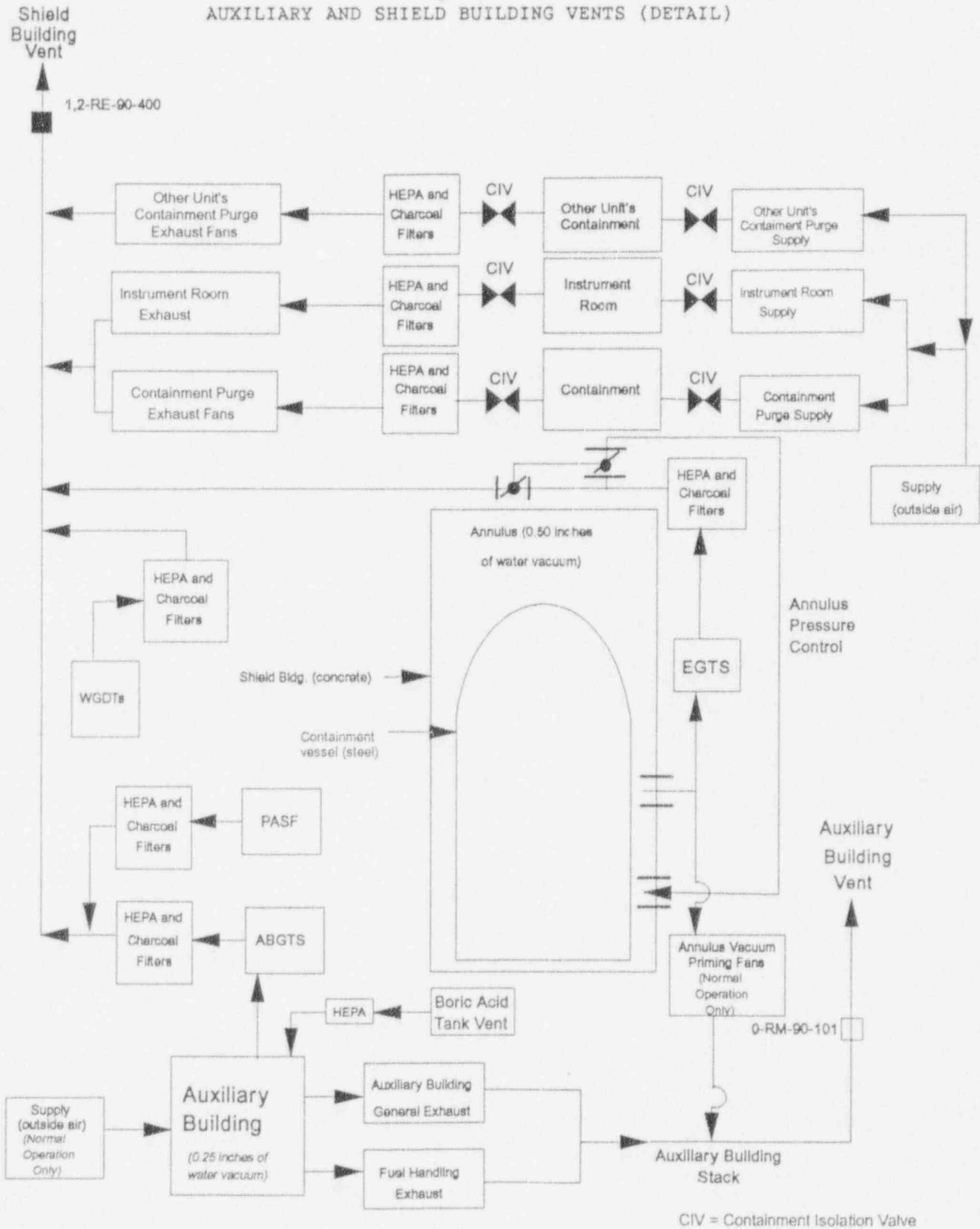
OFFSITE DOSE CALCULATION/ENVIRONMENTAL MONITORING METHODOLOGIES

Figure 7.1
 GASEOUS EFFLUENT RELEASE POINTS



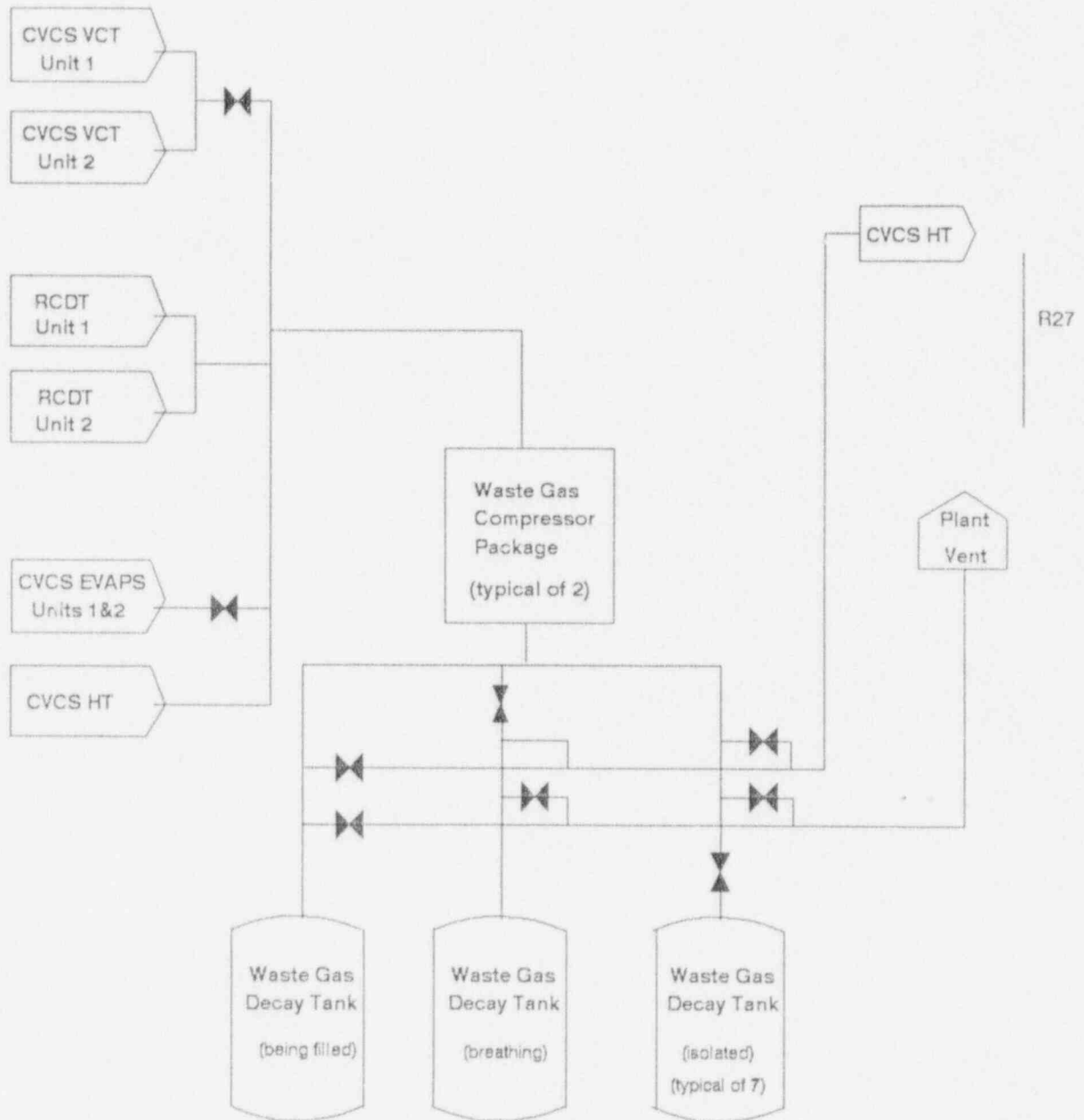
OFFSITE DOSE CALCULATION/ENVIRONMENTAL MONITORING METHODOLOGIES

Figure 7.2
 AUXILIARY AND SHIELD BUILDING VENTS (DETAIL)



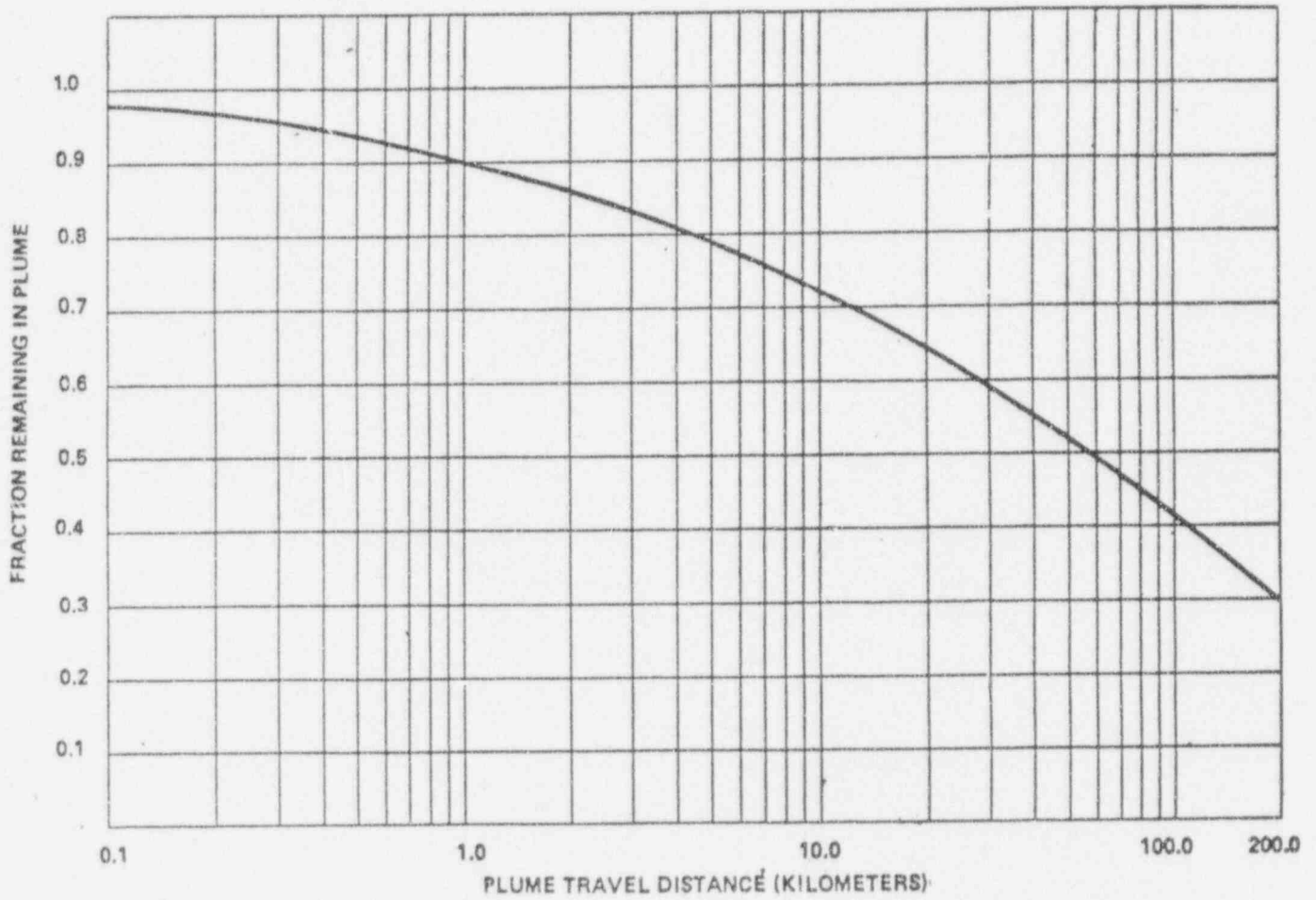
OFFSITE DOSE CALCULATION/ENVIRONMENTAL MONITORING METHODOLOGIES

Figure 7.3
GASEOUS RADWASTE TREATMENT SYSTEM



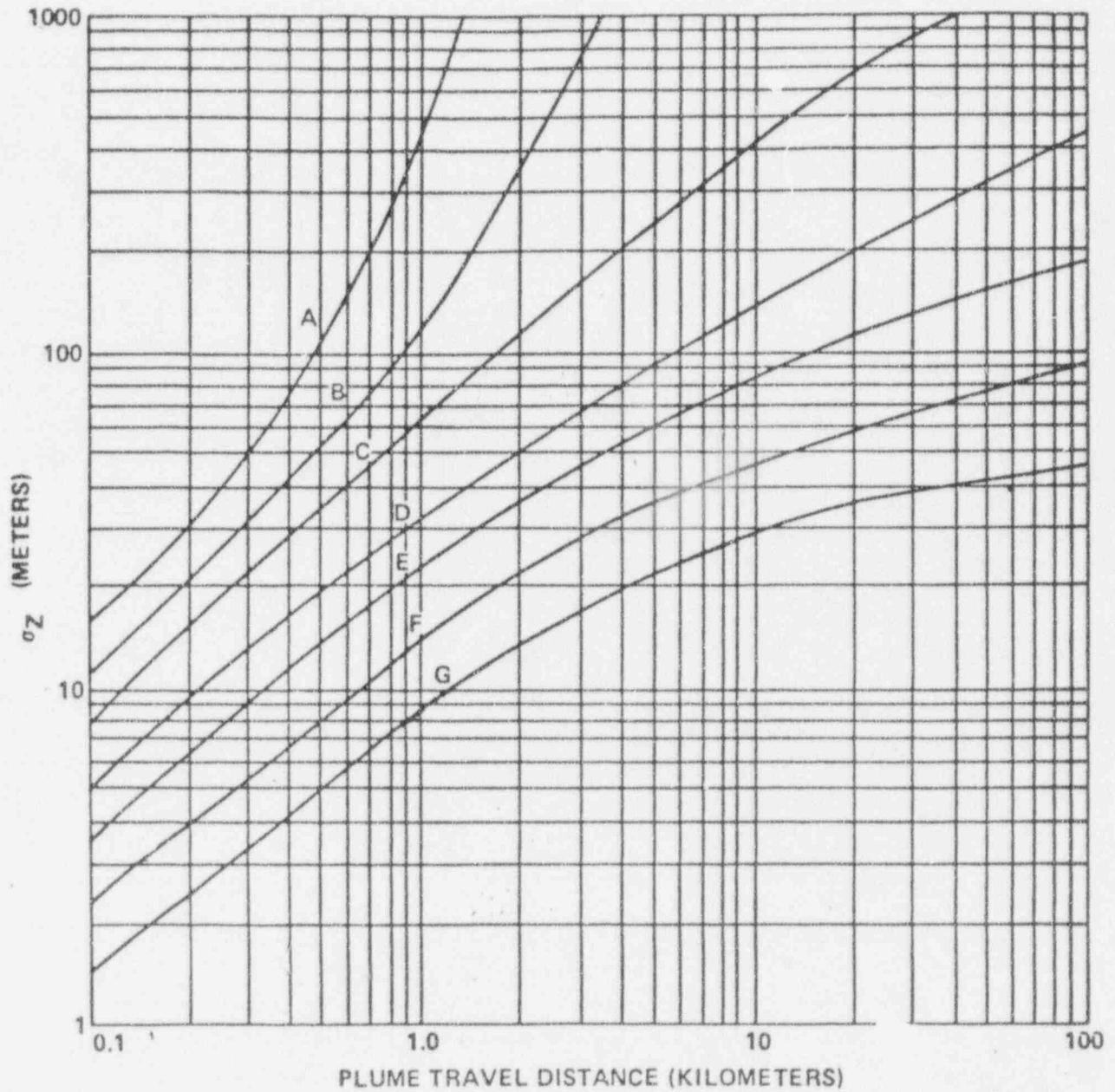
OFFSITE DOSE CALCULATION/ENVIRONMENTAL MONITORING METHODOLOGIES

Figure 7.4
PLUME DEPLETION EFFECT FOR GROUND LEVEL RELEASES
(All Stability Classes)



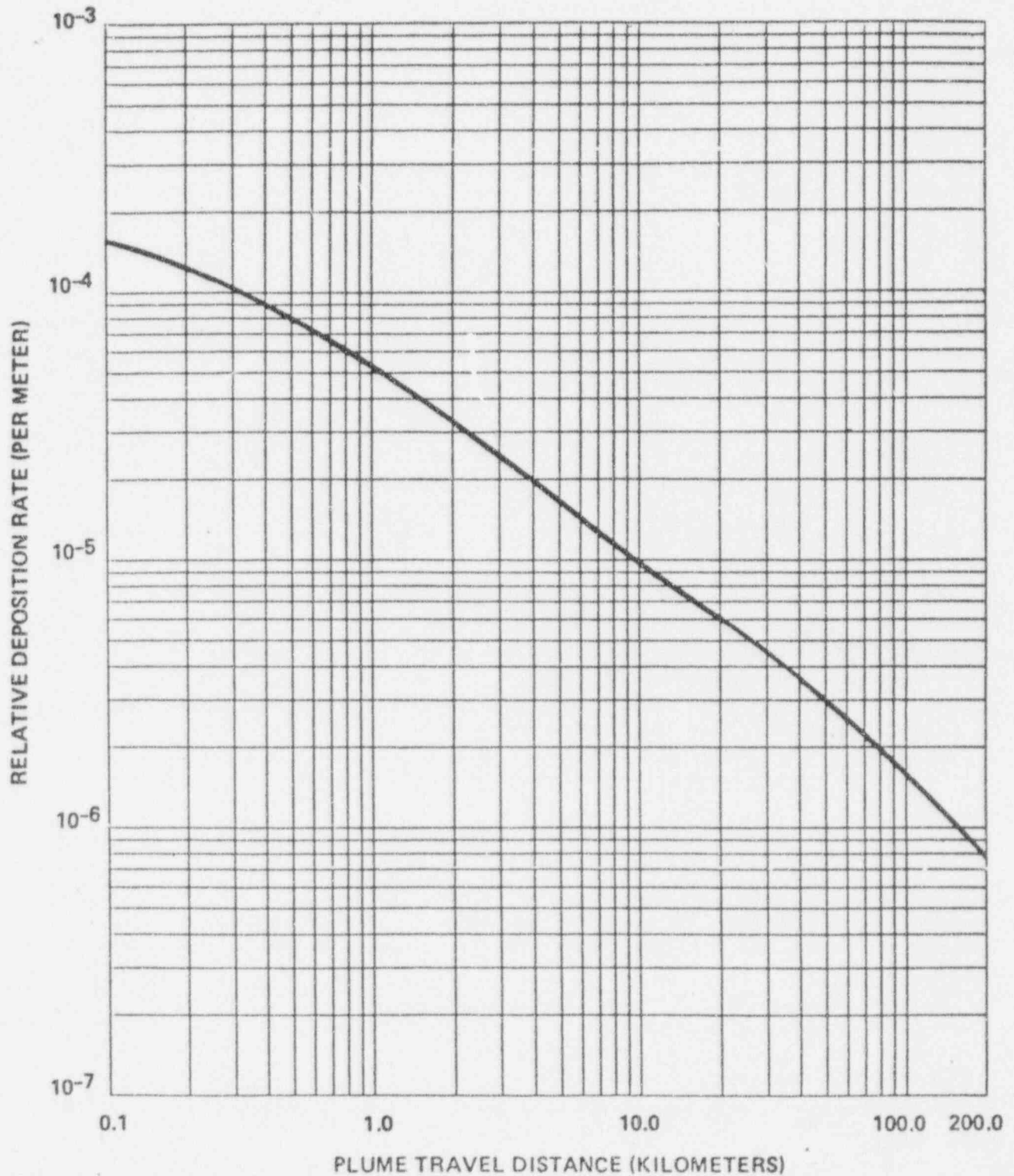
OFFSITE DOSE CALCULATION/ENVIRONMENTAL MONITORING METHODOLOGIES

Figure 7.5
VERTICAL STANDARD DEVIATION OF MATERIAL IN A PLUME



OFFSITE DOSE CALCULATION/ENVIRONMENTAL MONITORING METHODOLOGIES

Figure 7.6
RELATIVE DEPOSITION FOR GROUND LEVEL RELEASES
(All Stability Classes)



OFFSITE DOSE CALCULATION/ENVIRONMENTAL MONITORING METHODOLOGIES

SECTION 8.0

TOTAL DOSE

OFFSITE DOSE CALCULATION/ENVIRONMENTAL MONITORING METHODOLOGIES

8.1 ANNUAL MAXIMUM INDIVIDUAL DOSES - TOTAL REPORTED DOSE

To determine compliance with 40 CFR 190 as required in ODCM Administrative Control 5.2, the annual dose contributions to the maximum individual from SQN radioactive effluents and all other nearby uranium fuel cycle sources will be considered. The annual dose to the maximum individual will be conservatively estimated by first, summing the quarterly total body air submersion dose, the quarterly critical organ dose from gaseous effluents, the quarterly total body dose from liquid effluents, the quarterly critical organ dose from liquid effluents, and the direct radiation monitoring program, and then taking the sum for each quarter and summing over the four quarters.

OFFSITE DOSE CALCULATION/ENVIRONMENTAL MONITORING METHODOLOGIES

SECTION 9.0

RADIOLOGICAL ENVIRONMENTAL MONITORING PROGRAM

OFFSITE DOSE CALCULATION/ENVIRONMENTAL MONITORING METHODOLOGIES

9.1 MONITORING PROGRAM

An environmental radiological monitoring program shall be conducted in accordance with ODCM Control 1.3.1. The monitoring program described in Tables 9.1, 9.2, and 9.3, and in Figures 9.1, 9.2 and 9.3 shall be conducted. Results of this program shall be reported in accordance with ODCM Administrative Control 5.1.

The atmospheric environmental radiological monitoring program shall consist of monitoring stations from which samples of air particulates and atmospheric radioiodine shall be collected.

The terrestrial monitoring program shall consist of the collection of milk, soil, ground water, drinking water, and food crops. In addition, direct gamma radiation levels will be measured in the vicinity of the plant.

The reservoir sampling program shall consist of the collection of samples of surface water, sediment, clams, and fish.

Deviations are permitted from the required sampling schedule if specimens are unobtainable due to hazardous conditions, sample unavailability, or to malfunction of sampling equipment. If the latter, every effort shall be made to complete corrective action prior to the end of the next sampling period.

9.2 DETECTION CAPABILITIES

Analytical techniques shall be such that the detection capabilities listed in Table 2.3-3 are achieved.

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9.3 LAND USE CENSUS

A land use survey shall be conducted in accordance with Control 1.3.2. The results of the survey shall be reported in the Annual Radiological Environmental Operating Report.

9.4 INTERLABORATORY COMPARISON PROGRAM

Analyses shall be performed on radioactive materials supplied as part of an Interlaboratory Comparison Program which has been approved by the NRC. A summary of the results obtained in the intercomparison shall be included in the Annual Radiological Environmental Operating Report (or the EPA program code designation may be provided).

If analyses are not performed as required corrective actions taken to prevent a recurrence shall be reported in the Annual Radiological Environmental Operating Report.

OFFSITE DOSE CALCULATION/ENVIRONMENTAL MONITORING METHODOLOGIES

Table 9.1 (Sheet 1 of 4)
 RADIOLOGICAL ENVIRONMENTAL MONITORING PROGRAM

Exposure Pathway and/or Sample	Sample Locations*	Sampling and Collection Frequency	Type and Frequency of Analysis	
AIRBORNE				
Particulates	4 samples from locations (in different sectors) at or near the site boundary (LM-2,3,4, and 5)	Continuous sampler operation with sample collection once per 7 days (more frequently if required by dust loading)	Analyze for gross beta radioactivity \geq 24 hours following filter change. Perform gamma isotopic analysis on each sample if gross beta > 10 times year ¹ mean of control sample.	
	4 samples from communities approximately 6-10 miles distance from the plant. (PM-2,3,8, and 9)			Composite at least once per 31 days (by location for gamma scan).
	4 samples from control locations greater than 10 miles from the plant (RM-1,2,3, and 4)			
Radioiodine	Samples from same location as air particulates.	Continuous sampler operation with filter collection once per 7 days	I-131 at least once per 7 days	
Soil	Samples from same locations as air particulates	Once per year	Gamma scan, Sr-89, Sr-90 once per year	
DIRECT RADIATION	2 or more dosimeters placed at locations (in different sectors at or near the site boundary in each of the 16 sectors.	Once per 92 days	Gamma dose at least once per 92 days	

* Sample locations are listed in Tables 9.2 and 9.3 and shown on Figures 9.1, 9.2 and 9.3

OFFSITE DOSE CALCULATION/ENVIRONMENTAL MONITORING METHODOLOGIES

Table 9.1 (Sheet 2 of 4)
 RADIOLOGICAL ENVIRONMENTAL MONITORING PROGRAM

Exposure Pathway and/or Sample	Sample Locations*	Sampling and Collection Frequency	Type and Frequency of Analysis
DIRECT RADIATION (continued)	2 or more dosimeters placed at stations located >5 miles from the plant in each of the 16 sectors		
	2 or more dosimeters in at least 8 additional locations of special interest.		
WATERBORNE Surface	TRM 497.0 TRM 483.4 TRM 473.2	Collected by automatic sequential-type sampler** with composite samples collected at least once per 31 days	Gamma scan of each composite sample. Composite for H-3 analysis at least once per 92 days
Ground	1 sample adjacent to plant (location W-6) 1 sample from groundwater source up-gradient	At least once per 92 days	Gross beta and gamma scan, Sr-89 Sr-90 and H-3 analysis at least once per 92 days
Drinking	1 sample at the first potable surface water supply downstream from the plant (TRM 473.0)	Collected by automatic sequential type sampler** with composite sample collected at least once per 31 days	Gross beta and gamma scan of each composite sample. Composite for H-3, Sr-89, Sr-90 at least once per 92 days.

* Sample locations are listed in Tables 9.2 and 9.3 and shown on Figures 9.1, 9.2 and 9.3

** Samples shall be collected by collecting an aliquot at intervals not exceeding 2 hours.

OFFSITE DOSE CALCULATION/ENVIRONMENTAL MONITORING METHODOLOGIES

Table 9.1 (Sheet 3 of 4)
 RADIOLOGICAL ENVIRONMENTAL MONITORING PROGRAM

Exposure Pathway and/or Sample	Sample Locations*	Sampling and Collection Frequency	Type and Frequency of Analysis
WATERBORNE Drinking (continued)	1 sample at the next 2 downstream potable surface water suppliers (greater than 10 miles downstream) (TRM 470.5 and 465.3)	Grab sample once per 31 days	
	2 samples at control locations (TRM 497.0 and 503.8)***	Samples collected by automatic sequential type sampler** with composite sample collected at least once per 31 days.	R25
Sediment	TRM 496.5, 483.4, 480.8, TRM 472.8	At least once per 184 days	Gamma scan of each sample.
Shoreline	TRM 485, TRM 478 TRM 477	At least once per 184 days	Gamma scan of each sample.
INGESTION			
Milk	1 sample from milk producing animals in each of 1-3 areas indicated by the cow census where doses are calculated to be high- If samples are not available from a milk animal location, doses to that area will be estimated by projecting the doses from concentrations detected in milk from other sectors or samples of vegetation will be taken monthly where milk is not available	At least once per 15 days	Gamma isotopic and I-131 analysis of each sample. Sr-89, Sr-90 once per quarter

* Sample locations are listed in Tables 9.2 and 9.3 and shown on Figures 9.1, 9.2 and 9.3

** Samples shall be collected by collecting an aliquot at intervals not exceeding 2 hours.

*** The surface water sample collected at TRM 497.0 is considered a control for the raw drinking water sample.

OFFSITE DOSE CALCULATION/ENVIRONMENTAL MONITORING METHODOLOGIES

Table 9.1 (Sheet 4 of 4)
 RADIOLOGICAL ENVIRONMENTAL MONITORING PROGRAM

Exposure Pathway and/or Sample	Sample Locations*	Sampling and Collection Frequency	Type and Frequency of Analysis	
INGESTION	At least 1 sample			
Milk	from a control			
(continued)	location			
Fish	1 sample each from Chickamauga and Watts Bar Reservoirs	At least once per 184 days. One sample of each of the following species: Channel Catfish, Crappie, Smallmouth Buffalo	Gamma scan on edible portion	R28
Invertebrates (Asiatic Clams)	1 sample downstream from plant discharge** 1 sample upstream from the plant**	At least once per 184 days	Gamma scan on edible portion	R28
Food Products	1 sample each of principal food products grown at private gardens and/or farms in the immediate vicinity of the plant	At least once per 365 days at time of harvest. The types of foods available for sampling will vary. Following is a list of typical foods which may be available: Cabbage and/or Lettuce, Corn, Green Beans, Potatoes, Tomatoes	Gamma scan on edible portion	
Vegetation	Samples from farms producing milk but not providing a milk sample (Farm Em) Control sample from one control dairy farm (Farm S)	At least once per 31 days	I-131 and gamma scan at least once per 31 days. Sr-89, Sr-90 analysis at least once per 92 days.	R25 R25 R25

* Sample locations are listed in Tables 9.2 and 9.3 and shown on Figures 9.1, 9.2 and 9.3

** No permanent stations established. Locations depend on availability of clams.

OFFSITE DOSE CALCULATION/ENVIRONMENTAL MONITORING METHODOLOGIES

Table 9.2 (1 of 2)
 ENVIRONMENTAL RADIOLOGICAL MONITORING PROGRAM SAMPLING LOCATIONS

Map Location Number ^a	Station	Sector	Approximate Indicator (I) Distance or		Samples Collected ^b	
			(Miles)	Control (C)		
2	LM-2	N	0.8	I	AP, CF, S	
3	LM-3	SSW	2.0	I	AP, CF, S	R28
4	LM-4	NE	1.5	I	AP, CF, S	
5	LM-5	NNE	1.8	I	AP, CF, S	
7	PM-2	SW	3.8	I	AP, CF, S	
8	PM-3	W	5.6	I	AP, CF, S	
9	PM-8	SSW	8.7	I	AP, CF, S	
10	PM-9	WSW	2.6	I	AP, CF, S	
11	RM-1	SW	16.7	C	AP, CF, S	
12	RM-2	NNE	17.8	C	AP, CF, S	
13	RM-3	ESE	11.3	C	AP, CF, S	
14	RM-4	WNW	18.9	C	AP, CF, S	
15	Farm B	NE	43.0	C	M	
16	Farm C	NE	16.0	C	M	
17	Farm S	NNE	12.0	C	M, V	R25
18	Farm J	WNW	1.1	I	M	
19	Farm HW	NW	1.2	I	M, W ^c	
20	Farm EM	N	2.6	I	V	
24	Well No. 6	NNE	0.15	I	W	
31	TRM ^d 473.0 (C. F. Industries)	--	11.5 ^e	I	PW	
32	TRM 470.5 (E. I. DuPont)	--	14.0 ^e	I	PW	
33	TRM 465.3 (Chattanooga)	--	19.2 ^e	I	PW	
34	TRM 497.0	--	12.5 ^e	C ^f	SW	
35	TRM 503.8 (Dayton)	--	19.3 ^e	C	PW	
36	TRM 496.5	--	12.0 ^e	C	SD	
37	TRM 485.0	--	0.5 ^e	C	SS	
38	TRM 483.4	--	1.1 ^e	I	SD, SW	
39	TRM 480.8	--	3.7 ^e	I	SD	
40	TRM 477.0	--	7.5 ^e	I	SS	
41	TRM 473.2	--	11.3 ^e	I	SW	
42	TRM 472.8	--	11.7 ^e	I	SD	
44	TRM 478.8	--	6.5 ^e	I	SS	

OFFSITE DOSE CALCULATION/ENVIRONMENTAL MONITORING METHODOLOGIES

Table 9.2 (2 of 2)
 ENVIRONMENTAL RADIOLOGICAL MONITORING PROGRAM SAMPLING LOCATIONS

Map Location Number ^a	Station	Sector	Approximate Distance (Miles)	Indicator (I) or Control (C)	Samples Collected ^b	
						R28
46	TRM 471-530 (Chickamauga Reservoir)	---	--	I/C	F, CL	R25
47	TRM 530-602 (Watts Bar Reservoir)	--	--	C	F	
48	Farm H	NE	4.2	I	M	

^a See figures 9.1, 9.2, and 9.3

R27

^b Sample Codes

- AP = Air particulate filter
- CF = Charcoal filter
- CL = Clams
- F = Fish
- M = Milk
- PW = Public water
- R = Rainwater
- S = Soil
- SD = Sediment
- SS = Shoreline sediment
- SW = Surface water
- V = Vegetation
- W = Well water

^c A control for well water.

^d TRM Tennessee River Mile.

^e Distance from plant discharge (TRM 484.5)

^f Surface water sample also used as a control for public water.

OFFSITE DOSE CALCULATION/ENVIRONMENTAL MONITORING METHODOLOGIES

Table 9.3 (1 of 2)
 THERMOLUMINESCENT DOSIMETRY LOCATIONS

<u>Map Location Number</u>	<u>Station</u>	<u>Sector</u>	<u>Approximate Distance (Miles)</u>	<u>Onsite (On)^a or Offsite (Off)</u>
3	SSW-1C	SSW	2.0	On R28
4	NE-1A	NE	1.5	On
5	NNE-1	NNE	1.8	On
7	SW-2	SW	3.8	Off
8	W-3	W	5.6	Off
9	SSW-3	SSW	8.7	Off
10	WSW-2A	WSW	2.6	Off
11	SW-3	SW	16.7	Off
12	NNE-4	NNE	17.8	Off
13	ESE-3	ESE	11.3	Off
14	WNW-3	WNW	18.9	Off
49	N-1	N	0.6	On
50	N-2	N	2.1	Off
51	N-3	N	5.2	Off
52	N-4	N	10.0	Off
53	NNE-2	NNE	4.5	Off
54	NNE-3	NNE	12.1	Off
55	NE-1	NE	2.4	Off
56	NE-2	NE	4.1	Off
57	ENE-1	ENE	0.4	On
58	ENE-2	ENE	5.1	Off
59	E-1	E	1.2	On
60	E-2	E	5.2	Off
61	ESE-A	ESE	0.3	On R25
62	ESE-1	ESE	1.2	On
63	ESE-2	ESE	4.9	Off
64	SE-A	SE	0.4	On
65	E-A	E	0.3	On R25
66	SE-1	SE	1.4	On
67	SE-2	SE	1.9	On
68	SE-4	SE	5.2	Off
69	SSE-1	SSE	1.6	On
70	SSE-2	SSE	4.6	Off
71	S-1	S	1.5	On
72	S-2	S	4.7	Off
73	SSW-1	SSW	0.6	On
74	SSW-2	SSW	4.0	Off

^aTLDs designated onsite are those located two miles or less from the plant.
 TLDs designated offsite are those located more than two miles from the plant.

OFFSITE DOSE CALCULATION/ENVIRONMENTAL MONITORING METHODOLOGIES

Table 9.3 (2 of 2)
 THERMOLUMINESCENT DOSIMETRY LOCATIONS

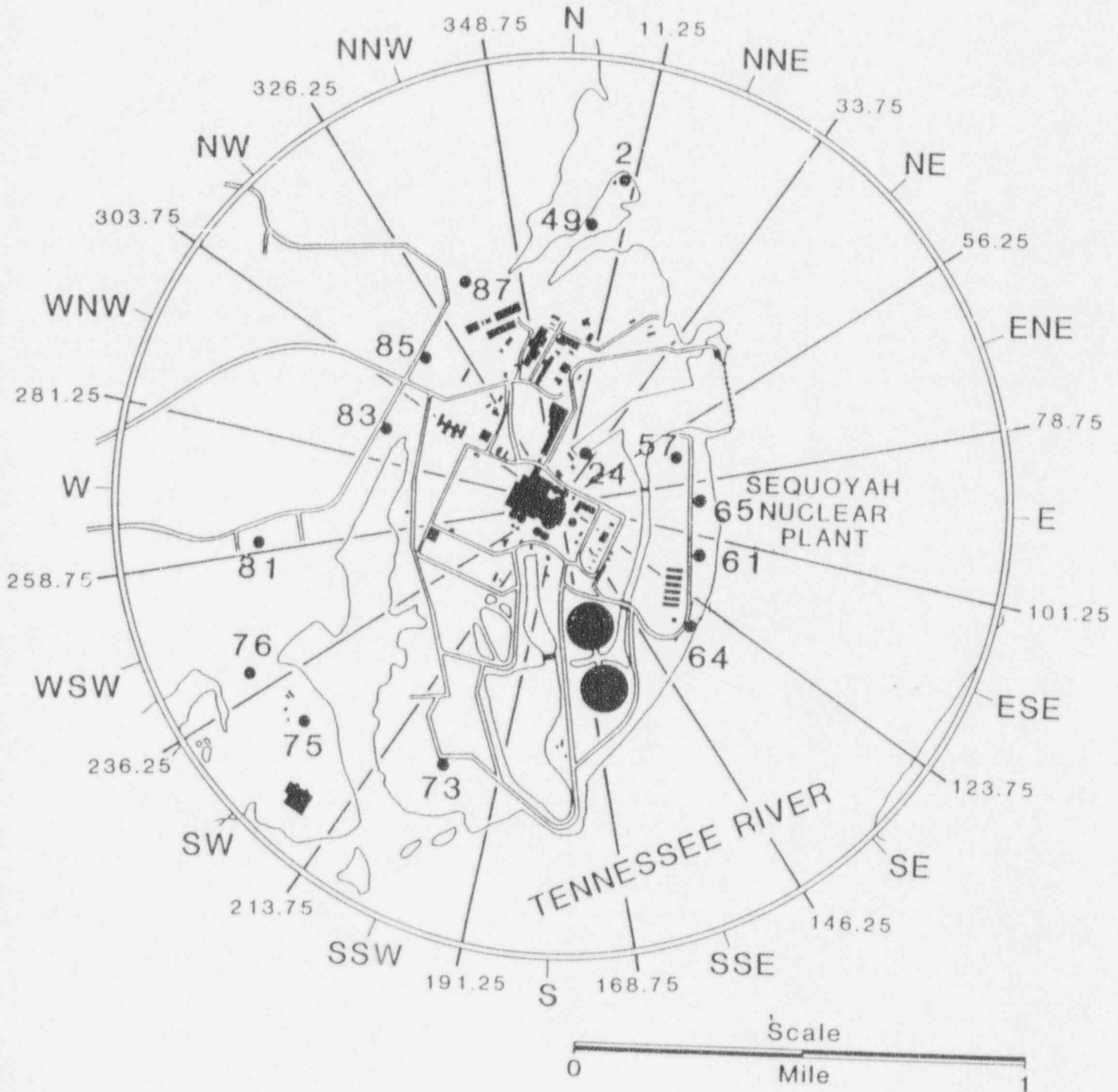
<u>Map Location Number</u>	<u>Station</u>	<u>Sector</u>	<u>Approximate Distance (Miles)</u>	<u>Onsite (On)^a or Offsite (Off)</u>
75	SW-1	SW	0.9	On
76	WSW-1	WSW	0.9	On
77	WSW-2	WSW	2.5	Off
78	WSW-3	WSW	5.7	Off
79	WSW-4	WSW	7.8	Off
80	WSW-5	WSW	10.1	Off
81	W-1	W	0.8	On
82	W-2	W	4.3	Off
83	WNW-1	WNW	0.4	On
84	WNW-2	WNW	5.3	Off
85	NW-1	NW	0.4	On
86	NW-2	NW	5.2	Off
87	NNW-1	NNW	0.6	On
88	NNW-2	NNW	1.7	On
89	NNW-3	NNW	5.3	Off
90	SSW-1B	SSW	1.5	On

R28

^aTLDs designated onsite are those located two miles or less from the plant.
 TLDs designated offsite are those located more than two miles from the
 plant.

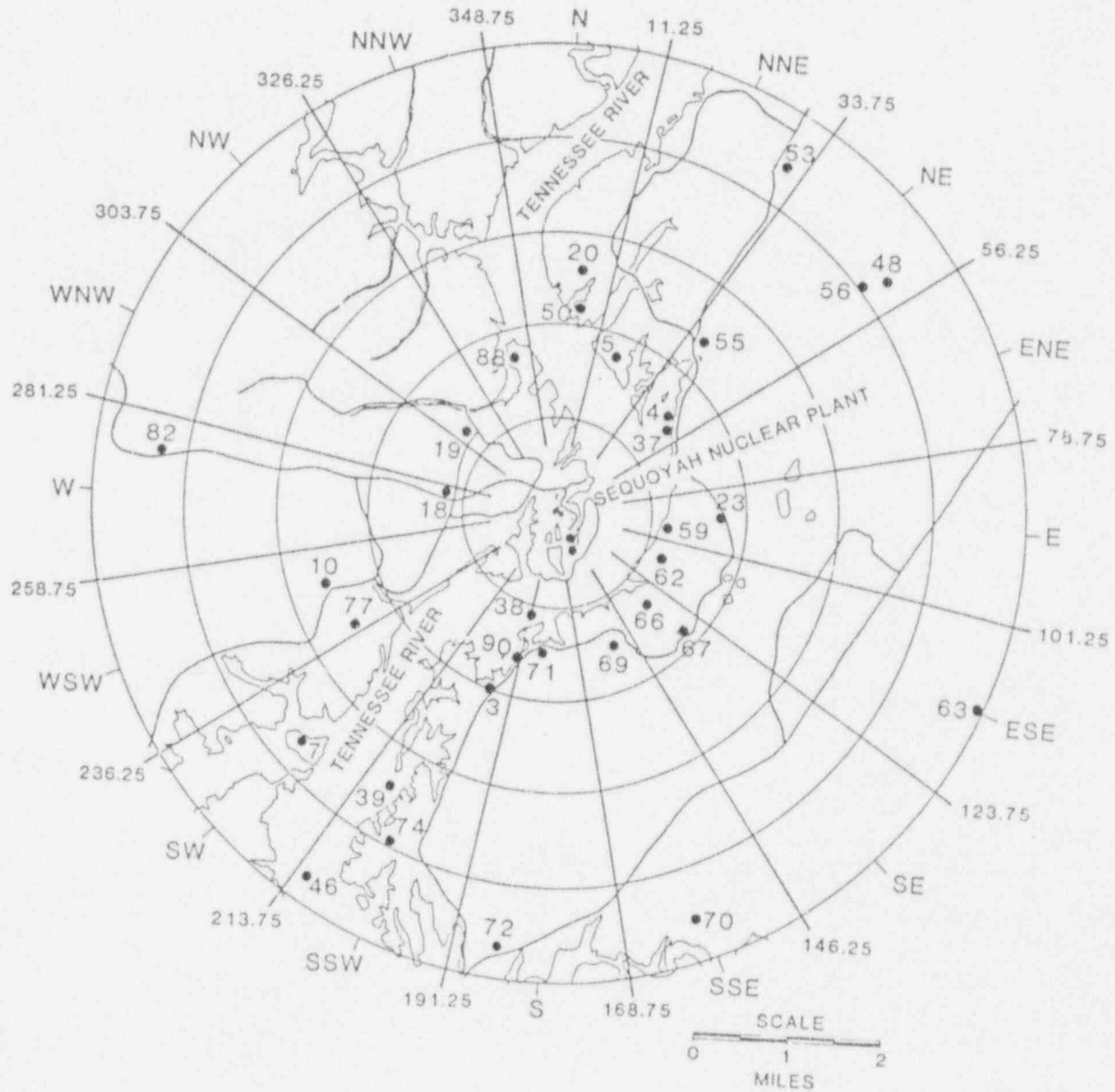
OFFSITE DOSE CALCULATION/ENVIRONMENTAL MONITORING METHODOLOGIES

Figure 9.1
ENVIRONMENTAL MONITORING LOCATIONS
WITHIN ONE MILE OF THE PLANT



OFFSITE DOSE CALCULATION/ENVIRONMENTAL MONITORING METHODOLOGIES

Figure 9.2
ENVIRONMENTAL MONITORING LOCATIONS
FROM ONE TO FIVE MILES FROM THE PLANT



OFFSITE DOSE CALCULATION/ENVIRONMENTAL MONITORING METHODOLOGIES

Figure 9.3
ENVIRONMENTAL MONITORING LOCATIONS
GREATER THAN FIVE MILES FROM THE PLANT

