



UNITED STATES
NUCLEAR REGULATORY COMMISSION

REGION IV

611 RYAN PLAZA DRIVE, SUITE 400
ARLINGTON, TEXAS 76011-8064

APR 25 1994

Dockets: 50-498
50-499
Licenses: NPF-76
NPF-80

Houston Lighting & Power Company
ATTN: William T. Cottle, Group
Vice President, Nuclear
P.O. Box 289
Wadsworth, Texas 77483

SUBJECT: NRC INSPECTION REPORT 50-498/94-09; 50-499/94-09 (NOTICE OF VIOLATION)

This refers to the inspection conducted by Mr. D. P. Loveless, and the inspectors identified in the attached report, of this office on January 30 through March 12, 1994. The inspection included a review of activities authorized for your South Texas Project, Units 1 and 2, facility. At the conclusion of the inspection, the findings were discussed with you and those members of your staff identified in the enclosed report.

Areas examined during the inspection are identified in the report. Within these areas, the inspection consisted of selective examinations of procedures and representative records, interviews with personnel, observation of activities in progress, inspection of plant equipment material condition. In addition to routine observations by the resident inspectors, this inspection included special, augmented inspection coverage during Unit 1 startup and power ascension. This augmented coverage included 24-hour per day inspection of plant activities, such as operator performance in the control room, shift turnover, operator plant tours, independent plant tours, verification of compliance with Technical Specifications, maintenance in progress, prejob briefings, surveillance testing, postmaintenance testing, postmodification testing, and startup testing.

The inspectors found that licensed operator performance in the control room was good, with improvement noted in professionalism, shift turnovers, communications, response to annunciators, and command and control. However, some inconsistency was noted in the areas of professionalism, communications, and self-verification.

The inspectors selected 34 Technical Specification requirements for independent verification. Your staff was found to be properly implementing each of these requirements.

Based on the results of this inspection, certain activities appeared to be in violation of NRC requirements, as specified in Sections 4.6 and 5.9 of the

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enclosed inspection report. The first violation is of concern because the failure to properly control the erection of scaffolding in the plant could result in an adverse interaction with plant safety systems. The second violation is of concern because reactor operators routinely determine which steps in plant procedures are not applicable to a given task. This action, continuing in an uncontrolled manner, violates your Technical Specification requirements to provide for appropriate reviews of procedural changes.

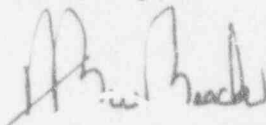
During this inspection period, plant maintenance workers identified that safety-related batteries in control room makeup dampers had failed and that controls were not in place to ensure their continued operability. In addition, operations support personnel identified during a procedure review that the local manual shunt trip for the reactor bypass breaker had not been tested prior to placing the breakers in service as required by Technical Specifications. Also during this period, operations support personnel identified during surveillance scheduling activities that reactor trip breaker and solid state protection logic test procedures had not been tested on a staggered basis as required by Technical Specifications. Although these issues resulted in violations of Technical Specifications, the violations are not being cited because the criteria in paragraph VII.B.1 of Appendix C to 10 CFR Part 2 of the NRC's "Rules of Practice," were satisfied. These violations were identified by your staff and prompt and effective actions were taken to correct the problem.

Additionally during this inspection, observations of Mode 3 testing of the turbine-driven auxiliary feedwater pump and a main feedwater isolation bypass valve were conducted. Based on the results of this and previous inspections, it was determined that these components should support operations of the Unit 1 reactor. Therefore, Restart Issue 1, "Turbine-Driven Auxiliary Feedwater Pump Reliability and Test Methodology," and Restart Issue 14, "Adequacy of the Licensee's Resolution of the Reliability and Operability of the Feedwater Isolation Bypass Valves," identified in NRC Inspection Report 50-498/93-31; 50-499/93-31, are resolved.

In accordance with 10 CFR 2.790 of the NRC's "Rules of Practice," a copy of this letter and its enclosures will be placed in the NRC Public Document Room. The response directed by this letter and enclosed Notice is not subject to the clearance procedures of the Office of Management and Budget, as required by the Paperwork Reduction Act of 1980, PL 96-511.

Should you have any questions concerning this inspection, we will be pleased to discuss them with you.

Sincerely,



A. Bill Beach, Director
Division of Reactor Projects

Enclosures:

1. Appendix A - Notice of Violation
2. Appendix B - NRC Inspection Report
50-498/94-09; 50-499/94-09

cc w/enclosures:

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E-Mail report to D. Sullivan (DJS)

bcc to DMB (IE01)

bcc distrib. by RIV:

L. J. Callan
Branch Chief (DRP/A)

MIS System

RIV File

R. Bachmann, OGC, MS: 15-B-18

A. Beach, D:DRP

Resident Inspector

Lisa Shea, RM/ALF, MS: MNBB 4503

DRSS-FIPB

Project Engineer (DRP/A)

Branch Chief (DRP/TSS)

RIV: <i>J</i> SRI: DRP/A	DRP/A <i>J</i>	C: DRP/A <i>J</i>	D: DRP	
DPLoveless	WBJones	WDJohnson	ABBeach	
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