

RAR-90-B6

December 3, 1990

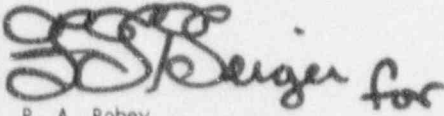
U. S. Nuclear Regulatory Commission
ATTN: Document Control Desk
Washington, D. C. 20555

SUBJECT: Quad Cities Nuclear Station Units 1 and 2
Monthly Performance Report
NRC Docket Nos. 50-254 and 50-265

Enclosed for your information is the Monthly Performance Report covering the operation of Quad-Cities Nuclear Power Station, Units One and Two, during the month of November, 1990.

Respectfully,

COMMONWEALTH EDISON COMPANY
QUAD-CITIES NUCLEAR POWER STATION


R. A. Robey
Technical Superintendent

RAR/LFD/klm

Enclosure

cc: A. B. Davis, Regional Administrator
T. Taylor, Senior Resident Inspector

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QUAD-CITIES NUCLEAR POWER STATION

UNITS 1 AND 2

MONTHLY PERFORMANCE REPORT

NOVEMBER, 1990

COMMONWEALTH EDISON COMPANY

AND

IOWA-ILLINOIS GAS & ELECTRIC COMPANY

NRC DOCKET NOS. 50-254 AND 50-265

LICENSE NOS. DPR-29 AND DPR-30

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1. INTRODUCTION

Quad-Cities Nuclear Power Station is composed of two Boiling Water Reactors, each with a Maximum Dependable Capacity of 769 MWe Net, located in Cordova, Illinois. The Station is jointly owned by Commonwealth Edison Company and Iowa-Illinois Gas & Electric Company. The Nuclear Steam Supply Systems are General Electric Company Boiling Water Reactors. The Architect/Engineer was Sargent & Lundy, Incorporated, and the primary construction contractor was United Engineers & Constructors. The Mississippi River is the condenser cooling water source. The plant is subject to license numbers DPR-29 and DPR-30, issued October 1, 1971, and March 21, 1972, respectively; pursuant to Docket Numbers 50-254 and 50-265. The date of initial Reactor criticalities for Units One and Two, respectively were October 18, 1971, and April 26, 1972. Commercial generation of power began on February 18, 1973 for Unit One and March 10, 1973 for Unit Two.

This report was compiled by Lynne Hamilton and Karen McDearmon, telephone number 309-654-2241, extensions 2185 and 2240.

II. SUMMARY OF OPERATING EXPERIENCE

A. Unit One

Unit One began the month of November operating in Economic Generation Control (EGC). From November 1 through November 11, the unit remained in EGC or operated near full power with brief interruptions to perform routine surveillances.

On November 11 at 1900 hours, normal unit shutdown commenced. At 0248 hours on November 12, the main turbine was tripped, and at 0315 hours, Unit One Reactor was manually scrammed to begin the End of Cycle Eleven Refueling Outage. Normal refueling activities were performed including the removal of the reactor head and steam dryer and core unloading. Core unloading was completed on November 26 at 1110 hours. Normal refueling activities continued throughout the remainder of the month.

B. Unit Two

Unit Two began the month of November operating in Economic Generation Control (EGC). Normal operational activities were performed for the month of November for Unit Two. The unit remained in EGC or operated near full power with minor interruptions to perform routine surveillances. Power levels were adjusted accordingly.

III. PLANT OR PROCEDURE CHANGES, TESTS, EXPERIMENTS,
AND SAFETY RELATED MAINTENANCE

A. Amendments to Facility License or Technical Specifications

Technical Specification Amendment No. 127 was issued on November 6, 1990 to Facility Operating License DWR-29 for Quad Cities Nuclear Power Station. This amendment revises the Technical Specifications to reflect the use of generically approved fuel type GE BxBNB by changing the Minimum Critical Power Ratio (MCPR) safety limit from 1.07 to 1.06.

B. Facility or Procedure Changes Requiring NRC Approval

There were no Facility or Procedure changes requiring NRC approval for the reporting period.

C. Tests and Experiments Requiring NRC Approval

There were no Tests or Experiments requiring NRC approval for the reporting period.

D. Corrective Maintenance of Safety Related Equipment

The following represents a tabular summary of the major safety related maintenance performed on Units One and Two during the reporting period. This summary includes the following: Work Request Numbers, Licensee Event Report Numbers, Components, Cause of Malfunctions, Results and Effects on Safe Operation, and Action Taken to Prevent Repetition.

UNIT 1 MAINTENANCE SUMMARY

<u>WORK REQUEST</u>	<u>SYSTEM</u>	<u>EID DESCRIPTION</u>	<u>WORK PERFORMED</u>
Q87992	7506	Damper, 1/2A SBT Fan Backdraft	As found: Damper counter weight had come loose from damper shaft. The set screws on the counter weight were loose and the tack weld between the counter weight and the shaft had broken. As left: Removed counter weight from shaft, tapped chased set screw holes, inspected louvers for damage and proper operation, re-installed counter weight, installed manual positioning arm, closed access door and resealed with mastik and tape.
Q86931	0756	LPRM 24-09	As found: LPRM bypassed and spiking high. As left: checked LPRM card, high voltage power supply and TDR detector were good. Ran I/V curves as per QIP 756-1. Left card in the bypass position.

UNIT 2 MAINTENANCE SUMMARY

<u>WORK REQUEST</u>	<u>SYSTEM</u>	<u>EID DESCRIPTION</u>	<u>WORK PERFORMED</u>
Q84822	0756	LPRM 32-25A	As found: LPRM was spiking. As left: Exchanged LPRM card with same type LPRM card, for test. Found no problem with original LPRM card and exchanged back with test card. Cleaned connectors.

IV. LICENSEE EVENT REPORTS

The following is a tabular summary of all licensee event reports for Quad-Cities Units One and Two occurring during the reporting period, pursuant to the reportable occurrence reporting requirements as set forth in sections 6.6.B.1. and 6.6.B.2. of the Technical Specifications.

<u>Licensee Event Report Number</u>	<u>Date</u>	<u>Title of Occurrence</u>
	<u>UNIT 1</u>	
90-23 *	10/31/90	Back Leakage through 1A Core Spray Floor Drain
90-27	11/4/90	Failure of CMC Auto Demand Mode
90-28	11/2/90	Spurious Closure of N2 Make-up Valve 1-1601-57
90-29	11/15/90	1-220-628 (outboard) Feedwater Check Valve Failed LLRT
90-30	11/8/90	Recirc Piping Outside Seismic Design Analysis
90-31	11/20/90	HPCI Temperature Switch Calibration Found High
	<u>UNIT 2</u>	
90-12	11/24/90	U-2 HPCI INOP Due to Flow Controller Malfunction

* 90-23 report for the previous month of October has been cancelled.

V. DATA TABULATIONS

The following data tabulations are presented in this report:

- A. Operating Data Report
- B. Average Daily Unit Power Level
- C. Unit Shutdowns and Power Reductions

**APPENDIX D
UNIT SHUTDOWNS AND POWER REDUCTIONS**

DOCKET NO. 50-254

UNIT NAME Quad Cities Unit One

DATE December 3, 1990

REPORT MONTH November, 1990

COMPLETED BY Lynne Hamilton

TELEPHONE 309-654-2241

NO.	DATE	TYPE M OR S	DURATION (HOURS)	REASON	METHOD OF SHUTTING DOWN REACTOR	LICENSEE EVENT REPORT NO.	SYSTEM CODE	COMPONENT CODE	CORRECTIVE ACTIONS/COMMENTS
90-11	901112	S	452.2	C	2	-----	RC	FUELXX	Unit One Shutdown for End of Cycle Eleven Refuel/sg Outage

DOCKET NO. 50-265

APPENDIX D
UNIT SHUTDOWNS AND POWER REDUCTIONS

UNIT NAME Quad Cities Unit Two

COMPLETED BY Lynne Hamilton

DATE December 3, 1990

REPORT MONTH November, 1990

TELEPHONE 309-654-2241

NO.	DATE	TYPE F OR S	DURATION (HOURS)	REASON	METHOD OF SHUTTING DOWN REACTOR	LICENSEE EVENT REPORT NO.	SYSTEM CODE	COMPONENT CODE	CORRECTIVE ACTIONS/COMMENTS
									None

VI. UNIQUE REPORTING REQUIREMENTS

The following items are included in this report based on prior commitments to the commission:

A. Main Steam Relief Valve Operations

There were no Main Steam Relief Valve Operations for the reporting period.

B. Control Rod Drive Scram Timing Data for Units One and Two

There were no Control Rod Drive scram timing data for Units One and Two for the reporting period.

VII. REFUELING INFORMATION

The following information about future reloads at Quad-Cities Station was requested in a January 26, 1978, licensing memorandum (78-24) from D. E. O'Brien to C. Reed, et al., titled "Dresden, Quad-Cities and Zion Station--NRC Request for Refueling Information", dated January 18, 1978.

QUAD CITIES REFUELING
INFORMATION REQUEST

QTP 300-S32
Revision 2
October 1989

1. Unit: Q1 Reload: 10 Cycle: 11
2. Scheduled date for next refueling shutdown: 11-12-90
3. Scheduled date for restart following refueling: 1-28-91
4. Will refueling or resumption of operation thereafter require a Technical Specification change or other license amendment:
Yes, a proposed change to Technical Specification will be made to relax the Minimum Critical Power Ratio (MCPR) safety limit. This proposal is based on the Unit One Reload 11 Cycle 12 fuel loading.
5. Scheduled date(s) for submitting proposed licensing action and supporting information:

AUGUST 31, 1990
6. Important licensing considerations associated with refueling, e.g., new or different fuel design or supplier, unreviewed design or performance analysis methods, significant changes in fuel design, new operating procedures:

NONE AT PRESENT TIME.
7. The number of fuel assemblies.
 - a. Number of assemblies in core: 724
 - b. Number of assemblies in spent fuel pool: 1537
8. The present licensed spent fuel pool storage capacity and the size of any increase in licensed storage capacity that has been requested or is planned in number of fuel assemblies:
 - a. Licensed storage capacity for spent fuel: 3657
 - b. Planned increase in licensed storage: 0
9. The projected date of the last refueling that can be discharged to the spent fuel pool assuming the present licensed capacity: 2008
* 144 new fuel bundles have arrived on site and will be stored in the Unit One new fuel storage vault.

QUAD CITIES REFUELING
INFORMATION REQUEST

1. Unit: Q2 Reload: 10 Cycle: 11
2. Scheduled date for next refueling shutdown: 9-7-91
3. Scheduled date for restart following refueling: 12-9-91
4. Will refueling or resumption of operation thereafter require a Technical Specification change or other license amendment:
NOT AS YET DETERMINED.
5. Scheduled date(s) for submitting proposed licensing action and supporting information:
NOT AS YET DETERMINED.
6. Important licensing considerations associated with refueling, e.g., new or different fuel design or supplier, unreviewed design or performance analysis methods, significant changes in fuel design, new operating procedures:
NONE AT PRESENT TIME.
7. The number of fuel assemblies.
 - a. Number of assemblies in core: 724
 - b. Number of assemblies in spent fuel pool: 2011
8. The present licensed spent fuel pool storage capacity and the size of any increase in licensed storage capacity that has been requested or is planned in number of fuel assemblies:
 - a. Licensed storage capacity for spent fuel: 3897
 - b. Planned increase in licensed storage: 0
9. The projected date of the last refueling that can be discharged to the spent fuel pool assuming the present licensed capacity: 2008

VIII. GLOSSARY

The following abbreviations which may have been used in the Monthly Report, are defined below:

ACAD/CAM	- Atmospheric Containment Atmospheric Dilution/Containment Atmospheric Monitoring
ANSI	- American National Standards Institute
APRM	- Average Power Range Monitor
ATWS	- Anticipated Transient Without Scram
BWR	- Boiling Water Reactor
CRD	- Control Rod Drive
EHC	- Electro-Hydraulic Control System
EOF	- Emergency Operations Facility
GSEP	- Generating Stations Emergency Plan
HEPA	- High-Efficiency Particulate Filter
HPCI	- High Pressure Coolant Injection System
HRSS	- High Radiation Sampling System
IPCLRT	- Integrated Primary Containment Leak Rate Test
IRM	- Intermediate Range Monitor
ISI	- Inservice Inspection
LER	- Licensee Event Report
LLRT	- Local Leak Rate Test
LPCI	- Low Pressure Coolant Injection Mode of RHRs
LPRM	- Local Power Range Monitor
MAPLHGR	- Maximum Average Planar Linear Heat Generation Rate
MCPR	- Minimum Critical Power Ratio
MFLCPR	- Maximum Fraction Limiting Critical Power Ratio
MPC	- Maximum Permissible Concentration
MSIV	- Main Steam Isolation Valve
NIOSH	- National Institute for Occupational Safety and Health
PCI	- Primary Containment Isolation
PCIDMR	- Preconditioning Interim Operating Management Recommendations
RBCCW	- Reactor Building Closed Cooling Water System
RBM	- Rod Block Monitor
RCIC	- Reactor Core Isolation Cooling System
RHRS	- Residual Heat Removal System
RPS	- Reactor Protection System
RWM	- Rod Worth Minimizer
SBGTS	- Standby Gas Treatment System
SBLC	- Standby Liquid Control
SDC	- Shutdown Cooling Mode of RHRS
SDV	- Scram Discharge Volume
SRM	- Source Range Monitor
TBCCW	- Turbine Building Closed Cooling Water System
TIP	- Traversing Incore Probe
TSC	- Technical Support Center