

LICENSEE EVENT REPORT

CONTROL BLOCK: _____ (1) (PLEASE PRINT OR TYPE ALL REQUIRED INFORMATION)

0 1 M E M Y P 1 2 0 0 - 0 0 0 0 0 - 0 0 3 4 1 1 1 1 1 4 _____ 5
7 8 9 LICENSEE CODE 14 15 LICENSE NUMBER 25 26 LICENSE TYPE 30 57 CAT 58

CON'T
0 1 REPORT SOURCE L 6 0 5 0 0 0 3 0 9 7 0 9 2 4 8 2 8 1 1 0 1 1 8 8 2 9
7 8 60 61 DOCKET NUMBER 68 69 EVENT DATE 74 75 REPORT DATE 80

EVENT DESCRIPTION AND PROBABLE CONSEQUENCES (10)

0 2 | While shutting down the reactor in preparation for refueling, six of the eighteen
0 3 | steam generator safety valves were tested as required by Technical Specifications.
0 4 | A pneumatically operated lift assisting device was used to attain the required
0 5 | equivalent steam lifting pressure. One of the valves did not operate with an
0 6 | equivalent lift pressure of 1105 psig. Two of the valves operated at 1075 and
0 7 | 1090 psig respectively. The remaining three valves operated at pressures higher
0 8 | than their previous setting but within the procedure established minimum
7 8 9 80

0 9 C C 11 E 12 B 13 V A L V E X 14 J 15 B 16
7 8 9 10 11 12 13 14 15 16 17 18 19 20

17 LER/RO REPORT NUMBER 18 2 21 22 23 24 2 6 26 27 28 0 3 29 30 L 31 32 0
EVENT YEAR SEQUENTIAL REPORT NO. OCCURRENCE CODE REPORT TYPE REVISION NO.

ACTION TAKEN 18 E 19 Z 20 Z 21 Z 22 0 0 0 0 23 Y 24 Y 25 A 26 C 7 1 0 47
FUTURE ACTION EFFECT ON PLANT SHUTDOWN METHOD HOURS ATTACHMENT SUBMITTED NPRD-4 FORM SUB. PRIME COMP. SUPPLIER COMPONENT MANUFACTURER

CAUSE DESCRIPTION AND CORRECTIVE ACTIONS (27)

1 0 | A review of our records does not indicate why valve set points drifted higher
1 1 | since the previous setting. This has not been a previous experience.
1 2 | To assure that the twelve remaining valves are set correctly, they will all be
1 3 | tested prior to plant operation following the current refueling outage.
1 4 | _____
7 8 9 80

1 5 D 28 0 0 0 0 29 NA 30 B 31 Surveillance Test 32
7 8 9 10 11 12 13 14 15 16 17 18 19 20 21 22 23 24 25 26 27 28 29 30 31 32 33 34 35 36 37 38 39 40 41 42 43 44 45 46 47 48 49 50

1 6 Z 33 Z 34 NA 35 NA 36
7 8 9 10 11 12 13 14 15 16 17 18 19 20 21 22 23 24 25 26 27 28 29 30 31 32 33 34 35 36 37 38 39 40 41 42 43 44 45 46 47 48 49 50

1 7 0 0 0 37 Z 38 NA 39
7 8 9 10 11 12 13 14 15 16 17 18 19 20 21 22 23 24 25 26 27 28 29 30 31 32 33 34 35 36 37 38 39 40 41 42 43 44 45 46 47 48 49 50

1 8 0 0 0 40 NA 41
7 8 9 10 11 12 13 14 15 16 17 18 19 20 21 22 23 24 25 26 27 28 29 30 31 32 33 34 35 36 37 38 39 40 41 42 43 44 45 46 47 48 49 50

1 9 Z 42 NA 43
7 8 9 10 11 12 13 14 15 16 17 18 19 20 21 22 23 24 25 26 27 28 29 30 31 32 33 34 35 36 37 38 39 40 41 42 43 44 45 46 47 48 49 50

2 0 N 44 NA 45
7 8 9 10 11 12 13 14 15 16 17 18 19 20 21 22 23 24 25 26 27 28 29 30 31 32 33 34 35 36 37 38 39 40 41 42 43 44 45 46 47 48 49 50

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NRC USE ONLY

EVENT DESCRIPTION AND PROBABLE CONSEQUENCES (Cont'd)

acceptable lift pressure of 1055 psig. All six valves were reset to within 1% of their design lift set pressure.

The Maine Yankee steam generators were constructed in accordance with the ASME Code Section 3 and designed for 985 psig. The code permits pressures to attain 110% (1083.5 psig) during transients. The vessels were also designed to accept 10 lifetime cycles of hydrostatic testing to 1235 psig. The Maine Yankee Technical Specifications require that 15 of the 18 valves be operable.

With no credit taken for pressure relieving devices or negative moderator temperature coefficient, extrapolation of the safety valve test results to project eighteen valve performance indicates that, during the last cycle, had the plant experienced a fuel power load rejection, without a reactor trip, steam generator pressure could have exceeded the code permitted 110%. However, the steam generators would not have been overstressed and the health and safety of the public would not have been affected.