



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

GPU NUCLEAR CORPORATION

AND

JERSEY CENTRAL POWER & LIGHT COMPANY

DOCKET NO. 50-219

OYSTER CREEK NUCLEAR GENERATING STATION

AMENDMENT TO AMENDED PROVISIONAL OPERATING LICENSE

Amendment No. 63
License No. DPR-16

1. The Nuclear Regulatory Commission (the Commission) has found that:
 - A. The application for amendment by GPU Nuclear Corporation and Jersey Central Power and Light Company (the licensees) dated March 4, 1981 complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations set forth in 10 CFR Chapter I;
 - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
 - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
 - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
 - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.


2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment and Paragraph 2.C(2) of Provisional Operating License No. DPR-16 is hereby amended to read as follows:

(2) Technical Specifications

The Technical Specifications contained in Appendices A and B, as revised through Amendment No. 63, are hereby incorporated in the license. GPU Nuclear Corporation shall operate the facility in accordance with the Technical Specifications.

3. This license amendment is effective as of the date of its issuance.

FOR THE NUCLEAR REGULATORY COMMISSION


Dennis M. Crutchfield, Chief
Operating Reactors Branch #5
Division of Licensing

Attachment:
Changes to the Technical
Specifications

Date of Issuance: October 15, 1982

ATTACHMENT TO LICENSE AMENDMENT NO. 63
PROVISIONAL OPERATING LICENSE NO. DPR-16
DOCKET NO. 50-219

Replace the following pages of the Appendix "A" Technical Specifications with the enclosed pages. The revised pages are identified by the captioned amendment number and contain vertical lines indicating the area of change.

Pages

3.1-7

3.1-11

3.1-14

4.1-6*

4.1-6a

4.2-1a

*There are no changes to the provisions contained thereon; Items 19 - 22 have been moved to page 4.1-6a.

TABLE 3.1.1 PROTECTIVE INSTRUMENTATION REQUIREMENTS

Function	Trip Setting	Reactor Modes in which Function Must Be Operable				Min. No. of Operable or Operating (Tripped) Trip Systems	Min. No. of Operable Instrument Channels Per Operable Trip Systems	Action Required*
		Shutdown	Refuel	Startup	Run			
A. Scram								
1. Manual Scram		X	X	X	X	2	1	Insert control rods
2. High Reactor Pressure	**		X(s)	X	X	2	2	
3. High Drywell Pressure	≤ 2 psig		X(u)	X(u)	X	2	2	
4. Low Reactor Water Level	**		X	X	X	2	2	
5. High Water Level in Scram Discharge Volume	≤ 37 gal.		X(a)	X(z)	X(z)	2	2	
6. Low Condenser Vacuum	≥ 23 " Hg		X(b)	X(b)	X	2	2	
7. High Radiation in Main Steamline Tunnel	≤ 10 x normal background		X(s)	X	X	2	2	
8. Average Power Range Monitor (APRM)	**		X(c,s)	X(c)	X(c)	2	3	
9. Intermediate Range Monitor (IRM)	**		X(d)	X(d)		2	3	

TABLE 3.1.1 PROTECTIVE INSTRUMENTATION REQUIREMENTS (CONTD)

Function	Trip Setting	Reactor Modes in Which Function Must be Operable				Min.No. of Operable or Operating (Tripped) Trip Systems	Min.No. of Operable Instrument Channels Per Operable Trip Systems	Action Required*
		Shutdown	Refuel	Startup	Run			
K. <u>Rod Block</u>								No control rod withdrawals permitted
1. SRM Upscale	5×10^5 cp ⁵	X		X(1)		1	3(y)	
2. SRM Downscale	100 cp ^{5(f)}	X		X(1)		1	3(y)	
3. IRM Downscale	5/125 fullscale(g)	X		X		2	3	
4. APRM Upscale	**	X(s)		X	X	2	3(c)	
5. APRM Downscale	2/150 fullscale				X	2	3(c)	
6. IRM Upscale	108/125 fullscale	X		X		2	3	
7. Scram Discharge Volume a) Water level high	18 gallons	X(z)		X(z)	X(z)	1	1	
L. <u>Condenser Vacuum Pump Isolation</u>								Insert control rods
1. High Radiation in Main Steam Tunnel	10 x Normal Background			During Startup and run when vacuum pump 1 operating		2	2	
M. <u>Diesel Generator Load Sequence Timers</u>	Time delay after energiz. of relay							
1. Containment Spray Pump	40 acc \pm 15%	X	X	X	X	2(m)	1(n)	Consider containment spray loop inoperable and comply with spec. 3.4.C (see Note Q)

TABLE 3.1.1 (Cont'd)

- v. These functions not required to be operable when the ADS is not required to be operable.
- w. These functions must be operable only when irradiated fuel is in the fuel pool or reactor vessel and secondary containment integrity is required per specification 3.5.B.
- y. The number of operable channels may be reduced to 2 per Specification 3.9-E and F.
- z. The bypass function to permit scram reset in the shutdown or refuel mode with control rod block must be operable in this mode.
- aa. Pump circuit breakers will be tripped in 10 seconds \pm 15% during a LOCA by relays SK7A and SK8A.
- bb. Pump circuit breakers will trip instantaneously during a LOCA.

TABLE 4.1.1 (cont'd)

4.1-6

	<u>Instrument Channel</u>	<u>Check</u>	<u>Calibrate</u>	<u>Test</u>	<u>Remarks (applies to Test & Calibration)</u>
14.	High Radiation in Reactor Building				
	Operating Floor	1/s	1/3 mo	1/wk	Using gamma source for calibration
	Ventilation Exhaust	1/s	1/3 mo	1/wk	Using gamma source for calibration
15.	High Radiation on Air Ejector Off-Gas	1/s	1/3 mo	1/wk	Using built-in calibration equipment
16.	IRM Level	N A	each shutdown	N A	During approach to shutdown only
	IRM Scram	*	*	*	Using built-in calibration equipment
17.	IRM Blocks	N A	Prior to startup and shutdown	Prior to startup & shutdown	Upscale and downscale
18.	Condenser Low Vacuum	N A	Each refueling outage	Each refueling outage	

*Calibrate prior to startup and normal shutdown and thereafter check 1/s and test 1/wk until no longer required.

Legend:

NA = Not applicable; 1/s = Once per shift; 1/d = Once per day; 1/3d = Once per 3 days; 1/wk = Once per week; 1/3 mo = Once every 3 months.

<u>Instrument Channel</u>	<u>Check</u>	<u>Calibrate</u>	<u>Test</u>	<u>Remarks (Applies to Test & Calibration)</u>
19. Manual Scram Buttons	N A	NA	1/3 mo	
20. High Temperature Main Steamline Tunnel	N A	Each refueling outage	Each refueling outage	Using heat source box
21. SRM	*	*	*	Using built-in calibration equipment
22. Isolation Condenser High Flow ΔP (Steam and Water)	N A	1/3 mo	1/3 mo	By application of test pressure
23. Turbine Trip Scram	N A		Every 3 months	
24. Generator Load Rejection Scram	N A	Every 3 months	Every 3 months	
25. Recirculation Loop Flow	N A	Each Refueling Outage	NA	By application of test pressure
26. Low Reactor Pressure Core Spray Valve Permissive	N A	Every 3 months	Every 3 months	By application of test pressure
27. Scram Discharge Volume (Rod Block)				
a) Water level high	N A	Each Refueling Outage	Every 3 months	By varying level in switch column.
b) Scram trip bypass	N A	N A	Each refueling outage	

*Calibrate prior to startup and normal shutdown and thereafter check 1/s and test 1/wk until no longer required.

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- F. At specific power operation conditions, the actual control rod configuration will be compared with the expected configuration based upon appropriately corrected past data. This comparison shall be made every equivalent full power month. The initial rod inventory measurement performed when equilibrium conditions are established after a refueling or major core alteration will be used as base data for reactivity monitoring during subsequent power operation throughout the fuel cycle.
- G. At power operating conditions, the actual control rod density will be compared with the 3.5 percent control rod density included in Specification 3.2.B.6. This comparison shall be made every equivalent full power month.
- H. The scram discharge volume drain and vent valves shall be verified open at least once per 31 days, except in shutdown mode*, and shall be cycled at least one complete cycle of full travel at least quarterly.
- I. All withdrawn control rods shall be determined OPERABLE by demonstrating the scram discharge volume drain and vent valves OPERABLE. This will be done at least once per refueling cycle by placing the mode switch in shutdown and by verifying that:
- a. The drain and vent valves close within 60 seconds after receipt of a signal for control rods to scram, and
 - b. The scram signal can be reset and the drain and vent valves open when the scram discharge volume trip is bypassed.

Basis: The core reactivity limitation (Specification 3.2.A) requires that core reactivity be limited such that the core could be made subcritical at any time during the operating cycle, with the strongest operable control rod fully withdrawn and all other operable rods fully inserted. Compliance with this requirement can be demonstrated conveniently only at the time of refueling. Therefore, the demonstration must be such that it will apply to the entire subsequent fuel cycle. The demonstration is performed with the reactor core in the cold, xenon-free condition and will show that the reactor is sub-critical at that time by at least $R+0.25\% \Delta K$ with the highest worth operable control rod fully withdrawn.

*These valves may be closed intermittently for testing under administrative control.