MOKIPHEAST BUILDING

General Offices . Selden Street, Berlin, Connecticut

P.O. BOX 270 HARTFORD, CONNECTICUT 06101 (203) 666-6911

October 15, 1982 MP-4229

Mr. Ronald C. Haynes Regional Administrator, Region I Office of Inspection and Enforcement U. S. Nuclear Regulatory Commission 631 Park Avenue King of Prussia, Pennsylvania 19406

Reference:

Provisional License DPR-21

Docket No. 50-245

Reportable Occurrence RO-82-19/1T

Dear Mr. Haynes:

This letter forwards the Licensee Event Report for Reportable Occurrence RO-82-19/1T, required to be submitted within 14 days pursuant to the requirements of the Millstone Unit 1 Technical Specifications, Section 6.9.1.8.a. This report was originally submitted under an incorrect Reportable Occurrence Number 82-20/1T. An additional three copies of the report are enclosed.

Yours truly,

NORTHEAST NUCLEAR ENERGY COMPANY

Canell for E. J. Mroczka for Station Superintendent Millstone Nuclear Power Station

EJM/TST:mo

Attachment:

LER RO-82-19/1T

cc: Director, Office of Inspection and Enforcement, Washington, D.C. (30)

Director, Office of Management Information and Program Control,

Washington, D. C. (3)

U.S. Nuclear Regulatory Commission, c/o Document Management Branch,

Washington, D.C. 20555

ATTACHMENT TO LER 82-19/1T NORTHEAST NUCLEAR ENERGY COMPANY MILLSTONE NUCLEAR POWER STATION - UNIT 1 PROVISIONAL LICENSE NUMBER DPR-21 DOCKET NUMBER 50-245

Identification of Occurrence

Failure of a safety system to initiate the required protective function at the setpoint specified in the Technical Specifications occurred when the Main Steam Relief Valves failed to lift at the required test pressure.

Conditions Prior to Occurrence

Prior to occurrence the unit was shutdown for a refueling outage.

Description of Occurrence

On September 28, 1982, at 0930 hours, while performing a setpoint bench test on three safety relief valves, all three valves failed to open at the required setpoint pressure. The bench tests on two valves were terminated when they reached 103 percent of setpoint pressure to facilitate diagnostic testing by General Electric. The third valve opened at 104 percent of setpoint pressure. With concern over the condition of the remaining three valves, the valves were removed for testing. Two of these three valves tested failed to open at the required test pressure. The bench tests were again terminated when they reached 103 percent of setpoint pressure. The last valve is scheduled to be tested.

Apparent Cause of Occurrence

The cause of the setpoint drift is currently unknown. Following a complete investigation by General Electric, the plant will submit additional information on the cause of the failures.

Analysis of Occurrence

The six main steam safety relief valves serve to reduce reactor pressure at specified setpoints above normal operating pressure. The first valve is set to relieve at 1095#, the second at 1110# and the remaining four at 1125#.

The actual pressure the valves would have relieved at is unknown at this time. Further diagnostic testing is required to reveal the actual lifting pressures. A detailed analysis and impact on the plant will be evaluated when this data is available.

Corrective Action

Two of the three valves in the first set tested will be replaced with generic valve assemblies or rebuilt. The third valve has been reworked and retested.

All failures from the second set of valves will be reworked and retested.

Additional information and possible corrective action will be submitted at a later date.