



GULF STATES UTILITIES COMPANY

RIVER BEND STATION POST OFFICE BOX 220 ST. FRANCISVILLE, LOUISIANA 70770

AREA CODE 504 635-8394 348-8651

December 4, 1990
REG-34117
File Nos. G9.5, G9.25.1.3

U.S. Nuclear Regulatory Commission
Document Control Desk
Washington, D.C. 20555

Gentlemen:

River Bend Station - Unit 1
Docket No. 50-458

Please find enclosed Licensee Event Report No. 90-036 for River Bend Station - Unit 1. This report is being submitted pursuant to 10CFR50.73.

Sincerely,

W. H. Odell

W. H. Odell
Manager-Oversight
River Bend Nuclear Group

LAJ DRD JMC
LAE/PDG/DRD/DCH/JMC/pg

cc: U.S. Nuclear Regulatory Commission
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Arlington, TX 76011

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7800 Shoal Creek Blvd., Suite 400 North
Austin, TX 78757

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LICENSEE EVENT REPORT (LER)

ESTIMATED BURDEN PER RESPONSE TO COMPLY WITH THIS INFORMATION COLLECTION REQUEST: 50.0 HRS. FORWARD COMMENTS REGARDING BURDEN ESTIMATES TO THE RECORDS AND REPORTS MANAGEMENT BRANCH (F-630), U.S. NUCLEAR REGULATORY COMMISSION, WASHINGTON, DC 20555. AND TO THE PAPERWORK REDUCTION PROJECT (3150-0104), OFFICE OF MANAGEMENT AND BUDGET, WASHINGTON, DC 20503.

FACILITY NAME (1) RIVER BEND STATION		DOCKET NUMBER (2) 0 5 0 0 0 4 5 8	PAGE (3) 1 OF 0 2
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TITLE (4) Division I Balance-of-Plant Isolation During Modifications to Wiring in a Control Room Panel

EVENT DATE (5)			LER NUMBER (6)			REPORT DATE (7)			OTHER FACILITIES INVOLVED (8)		
MONTH	DAY	YEAR	YEAR	SEQUENTIAL NUMBER	REVISION NUMBER	MONTH	DAY	YEAR	FACILITY NAMES		DOCKET NUMBER(S)
11	04	90	90	036	00	12	04	90			0 5 0 0 0
											0 5 0 0 0

OPERATING MODE (9) 5

POWER LEVEL (10) 10

THIS REPORT IS SUBMITTED PURSUANT TO THE REQUIREMENTS OF 10 CFR 5 (Check one or more of the following) (11)

20.402(b)	20.406(e)	<input checked="" type="checkbox"/>	50.73(a)(2)(iv)	73.71(b)
20.406(a)(1)(i)	50.36(a)(1)	<input type="checkbox"/>	50.73(a)(2)(iv)	73.71(c)
20.406(a)(1)(ii)	50.36(a)(2)	<input type="checkbox"/>	50.73(a)(2)(v)	OTHER (Specify in Abstract below and in Text, NRC Form 366A)
20.406(a)(1)(iii)	50.73(a)(2)(i)	<input type="checkbox"/>	50.73(a)(2)(viii)(A)	
20.406(a)(1)(iv)	50.73(a)(2)(ii)	<input type="checkbox"/>	50.73(a)(2)(viii)(B)	
20.406(a)(1)(v)	50.73(a)(2)(iii)	<input type="checkbox"/>	50.73(a)(2)(i)	

LICENSEE CONTACT FOR THIS LER (12)

NAME	TELEPHONE NUMBER
L. A. England, Director - Nuclear Licensing	5 0 4 3 8 1 - 4 1 4 5

COMPLETE ONE LINE FOR EACH COMPONENT FAILURE DESCRIBED IN THIS REPORT (13)

CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NPRDS	CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NPRDS

SUPPLEMENTAL REPORT EXPECTED (14)

YES (If yes, complete EXPECTED SUBMISSION DATE) NO

EXPECTED SUBMISSION DATE (15)

MONTH	DAY	YEAR

ABSTRACT (Limit to 1400 spaces, i.e., approximately fifteen single-space typewritten lines) (16)

At 0026 on 11/04/90 with the unit in Operational Condition 5 (Refueling), an unplanned Division I balance of plant (BOP) isolation occurred when an instrument jack plug in one of the control room panels was unplugged during implementation of a modification request (MR). This isolation caused an interruption of suction flow to the RHR pump running in the spent fuel pool cooling assist mode. This report is submitted pursuant to 10CFR50.73(a)(2)(iv) to document an unplanned engineered safety feature actuation.

System Engineering provided a list of possible isolations applicable to this MR. The Shift Supervisor reviewed this list and preventive measures were taken; however, he overlooked the BOP isolation and this resulted in the event. Training on this event will be provided in the requalification training for licensed operators. This training will be completed by March 31, 1991.

During this event, system flow was interrupted for approximately 6 minutes. Reactor vessel temperature did not change and there were no adverse effects on plant systems. Therefore, this event did not adversely affect the health and safety of the public.

LICENSEE EVENT REPORT (LER)
TEXT CONTINUATIONESTIMATED BURDEN PER RESPONSE TO COMPLY WITH THIS
INFORMATION COLLECTION REQUEST: 602 HRS. FORWARD
COMMENTS REGARDING BURDEN ESTIMATE TO THE RECORDS
AND REPORTS MANAGEMENT BRANCH (F-630), U.S. NUCLEAR
REGULATORY COMMISSION, WASHINGTON, DC 20555, AND TO
THE PAPERWORK REDUCTION PROJECT (3150-0104), OFFICE
OF MANAGEMENT AND BUDGET, WASHINGTON, DC 20503.

FACILITY NAME (1) RIVER BEND STATION	DOCKET NUMBER (2) 0 5 0 0 0 4 5 8	LER NUMBER (6)			PAGE (3)	
		YEAR	SEQUENTIAL NUMBER	REVISION NUMBER		
		9 0	- 0 3 6	- 0 0	0 2	OF 0 2

TEXT (if more space is required, use additional NRC Form 386A's) (17)

REPORTED CONDITION

At 0026 on 11/04/90 with the unit in Operational Condition 5 (Refueling), an unplanned Division I Balance of Plant (BOP) isolation occurred when an instrument jack plug in one of the control room panels (*PL*) was unplugged during implementation of a modification request (MR). The closure of a motor operated isolation valve (*ISV*), in the spent fuel pool cooling system (*DA*), caused interruption of suction flow to the RHR pump running in the spent fuel pool cooling assist mode. The jack plug was restored, the isolation signal was reset and the isolation valve (*ISV*) opened to restore system flow. This report is submitted pursuant to 10CFR50.73(a)(2)(iv) to document an unplanned engineered safety feature (ESF) actuation.

INVESTIGATION

This event occurred during the implementation of a modification in one of the control room panels. The jack plug was removed in order to terminate leads. System engineering personnel reviewed the affected system isolation circuits prior to the start of work and created a list of possible isolations. The Shift Supervisor reviewed this list and preventive measures were taken; however, he overloo'ed the BOP isolation and this resulted in the event.

A review of previous reports revealed no similar LERs.

CORRECTIVE ACTION

Training on this event will be provided in the requalification training for licensed operators. This training will be completed by March 31, 1991.

SAFETY ASSESSMENT

Following restoration of the jack plug, the isolation signal was reset and the isolation valve was opened to restore system flow. System flow was interrupted for approximately 6 minutes. Reactor vessel temperature did not change and there were no adverse effects on plant systems. Therefore, this event did not adversely affect the health and safety of the public.

NOTE: Energy Industry Identification System Codes are identified in the text as (*XX*).