



**GPU Nuclear Corporation**  
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Writer's Direct Dial Number:

C321-93-2376  
April 19, 1994

U.S. Nuclear Regulatory Commission  
Attn: Document Control Desk  
Washington, DC 20555

Gentlemen:

Subject: Oyster Creek Nuclear Generating Station  
Docket No. 50-219  
Facility Operating License No. DPR-16  
Technical Specification Change Request No. 213  
Reactor Pressure Vessel Testing - EMRV Bypass

Pursuant to 10 CFR 50.90, GPU Nuclear Corporation (GPUN), operator of the Oyster Creek Nuclear Generating Station (OCNGS), Facility Operating License No. DPR-16, requests a change to Appendix A of that license.

The attached Technical Specification Change Request (TSCR) proposes updating and clarification of Technical Specification 3.4.B.1 to be consistent with existing Specifications 1.39 and 4.3.D (ASME Code Section XI, Article 5000 requirements). This TSCR deletes reference to the ASME Code Section XI, IS-500 ten year hydrotest inspection interval and replaces this with references to (1) the Technical Specification 1.39 definition for Reactor Vessel Pressure Testing and (2) the Technical Specification 3.3.A.(i) Reactor Vessel Pressure Testing limits (P/T and 250° F maximum test temperature).

This TSCR will clarify that the five electromatic relief valves' (EMRV) pressure relief function may be inoperable or bypassed during system pressure testing required by ASME Code Section XI, Article IWA-5000, including system leakage and hydrostatic test, with reactor vessel completely solid, core not critical and Technical Specification 3.2.A (Core Reactivity Limits) satisfied.

We request the NRC staff to place an appropriate priority for a timely review of the proposed Technical Specification change, such that this clarification will be in place for the 15R outage.

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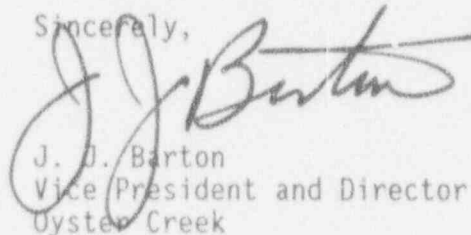
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This change request has been reviewed in accordance with Section 6.5 of the OCNGS Technical Specifications, and using the standards in 10 CFR 50.92 we have concluded that these proposed changes do not constitute a significant hazards consideration.

Pursuant to 10 CFR 50.91 (b) (1), a copy of this change request has been sent to the State of New Jersey Department of Environmental Protection.

Sincerely,

A handwritten signature in cursive script, appearing to read "J. J. Barton".

J. J. Barton  
Vice President and Director  
Oyster Creek

cc: Administrator, Region 1  
NRC Senior Resident Inspector, Oyster Creek  
Oyster Creek NRC Project Manager

UNITED STATES OF AMERICA  
NUCLEAR REGULATORY COMMISSION

In the Matter of )  
GPU Nuclear Corporation )

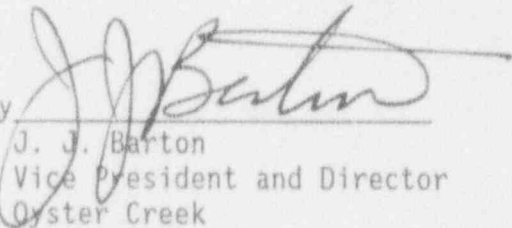
Docket No. 50-219

CERTIFICATE OF SERVICE

This is to certify that a copy of Technical Specification Change Request No. 213, for Oyster Creek Nuclear Generating Station Technical Specifications, filed with the U.S. Nuclear Regulatory Commission on April 19, 1994, has this day of 4/19, 1994, been served on the Mayor of Lacey Township, Ocean County, New Jersey by deposit in the United States mail, addressed as follows:

The Honorable Theodore Hutler, Jr.  
Mayor of Lacey Township  
818 West Lacey Road  
Forked River, NJ 08731

By

  
J. J. Barton  
Vice President and Director  
Oyster Creek