VIRGINIA ELECTRIC AND POWER COMPANY RICHMOND, VIRGINIA 28261

December 6, 1990

U.S. Nuclear Regulatory Commission Attention: Document Control Desk Washington, D.C. 20555 Serial No. 90-745 NAPS/RCS/rcs Docket No. 50-339 License No. NPF-7

Gentlemen:

VIRGINIA ELECTRIC AND POWER COMPANY NORTH ANNA POWER STATION UNIT 2 DIFFERENTIAL SETTLEMENT BETWEEN THE UNIT 2 CONTAINMENT AND THE UNIT 2 MAIN STEAM VALVE HOUSE

On November 2, 1990, the differential settlement between the Unit 2 containment (survey point 131) and the Unit 2 main steam valve house (survey point 124), specified in Table 3.7-5 of Technical Specification 3.7.12.1, was confirmed to have exceeded 75 percent of the allowable value of 0.030 feet. The differential settlement between survey points 131 and 124, calculated during the performance of Periodic Test I-PT-112, "Category I Structures - Settlement Monitoring", on November 2, 1990 was 0.025 feet, or 83 percent of 0.030 feet. There is no difference in the differential settlement between points 131 and 124 measured on November 2, 1990 and that measured on May 30, 1990, the date of the last previous performance of 1-PT-112. This special report is submitted pursuant to the requirements of Technical Specification 3.7.12.1.

The bases for monitoring the differential settlement between the Unit 2 containment and the Unit 2 main steam valve house are the stresses placed on the main steam, feedwater, service water, and condenser air ejector piping. These lines possess adequate flexibility to absorb 0.030 feet of differential settlement as described in earlier special reports, Serial No. 09-540, submitted July 28, 1989 and Serial No. 90-385, submitted June 29, 1990. Both the July 1989 submittal and the June 1990 submittals were the result of survey points 131 and 124 exceeding 75 percent of 0.030 feet differential settlement based on results of differential settlement surveys conducted in May 1989 and May 1990, respectively.

On October 2, 1989, Virginia Electric and Power Company submitted a technical specification change request (Serial No. 89-682) which proposed the deletion of the requirement to monitor survey points 131 and 124 along with other rock founded structures. Technical Report, "Evaluation of Settlement Monitoring Program for Main Plant Structures in Support of Proposed Changes to Technical Specification 3/4.7.12," which was Attachment 4 to this October 1989 submittal, provides the technical basis for deletion of the requirement to monitor these and other points.

9012110234 901206 PDR ADDCK 05000339 8 PDC TE22 AUT Based on the above, the structural integrity of the piping represented by points 131 and 124 has not been compromised. This special report has been reviewed by the Station Nuclear Safety and Operating Committee.

Should you have any questions regarding this report, please contact us.

Very truly yours

W. L. Stewart

Senior Vice President - Nuclear

cc: U.S. Nuclear Regulatory Commission

Region II

101 Marietta Street, N.W.

Suite 2900

Atlanta, Georgia 30323

Mr. M. S. Lesser

NRC Senior Resident Inspector

North Anna Power Station